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Docket No. 50-302

Mr. J. A. Hancock  
 Director, Nuclear Operations  
 Florida Power Corporation  
 P. O. Box 14042, Mail Stop C-4  
 St. Petersburg, Florida 33733

Dear Mr. Hancock:

The Commission has issued the enclosed Amendment No. 27 to Facility Operating License No. DPR-72 for the Crystal River Unit No. 3 Nuclear Generating Plant in partial response to your request of January 11, 1980 (Technical Specification (TS) Change Request No. 57).

The amendment modifies the TS to exclude the pressurizer steam space sampling line valve, CAV-1, from the provisions of TS 3.0.4 until startup from the next refueling outage (Cycle 3) in order to allow time for repairs to CAV-1.

You have committed to abide by administrative controls when taking pressurizer steam space samples until CAV-1 is restored to service. We request that you notify the Commission within 45 days of the date when CAV-1 is functional, so that we will have the option to delete the then obsolete change as part of a subsequent amendment.

The portions of TS Change Request No. 57 not included in this action will be the subject of future correspondence.

Copies of the Safety Evaluation and Notice of Issuance are also enclosed.

Sincerely,

Original signed by  
 Robert W. Reid  
 Robert W. Reid, Chief  
 Operating Reactors Branch #4  
 Division of Operating Reactors

**Enclosures:**

1. Amendment No. 27 to DPR-72
2. Safety Evaluation
3. Notice

cc w/enclosures: See next page

STS *D.B.*  
 DBriakman  
 1/22/80

OELDA *H.K.*  
 S H...  
 1/23/80

*CP*  
*50*

OFFICE	ORB#4: DOR	ORB#4: DOR	ORB#4: DOR	C-ORB#4: DOR	C-ORB#4: DOR	A-AD-ORP: DOR
SURNAME	RIngram <i>M</i>	SWookey/cb	MFairtile	RReid	GK... GK... GK...	WGammill
DATE	1/21/80	1/22/80 <i>SW</i>	1/22/80	1/22/80	1/21/80	1/22/80

*1980*

**Florida Power Corporation**

cc w/enclosure(s):

Mr. S. A. Brandimore  
Vice President and General Counsel  
P. O. Box 14042  
St. Petersburg, Florida 33733

Mr. Wilbur Langely, Chairman  
Board of County Commissioners  
Citrus County  
Iverness, Florida 36250

U. S. Environmental Protection Agency  
Region IV Office  
ATTN: EIS COORDINATOR  
345 Courtland Street, N.E.  
Atlanta, Georgia 30308

Director, Technical Assessment  
Division  
Office of Radiation Programs  
(AW-459)  
U. S. Environmental Protection Agency  
Crystal Mall #2  
Arlington, Virginia 20460

Crystal River Public Library  
Crystal River, Florida 32629

Mr. J. Shreve  
The Public Counsel  
Room 4 Holland Bldg.  
Tallahassee, Florida 32304

Administrator  
Department of Environmental Regulation  
Power Plant Siting Section  
State of Florida  
Montgomery Building  
2562 Executive Center Circle, E.  
Tallahassee, Florida 32301

Attorney General  
Department of Legal Affairs  
The Capitol  
Tallahassee, Florida 32304

Mr. Robert B. Borsum  
Babcock & Wilcox  
Nuclear Power Generation Division  
Suite 420, 7735 Old Georgetown Road  
Bethesda, Maryland 20014

cc w/enclosures & incoming  
dtd: 1/11/80  
Bureau of Intergovernmental  
Relations  
660 Apalachee Parkway  
Tallahassee, Florida 32304



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

FLORIDA POWER CORPORATION

CITY OF ALACHUA

CITY OF BUSHNELL

CITY OF GAINESVILLE

CITY OF KISSIMMEE

CITY OF LEESBURG

CITY OF NEW SMYRNA BEACH AND UTILITIES COMMISSION, CITY OF NEW SMYRNA BEACH

CITY OF OCALA

ORLANDO UTILITIES COMMISSION AND CITY OF ORLANDO

SEBRING UTILITIES COMMISSION

SEMINOLE ELECTRIC COOPERATIVE, INC.

