

September 16, 1986

DMB 016

Docket No. 50-302

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 JPartlow
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 WJones
 WRegan
 ACRS-10
 CMiles
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 BMOzafari
 HSilver
 Gray File
 NThompson

Mr. Walter S. Wilgus
 Vice President, Nuclear Operations
 Florida Power Corporation
 ATTN: Manager, Nuclear Licensing
 & Fuel Management
 P. O. Box 14042; M.A.C. H-3
 St. Petersburg, Florida 33733

Dear Mr. Wilgus:

The Commission has issued the enclosed Amendment No. 91 to Facility Operating License No. DPR-72 for the Crystal River Unit No. 3 Nuclear Generating Plant (CR-3). This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated April 23, 1984.

This amendment permits the operation of certain containment isolation valves when they would normally be required to be isolated, provided that a dedicated operator is posted to isolate the valve if necessary. The description of a dedicated operator was more fully defined in the note on Table 3.6-1 by mutual agreement.

As discussed in the enclosed Safety Evaluation, changes concerning steam generator shell side sampling valves (CAV-4, 5, 6 and 7) and the hydrogen purge system valves (LRV-70, 71, 72 and 73) are denied and therefore are not included in this amendment.

Notice of Issuance will be included in the Commission's biweekly Federal Register notice. A separate Notice of Denial of Amendment is being forwarded to the Federal Register for publication.

Sincerely,

ORIGINAL SIGNED BY

Brenda Mozafari, Project Manager
 PWR Project Directorate #6
 Division of PWR Licensing-B

Enclosures:

1. Amendment No. 91 to DPR-72
2. Safety Evaluation

cc w/enclosures:
 See next page

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 P PDR

*See previous white for concurrences.

PBD-6	PBD-6	PBD-6	PBD-6	PBD-6	PBD-6	OGC	PBPE-B
RIngram*	BMOzafari;jak*	HSilver*	RWeller*	GEdison	JStolz	MKarman*	JWermiel*
9/10/86	9/10/86	9/11/86	9/11/86	9/11/86	9/11/86	9/9/86	9/11/86

Concurrence made as requested + disclosed w/9/16/86.

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Sincerely,

Brenda Mozafari, Project Manager
PWR Project Directorate #6
Division of PWR Licensing-B

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- 2. Safety Evaluation

cc w/enclosures:
See next page

*See previous white for concurrences.

PBD-6 RIngram*	9/10/86	PBD-6 BMOzafari;jak*	9/10/86	PBD-6 HSilver*	7/1/86	PBD-6 RWeller*	7/1/86	PBD-6 GEdison*	7/3/86	PBD-6 JStolz*	7/2/86	OGC MKarman*	7/8/86	PBPE-B JWermiel	9/11/86
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Docket No. 50-302

DISTRIBUTION

Mr. Walter S. Wilgus
Vice President, Nuclear Operations
Florida Power Corporation
ATTN: Manager, Nuclear Licensing
& Fuel Management
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As discussed in the enclosed Safety Evaluation, changes concerning steam generator shell side sampling valves (CAV-4, 5, 6 and 7) and the hydrogen purge system valves (LRV-70, 71, 72 and 73) are not denied and therefore are not included in this amendment.

Notice of Issuance will be included in the Commission's biweekly Federal Register notice. A separate Notice of Denial of Amendment is being forwarded to the Federal Register for publication.

Sincerely,

Brenda Mozafari, Project Manager
PWR Project Directorate #6
Division of PWR Licensing-B

Enclosures:

- 1. Amendment No. to DPR-72
- 2. Safety Evaluation

cc w/enclosures:
See next page

Raw PBD-6 RIngram 7/1/86
JEM PBD-6 BMOzafari;jak 6/30/86
[Signature] PBD-6 HSilver 7/1/86
Raw PBD-6 RWeller 7/1/86
[Signature] PBD-6 JSolz 7/2/86
OGC PBD-6 M. Karman 7/8/86
[Signature]

September 16, 1986

DOCKET NO. 50-302

DISTRIBUTION
Docket File
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RIngram

MEMORANDUM FOR: Rules and Procedures Branch
Division of Rules and Records
Office of Administration

FROM: Office of Nuclear Reactor Regulation

SUBJECT: CRYSTAL RIVER UNIT NO. 3 NUCLEAR GENERATING PLANT

One signed original of the *Federal Register* Notice identified below is enclosed for your transmittal to the Office of the Federal Register for publication. Additional conformed copies (5) of the Notice are enclosed for your use.

