

May 5, 1989

Docket No. 50-302

DISTRIBUTION
See attached sheet

Mr. W. S. Wilgus
Vice President, Nuclear Operations
Florida Power Corporation
ATTN: Manager, Nuclear Licensing
P. O. Box 219
Crystal River, Florida 32629

Dear Mr. Wilgus:

SUBJECT: CRYSTAL RIVER UNIT 3 - ISSUANCE OF AMENDMENT RE: ADMINISTRATIVE
CONTROLS AND SECURITY, EMERGENCY AND FIRE PROTECTION PLANS
(TAC NO. 49440)

The Commission has issued the enclosed Amendment No. 111 to Facility Operating License No. DPR-72 for the Crystal River Unit No. 3 Nuclear Generating Plant (CR-3). This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated April 14, 1983, as modified by your submittals dated December 13, 1983, July 25, 1984, January 24, 1986, October 16, 1986 and August 18, 1987.

This amendment clarifies portions of Section 6, Administrative Controls, and modifies the audit frequencies of the Security, Emergency, and Fire Protection plans to be consistent with 10 CFR 73.40(d). In addition, TS 6.9.1.6 has been modified to show the currently correct address for submittal of monthly reports.

It is further noted that the requested footnote to Figures 6.2-1 and 6.2-2 has not been approved, as discussed in the Safety Evaluation. As you know, the NRC staff has recently addressed (Generic Letter (GL) 88-06) the need for a license amendment prior to implementing a change in organizational structure. That Generic Letter permits removal of organizational charts from the TS, with appropriate changes to the administrative control requirements of the TS. In your letter dated April 27, 1989, you stated that you will proceed expeditiously to request an amendment removing the organization charts in accordance with GL 88-06. A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

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CP-1 cc

DATED: May 5, 1989

AMENDMENT NO. 111 TO FACILITY OPERATING LICENSE NO. DPR-72-CRYSTAL RIVER UNIT 3

Docket File

NRC & Local PDRs

PDII-2 Reading

S. Varga, 14/E/4

G. Lainas, 14/H/3

H. Berkow

D. Miller

H. Silver

G. Pick

OGC-WF

D. Hagan, 3302 MNBB

E. Jordan, 3302 MNBB

B. Grimes, 9/A/2

T. Meek(4), P1-137

Wanda Jones, P-130A

E. Butcher, 11/F/23

ACRS (10)

GPA/PA

ARM/LFMB

P. Frederickson, RII

cc: Plant Service list

Mr. W. S. Wilgus

- 2 -

May 5, 1989

The Commission has also filed the enclosed Notice of Denial of Portion of Application for Amendment to Facility Operating License with the Office of the Federal Register.

Sincerely,

Original signed by

Harley Silver, Project Manager
Project Directorate II-2
Division of Reactor Projects-I/II
Office of Nuclear Reactor Regulation

Enclosures:

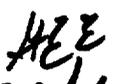
1. Amendment No. 111 to DPR-72
2. Safety Evaluation
3. Notice of Denial

cc w/enclosures:
See next page

[AMEND 49440]

LA/RDIA-2
DMS/Ver
05/2/89


PM:PDII-2
HSilver:bd
05/3/89


D:PDII-2
HBerkow
05/3/89


DGC
05/5/89

Mr. W. S. Wilgus
Florida Power Corporation

Crystal River Unit No. 3 Nuclear
Generating Plant

cc:

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Executive Office of the Governor
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Department of Legal Affairs
The Capitol
Tallahassee, Florida 32304



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER CORPORATION
CITY OF ALACHUA
CITY OF BUSHNELL
CITY OF GAINESVILLE
CITY OF KISSIMMEE
CITY OF LEESBURG
CITY OF NEW SMYRNA BEACH AND UTILITIES COMMISSION, CITY OF NEW SMYRNA BEACH
CITY OF OCALA
ORLANDO UTILITIES COMMISSION AND CITY OF ORLANDO
SEBRING UTILITIES COMMISSION
SEMINOLE ELECTRIC COOPERATIVE, INC.
CITY OF TALLAHASSEE

DOCKET NO. 50-302

CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 111
License No. DPR-72

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power Corporation, et al. (the licensees) dated April 14, 1983, as modified by your submittals dated December 13, 1983, July 25, 1984, January 24, 1986, October 16, 1986 and August 18, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-72 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 111, are hereby incorporated in the license. Florida Power Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

H E Edison for

Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects-I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 5, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 111

FACILITY OPERATING LICENSE NO. DPR-72

DOCKET NO. 50-302

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Remove

6-1
6-2
6-3
6-5
6-6
6-7
6-8
6-9
6-10
6-11
6-12
6-12a
6-15

Insert

6-1
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6-10
6-11
6-12
6-12a
6-15

ADMINISTRATIVE CONTROLS

- e. All CORE ALTERATIONS after the initial fuel loading shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
- f. A site Fire Brigade of at least 5 members shall be maintained onsite at all times.# The Fire Brigade shall not include 5 members of the minimum shift crew necessary for safe shutdown of the unit and any personnel required for other essential functions during a fire emergency.