CITY OF TALLAHASSEE

DOCKET NO. 50-302

CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 27  
License No. DPR-72

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Florida Power Corporation, et al (the licensees) dated January 11, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

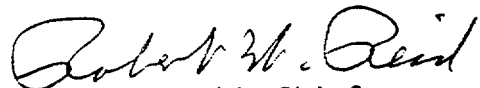
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-72 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 27, are hereby incorporated in the license. Florida Power Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief  
Operating Reactors Branch #4  
Division of Operating Reactors

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: January 23, 1980

ATTACHMENT TO LICENSE AMENDMENT NO. 27

FACILITY OPERATING LICENSE NO. DPR-72

DOCKET NO. 50-302

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages

3/4 6-17

3/4 6-21

TABLE 3.6-1  
CONTAINMENT ISOLATION VALVES

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME</u> (seconds)
A. CONTAINMENT ISOLATION		
1. BSV-27 check #	iso. dur. nor. operation	NA
BSV-3 #	and open dur. RB spray	60
BSV-26 check #		NA
BSV-4 #		60
2. CAV-126	iso. CA sys. fr. RC letdn.	60
CAV-1**	iso. CA sys. fr. pzs.	60
CAV-3		60
CAV-2	iso. CA sys.	60
CAV-4 #	isolate liquid sampling	60
CAV-6 #	system	60
CAV-5 #		60
CAV-7 #		60
3. CFV-20 check	iso. N <sub>2</sub> supply fr. CFT-1A	NA
CFV-28		60
CFV-17 check	iso. N <sub>2</sub> supply fr. CFT-1B	NA
CFV-27		60
CFV-18 check	iso. MU system fr. CFT-1B	NA
CFV-26		60
CFV-19 check	iso. MU system fr. CFT-1A	NA
CFV-25		60
CFV-42	iso. liquid sampling fr. CF system	60
CFV-15	iso. WD sys. fr. CF tanks	60
CFV-16		60
CFV-29		60
CFV-11	iso. CF tanks fr. liquid	60
CFV-12	sampling system	60
4. CIV-41	iso. CI sys. fr. RB	60
CIV-40		60
CIV-34		60
CIV-35		60

TABLE 3.6-1 (Continued)  
CONTAINMENT ISOLATION VALVES

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME</u> (seconds)
5. DHV-93 check CHV-91	iso. DH system fr. pwr.	NA 60
DHV-43 # DHV-42 #	iso. DH sys. fr. RB sump	120 120
DHV-4# & 41#	iso. DH sys. fr. RC	120
DHV-6 # DHV-5 #	iso. DH system from Reactor Vessel	60 60
6. DWV-162 check DWV-160	iso. system	NA 60
7. FWV-44 check # FWV-45 check #	iso. feedwater from RCSG-1A	NA NA
FWV-43 check # FWV-45 check #	iso. feedwater from RCSG-1B	NA NA
8. MSV-130 #	from RCSG-1A	60
MSV-148 #	from RCSG-1B	60
MSV-411 #	iso. main steam lines from RCSG-1A	60
MSV-412 #	iso. main steam lines from RCSG-1A	60
MSV-413 #	iso. main steam lines from RCSG-1B	60
MSV-414 #	iso. main steam lines from RCSG-1B	60
9. MUV-40 MUV-41 MUV-49 MUV-253	iso. MU system from RC	60 60 60 60
MUV-261 MUV-260 MUV-259 MUV-258	iso. MU system from control bleed-off	60 60 60 60

TABLE 3.6-1 (Continued)  
CONTAINMENT ISOLATION VALVES

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME</u> (seconds)
5. SAV-24	iso. SA from RB	NA
6. SFV-18 SFV-19	iso. SF system	NA NA
SFV-119# SFV-120#	iso. Fuel Transfer tubes from F.T. Canal	NA NA
7. WSV-1 WSV-2	containment monitoring system from RB	NA NA
D. PENETRATIONS REQUIRING TYPE B TESTS		
Blind Flange 119	iso. RB	NA
Blind Flange 120		NA
Blind Flange 116		NA
Blind Flange 202		NA
Blind Flange 348	iso. fuel transfer tube from	NA
Blind Flange 436	Transfer Canal	NA
Equipment Hatch	iso. RB	NA
Personnel Hatch	iso. RB	NA

# Not subject to Type C Leakage Test

\*\*The provisions of Specification 3.0.4 are not applicable until startup for Cycle 3 operation. Isolation valves closed to satisfy the requirements of Specification 3.6.3.1 ACTION b. and c. may be re-opened on an intermittent basis under administrative control for up to 4 hours in any 24 hour period as necessary for sampling.



## CONTAINMENT SYSTEMS

### 3/4.6.4 COMBUSTIBLE GAS CONTROL

#### HYDROGEN ANALYZERS

#### LIMITING CONDITION FOR OPERATION

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3.6.4.1 A containment hydrogen analyzer and a gas chromatograph shall be OPERABLE.

APPLICABILITY: MODES 1 and 2.