- Notice of Receipt of Application for Construction Permit(s) and Operating License(s).
- Notice of Receipt of Partial Application for Construction Permit(s) and Facility License(s); Time for Submission of Views on Antitrust Matters.
- Notice of Consideration of Issuance of Amendment to Facility Operating License.
- Notice of Receipt of Application for Facility License(s); Notice of Availability of Applicant's Environmental Report; and Notice of Consideration of Issuance of Facility License(s) and Notice of Opportunity for Hearing.
- Notice of Availability of NRC Draft/Final Environmental Statement.
- Notice of Limited Work Authorization.
- Notice of Availability of Safety Evaluation Report.
- Notice of Issuance of Construction Permit(s).
- Notice of Issuance of Facility Operating License(s) or Amendment(s).
- Order.
- Exemption.
- Notice of Granting Exemption.
- Environmental Assessment.
- Notice of Preparation of Environmental Assessment.

Other: Notice of Denial of Amendment to Facility Operating License and Opportunity for Hearing.

PWR Project Directorate #6
Office of Nuclear Reactor Regulation

Enclosure:
As stated

Contact:
Phone:

NOTE: *Please insert date on the 2nd page 4th paragraph of this notice for a 30-day intervention period, and call Julie on extension 22288 to inform her of the date inserted. Referenced documents have been provided PDR.

OFFICE									
SURNAME									RIngram PBD-6
DATE									9/17/86

Mr. W. S. Wilgus
Florida Power Corporation

Crystal River Unit No. 3 Nuclear
Generating Plant

cc:

Mr. R. W. Neiser
Senior Vice President
and General Counsel
Florida Power Corporation
P. O. Box 14042
St. Petersburg, Florida 33733

State Planning and Development
Clearinghouse
Office of Planning and Budget
Executive Office of the Governor
The Capitol Building
Tallahassee, Florida 32301

Mr. P. McKee
Nuclear Plant Manager
Florida Power Corporation
P. O. Box 219
Crystal River, Florida 32629

Mr. F. Alex Griffin, Chairman
Board of County Commissioners
Citrus County
110 North Apopka Avenue
Inverness, Florida 36250

Mr. Robert B. Borsum
Babcock & Wilcox
Nuclear Power Generation Division
Suite 220, 7910 Woodmont Avenue
Bethesda, Maryland 20814

Resident Inspector
U.S. Nuclear Regulatory Commission
Route #3, Box 717
Crystal River, Florida 32629

Regional Administrator, Region II
U.S. Nuclear Regulatory Commission
101 Marietta Street, Suite 3100
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Mr. Allan Schubert, Manager
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Tallahassee, Florida 32301

Administrator
Department of Environmental Regulation
Power Plant Siting Section
State of Florida
2600 Blair Stone Road
Tallahassee, Florida 32301

Attorney General
Department of Legal Affairs
The Capitol
Tallahassee, Florida 32304



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER CORPORATION
CITY OF ALACHUA
CITY OF BUSHNELL
CITY OF GAINESVILLE
CITY OF KISSIMMEE
CITY OF LEESBURG
CITY OF NEW SMYRNA BEACH AND UTILITIES COMMISSION, CITY OF NEW SMYRNA BEACH
CITY OF OCALA
ORLANDO UTILITIES COMMISSION AND CITY OF ORLANDO
SEBRING UTILITIES COMMISSION
SEMINOLE ELECTRIC COOPERATIVE, INC.
CITY OF TALLAHASSEE

DOCKET NO. 50-302

CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 91
License No. DPR-72

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power Corporation, et al. (the licensees) dated April 23, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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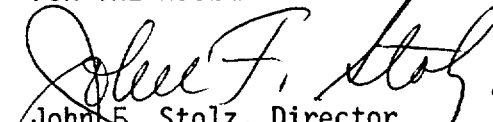
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-72 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 91, are hereby incorporated in the license. Florida Power Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