Fire Brigade composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of Fire Brigade members provided immediate action is taken to restore the Fire Brigade to within the minimum requirements.

11-27-79

6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Director, Nuclear Plant Operations shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

6.1.2 The Director, Nuclear Plant Operations shall be responsible for an annual fire protection inspection which shall be performed utilizing either qualified offsite licensee personnel or an outside fire protection firm. The inspection shall consist of: a) an inspection of safety-related areas of the Plant to verify that they are in conformance with the fire hazards analysis; and b) a review of the Fire Brigade organization, training, and drills to verify their conformance with the requirements of Section 27 of the NFPA Code-1976.

6.1.3 The Director, Nuclear Plant Operations shall be responsible for an inspection of the fire protection program to be performed by a qualified outside fire consultant at least once per 36 months.

6.2 ORGANIZATION

OFFSITE

6.2.1 The offsite organization for facility management and technical support shall be as shown on Figure 6.2-1.

FACILITY STAFF

6.2.2 The Facility organization shall be as shown on Figure 6.2-2 and:

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b. At least one licensed Operator shall be in the control room when fuel is in the reactor.
- c. At least two licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.
- d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.

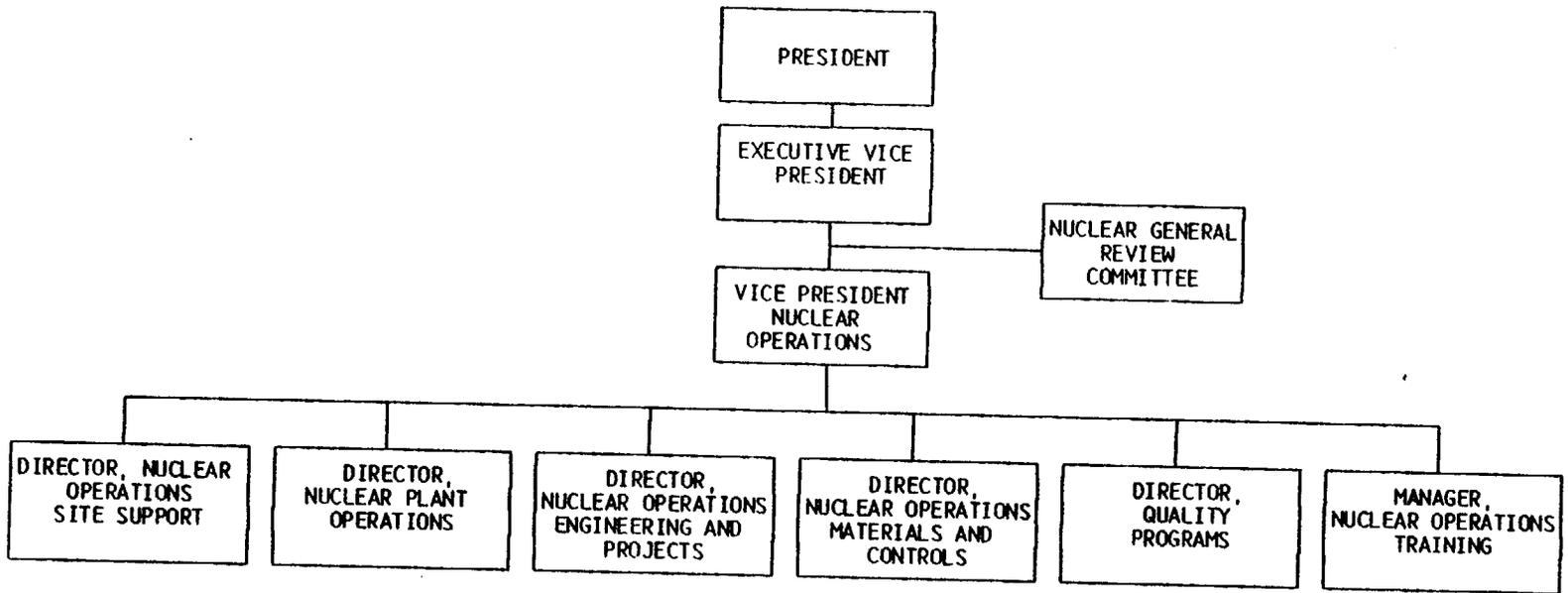
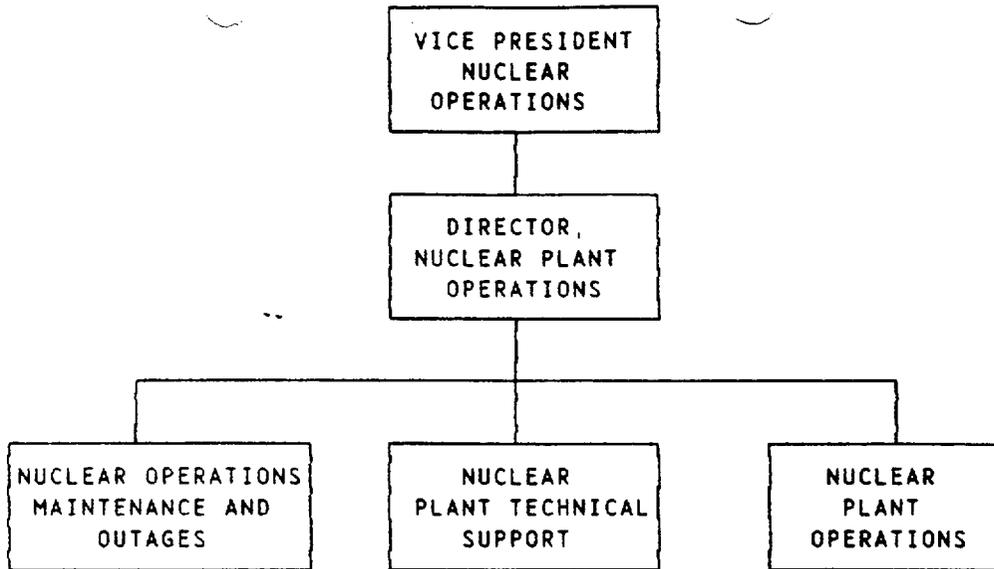