#### ACTION:

With one hydrogen analysis device inoperable, restore the inoperable device to OPERABLE status within 30 days or be in at least HOT STANDBY within the next 6 hours.

#### SURVEILLANCE REQUIREMENTS

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4.6.4.1 Each hydrogen analysis device shall be demonstrated OPERABLE at least once per 92 days on a STAGGERED TEST BASIS by performing a CHANNEL CALIBRATION using sample gases containing:

- a. One volume percent hydrogen, balance nitrogen, and
- b. Four volume percent hydrogen, balance nitrogen.

UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555



SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
SUPPORTING AMENDMENT NO. 27 TO FACILITY OPERATING LICENSE NO. DPR-72  
FLORIDA POWER CORPORATION, ET AL  
CRYSTAL RIVER UNIT NO. 3 NUCLEAR GENERATING PLANT  
DOCKET NO. 50-302

Introduction

By letter dated January 11, 1980, Florida Power Corporation (FPC) submitted Technical Specification (TS) Change Request No. 57. This change request proposed intermittent operation of select containment isolation valves for sampling or surveillance testing purposes and a qualification to the limiting condition for operation for specific containment isolation valves.

Background

Based upon discussions with FPC, it was determined that containment isolation valve CAV-1 is presently inoperable in the open position. As a result, we have restricted this evaluation of the January 11, 1980 submittal to the review of an exclusion of the valve CAV-1 from the provisions of TS 3.0.4 in order to expedite our review.

Discussion & Evaluation

Containment Isolation Valve CAV-1 is an electric motor operated isolation valve located inside the reactor building on the pressurizer steam space sampling line. In accordance with TS requirements, the manual valve upstream from CAV-1 has been closed since CAV-1 is inoperable in the open position. The pneumatic valve downstream from CAV-1, which is located outside the reactor building, is maintained in its normal closed position except during sampling periods. Therefore, double isolation of this sampling line is maintained.

We have reviewed the radiological consequences of a postulated accident considering potential releases from failure of the pressurizer steam space sampling line. We have concluded that the radiological consequences from this postulated accident are a small fraction of the dose guidelines of 10 CFR 100. Additional occupational exposure is received during the pressurizer steam space sampling procedure with the current valve's arrangement. Based upon the maximum estimated radiation dose of 10 mrem per sampling procedure, we have concluded that the additional occupational dose will be acceptable for the period of time until CAV-1 is restored to service.

We have reviewed the interim valving arrangement for the pressurizer steam space sampling line and have determined that the facility's administrative procedures are acceptable for the period of time until startup for Cycle 3. We approve of an exclusion in the TS for CAV-1 to allow time for the maintenance and repair of the valve.

### Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

### Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: January 23, 1980

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-302FLORIDA POWER CORPORATIONCITY OF ALACHUACITY OF BUSHNELLCITY OF GAINESVILLECITY OF KISSIMMEECITY OF LEESBURGCITY OF NEW SMYRNA BEACH AND UTILITIES COMMISSION, CITY OF NEW SMYRNA BEACHCITY OF OCALAORLANDO UTILITIES COMMISSION AND CITY OF ORLANDOSEBRING UTILITIES COMMISSIONSEMINOLE ELECTRIC COOPERATIVE, INC.CITY OF TALLAHASSEENOTICE OF ISSUANCE OF AMENDMENT TO FACILITY  
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 27 to Facility Operating License No. DPR-72, issued to the Florida Power Corporation, City of Alachua, City of Bushnell, City of Gainesville, City of Kissimmee, City of Leesburg, City of New Smyrna Beach and Utilities Commission, City of New Smyrna Beach, City of Ocala, Orlando Utilities Commission and City of Orlando, Sebring Utilities Commission, Seminole Electric Cooperative, Inc., and the City of Tallahassee (the licensees) which revised the Technical Specifications for operation for the Crystal River Unit No. 3 Nuclear Generating Plant (the facility) located in Citrus County, Florida. The amendment is effective as of the date of issuance.

This amendment revises the Technical Specifications to exclude the pressurizer steam space sampling line valve, CAV-1, from the provisions of TS 3.0.4 until startup from the next refueling outage (Cycle 3).

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appro-

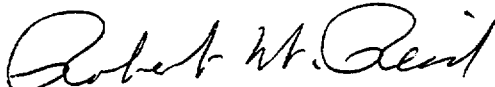
appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated January 11, 1980, (2) Amendment No. 27 to License No. DPR-72, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW, Washington, D. C., and at the Crystal River Public Library, Crystal River, Florida. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 23rd day of January, 1980.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Robert W. Reid, Chief  
Operating Reactors Branch #4  
Division of Operating Reactors