John E. Stolz, Director
PWR Project Directorate #6
Division of PWR Licensing-B

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 16, 1986

ATTACHMENT TO LICENSE AMENDMENT NO. 91

FACILITY OPERATING LICENSE NO. DPR-72

DOCKET NO. 50-302

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Remove

3/4 6-17
3/4 6-21
3/4 6-21a

Insert

3/4 6-17
3/4 6-21
3/4 6-21a

TABLE 3.6-1
CONTAINMENT ISOLATION VALVES

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME</u> (seconds)
A. CONTAINMENT ISOLATION		
1. BSV-27 check #	closed dur. nor. operation and open dur. RB spray	NA
BSV-3 #	"	60
BSV-26 check #	"	NA
BSV-4 #	"	60
2. CAV-126(A)**	iso. CA sys. fr. RC letdn.	60
CAV-1 (A)**	iso. CA sys. fr. pzzr.	60
CAV-3 (A)**	"	60
CAV-2 (B)**	iso. CA sys. fr. RB	60
CAV-4 # (A)*	isolate liquid sampling system	60
CAV-6 # (B)*	"	60
CAV-5 # (A)*	"	60
CAV-7 # (B)*	"	60
3. CFV-20 check	iso. N ₂ supply fr. CFT-1A	NA
CFV-28 (A/B)**	"	60
CFV-18 check	iso. MU system fr. CFT-1B	NA
CFV-26 (A/B)**	"	60
CFV-19 check	iso. MU system fr. CFT-1A	NA
CFV-25 (A/B)**	"	60
CFV-42 (B)*. +	iso. liquid sampling fr. CF system	60
CFV-15 (A)*	iso. WD sys. fr. CF tanks	60
CFV-16 (A)*	"	60
CFV-29 (B)*	"	60
CFV-11 (A)**	iso. CF tanks fr. liquid sampling system	60
CFV-12 (A)**	"	60
CFV-17 check	iso. N ₂ supply fr. CFT-1B	NA
CFV-27 (A/B)**	"	60

TABLE 3.6-1 (Continued)
CONTAINMENT ISOLATION VALVES

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME</u> (seconds)
4. CIV-41*	iso. CI sys. fr. RB	60
CIV-40*	"	60
CIV-34*	"	60
CIV-35*	"	60
5. DHV-93 check	iso. DH system fr. pzz.	NA
DHV-91*	"	60
DHV-43 #	iso. DH sys. fr. RB sump	120
DHV-42 #	"	120
DHV-4# & 41#	iso. DH sys. fr. RC	120
DHV-6 #	iso. DH system from Reactor Vessel	60
DHV-5 #	"	60
6. DWV-162 check	iso. DW system fr. RB	NA
DWV-160 (A/B)*	"	60
7. FWV-44 check #	iso. feedwater from RCSG-1A	NA
FWV-45 check #	"	NA
FWV-43 check #	iso. feedwater from RCSG-1B	NA
FWV-46 check #	"	NA
8. MSV-130 #(A/B)*	iso. MDT-1 from RCSG-1A	60
MSV-148 #(A/B)*	iso. MDT-1 from RCSG-1B	60
MSV-411 # *	iso. main steam lines from RCSG-1A	60
MSV-412 # *	iso. main steam lines from RCSG-1A	60
MSV-413 # *	iso. main steam lines from RCSG-1B	60
MSV-414 # *	iso. main steam lines from RCSG-1B	60

TABLE 3.6-1 (continued)

CONTAINMENT ISOLATION VALVES

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME</u> (seconds)
LRV-43	iso. LR test panel from RB	NA
LRV-44	"	NA
LRV-46	"	NA
LRV-87 & 88*	iso. H ₂ recombiner ports from RB	NA
LRV-89 & 90*	"	NA
LRV-91 & 92*	"	NA
LRV-93 & 94*	"	NA
3. MSV-146 #	iso. misc. waste storage tank from RCSG-1B	NA
MSV-128 #	"	NA
4. NGV-62	iso. NG system from steam generators	NA
NGV-81 #	"	NA
NGV-82	iso. NG system from pwr.	NA
5. SAV-24+	iso. SA from RB	NA
SAV-23+	"	NA
SAV-122	"	NA
6. SFV-18	iso. SF system	NA
SFV-19	"	NA
SFV-119#	iso. Fuel Transfer tubes from F. T. Canal	NA
SFV-120#	"	NA
7. WSV-1	iso. containment monitoring system from RB	NA
WSV-2	"	NA

D. PENETRATIONS REQUIRING TYPE B TESTS

Blind Flange 119	iso. RB	NA
Blind Flange 120	"	NA
Blind Flange 202	"	NA

TABLE 3.6-1 (Continued)
CONTAINMENT ISOLATION VALVES

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME</u> (seconds)
Blind Flange 348	iso. fuel transfer tube from Transfer Canal	NA
Blind Flange 436	"	NA
Equipment Hatch	iso. RB	NA
Personnel Hatch	iso. RB	NA

Not subject to Type C Leakage Test

* The provisions of Specification 3.0.4 are not applicable.

+ May be opened on an intermittent basis under administrative control consisting of a dedicated operator stationed at the valve in continuous communication with the control room with necessary parameters to rapidly isolate the penetration on a valid indication.

- (A) Isolates on Diverse Isolation Actuation Signal A
- (B) Isolates on Diverse Isolation Actuation Signal B
- (A/B) Isolates on Diverse Isolation Actuation Signal A or B



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 91 TO FACILITY OPERATING LICENSE NO. DPR-72
FLORIDA POWER CORPORATION, ET AL.
CRYSTAL RIVER UNIT NO. 3 NUCLEAR GENERATING PLANT
DOCKET NO. 50-302

INTRODUCTION

By letter dated April 23, 1984, Florida Power Corporation (FPC) proposed a change to the Crystal River Unit 3 (CR-3) Technical Specifications (TSs). This change would allow operation (opening) of certain containment isolation valves when they would normally be required to be isolated provided that a dedicated operator is posted to isolate the valve when necessary. A portion of the change request has been denied for reasons discussed in this evaluation.

BACKGROUND

NUREG-0103, Standard Technical Specifications for Babcock & Wilcox Pressurized Water Reactors, was issued to provide guidance to applicants in the development of plant specific TSs. One feature of NUREG-0103 is to allow certain valves listed in STS Table 3.6.2 of Specification 3.6.4, Containment Isolation Valves, to be opened on an intermittent basis under administrative control. The administrative controls, while not specified by NRC, are usually in the form of a procedure which directs the establishment of a dedicated operator in continuous direct communication with the control room to facilitate a rapid valve closure should the conditions warrant such action. During the development of the CR-3 TSs this provision was not incorporated. It is also noted that CR-3 Surveillance Requirement 4.6.1.1(a)(1) references the use of this provision as an exemption to the 31-day verification in that all penetrations not capable of being closed by OPERABLE containment automatic isolation valves and required to be closed during accident conditions are closed by valves, blind flanges, or deactivated automatic valves secured in their positions, except as provided in Table 3.6-1 of Specification 3.6.3.1. Selecting which valves are to be controlled in this manner requires suitable justification for opening an inoperable valve (penetration).

EVALUATION

The request to operate the following valves under administrative controls has been approved: CAV-1, 2, 3 and 126; CFV-25, 26, 27, 28, 11, 12 and 42; and SAV-23 and 24. The staff has denied the request to operate the following under administrative controls: CAV-4, 5, 6 and 7; and LRV-70, 71, 72 and 73.

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For valves CAV-1, 2, 3 and 126, it is determined that a change to allow opening under administrative controls is acceptable when Reactor Coolant System sampling would otherwise be prevented by an inoperable valve. It is concluded that the administrative controls should be clearly defined by adding the following note to TS Table 3.6-1:

May be opened on an intermittent basis under administrative control consisting of a dedicated operator stationed at the valve in continuous communication with the control room with necessary parameters to rapidly isolate the penetration on a valid indication.

Likewise, the staff determined it to be acceptable to operate valves CFV-25, 26, 11, 12 and 42 under administrative controls to assure adequate boron concentration when necessary. It is concluded that the administrative controls are sufficient to adequately control the valves.

Valves CFV-27 and 28, which supply nitrogen cover-pressure to the core flood tanks, are needed to maintain pressure in the Reactor Coolant System during an accident. The staff feels it is acceptable to operate these valves under the specified administrative controls.

For valves SAV-23 and 24, the staff determined that it is acceptable to operate the valves under administrative controls. Particular attention was given to the fact that both valves are manual type valves. These valves would be used during maintenance activities in the Reactor Building while in HOT STANDBY or HOT SHUTDOWN to provide air for respirators. Service air pressure leakage would be into containment and would provide suitable assurance that personnel could be evacuated and the penetration isolated without a release.

The request for a change to allow hydrogen purge system valves LRV-70, 71, 72 and 73 to be operated under administrative controls is denied. A system that provides containment purging during normal plant operations should meet the requirements of NUREG-0737, II.E.4.2, and its Attachment 1. The system proposed by the licensee is neither designed nor proposed to be operated in accordance with these criteria.

In addition, the request to operate valves CAV-4, 5, 6 and 7 under administrative controls is denied. The basis for the request was to resolve a conflict between the environmental TS requirement (2.4.1.0) to sample steam generator chemistry and the containment isolation TS which precludes opening the subject valves. With the implementation of Radiological Effluent Technical Specifications (RETS), the requirement of 2.4.1.0 no longer exists and thus the basis for change is not valid.

ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in

individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: September 16, 1986

Principal Contributors: T. Stetka, J. Rogge, V. Panciera, H. Silver and B. Mozafari.

FLORIDA POWER CORPORATION
CITY OF ALACHUA
CITY OF BUSHNELL
CITY OF GAINESVILLE
CITY OF KISSIMMEE
CITY OF LEESBURG
CITY OF NEW SMYRNA BEACH AND UTILITIES COMMISSION, CITY OF NEW SMYRNA BEACH
CITY OF Ocala
ORLANDO UTILITIES COMMISSION AND CITY OF ORLANDO
SEBRING UTILITIES COMMISSION
SEMINOLE ELECTRIC COOPERATIVE, INC.
CITY OF TALLAHASSEE

DOCKET NO. 50-302

CRYSTAL RIVER UNIT NO. 3 NUCLEAR GENERATING PLANT

NOTICE OF DENIAL OF AMENDMENT TO FACILITY OPERATING LICENSE
AND OPPORTUNITY FOR HEARING

The U.S. Nuclear Regulatory Commission (the Commission) has denied in part a request by Florida Power Corporation, et al. (the licensees) for an amendment to Facility Operating License No. DPR-72, issued to Florida Power Corporation for operation of the Crystal River Unit No. 3 Nuclear Generating Plant located in Crystal River, Florida. Notice of consideration of issuance of this amendment was published in the FEDERAL REGISTER on November 21, 1984 (49 FR 45948).

The amendment, as proposed in the justification for the change by the licensees, would permit the operation of certain containment isolation valves when they would normally be required to be isolated, provided that a dedicated operator is posted to isolate the valve, if necessary. Included in the licensees' proposed change were the valves CAV-4, 5, 6 and 7. The basis for the request for change to these valves was to resolve a conflict between the environmental TS requirement (2.4.1.0) to sample steam generator chemistry and the containment isolation TS which precludes opening the subject valves. With the implementation of the Radiological Effluent Technical Specifications (RETS), the requirement of 2.4.1.0 no longer exists. Thus, the Commission has determined the basis for change is no longer valid.

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Also, the Commission has determined that valves LRV-70, 71, 72 and 73 should not be opened under administrative control. The system proposed by the licensees (for containment purging during normal plant operations) does not meet the requirements of NUREG-0737, II.E.4.2, and its Attachment 1.

All other provisions of the amendment request have been approved by Amendment No. 91 dated September 16, 1986. Notice of Issuance of Amendment No. 91 will be published in the Commission's biweekly FEDERAL REGISTER notice.

The licensees were notified of the Commission's denial of the proposed Technical Specification changes by letter dated September 16, 1986.

By October 29, 1986, the licensees may demand a hearing with respect to the denial described above and any person whose interest may be affected by this proceeding may file a written petition for leave to intervene.

A request for hearing or petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Docketing and Service Branch, or may be delivered to the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C., by the above date.

A copy of any petitions should also be sent to the Office of the General Counsel-Bethesda, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, and to R. W. Neiser, Senior Vice President and General Counsel, Florida Power Corporation, P.O. Box 14042, St. Petersburg, Florida 33733, attorney for the licensees.

For further details with respect to this action, see (1) the application for amendment dated April 23, 1984, (2) the Commission's letter to Florida Power Corporation dated September 16, and (3) the Commission's Safety Evaluation

issued with Amendment No. 91 to DPR-72 dated September 16, which are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C., and at the Crystal River Public Library, 668 N.W. First Avenue, Crystal River, Florida. A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of PWR Licensing-B.

Dated at Bethesda, Maryland, this 16th day of September 1986.

FOR THE NUCLEAR REGULATORY COMMISSION



Gordon E. Edison, Acting Director
PWR Project Directorate #6
Division of PWR Licensing-B