FIGURE 6.2-1
CORPORATE ORGANIZATION



- | | | |
|---|--|---|
| <ul style="list-style-type: none"> o MAINTENANCE - ELECTRICAL - MECHANICAL - INSTRUMENT & CONTROLS - BUILDING SERVICE o OUTAGE PLANNING & MODIFICATIONS | <ul style="list-style-type: none"> o INSERVICE INSPECTIONS o RELIABILITY & PERFORMANCE REVIEWS o OPERATING EXPERIENCE REVIEWS o NUCLEAR SECURITY | <ul style="list-style-type: none"> o OPERATIONS STAFF - SHIFT SUPERVISOR (SOL) - LICENSED OPERATORS (OL) - NON LICENSED OPERATORS o CHEMISTRY & RADIATION PROTECTION - HEALTH PHYSICS - CHEMISTRY CONTROL - WASTE MANAGEMENT o OPERATIONS PLANNING - TECHNICAL SPECIFICATION COORDINATION |
|---|--|---|

LEGEND: SOL - Senior Operator License
 OL - Operator License

FIGURE 6.2-2 - FACILITY ORGANIZATION

TABLE 6.2-1
MINIMUM SHIFT CREW COMPOSITION#

LICENSE CATEGORY	APPLICABLE MODES	
	1, 2, 3, & 4	5 & 6
SOL	2	1*
OL	2	1
Non-Licensed	3	1
Operations Tech. Advisor	1	0

*Does not include the licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling Individual supervising CORE ALTERATIONS after the initial fuel loading.

#Shift crew composition may be less than the minimum requirement for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements of Table 6.2-1.

ADMINISTRATIVE CONTROLS

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the Chemistry and Radiation Protection Superintendent who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975, and the Operations Technical Advisor, who shall have a Bachelor's degree, or the equivalent, in a scientific or engineering discipline with specific training in plant design and response and analysis of the plant for transients and accidents.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Vice President, Nuclear Operations and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and 10 CFR Part 55.

6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Vice President, Nuclear Operations and shall meet or exceed the requirements of Section 27 of the NFPA Code-1976, except for Fire Brigade training sessions which shall be held at least quarterly.

6.5 REVIEW AND AUDIT

6.5.1 PLANT REVIEW COMMITTEE (PRC)

FUNCTION

6.5.1.1 The Plant Review Committee shall function to advise the Director, Nuclear Plant Operations on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The Plant Review Committee shall be composed of the:

Chairman: Manager, Nuclear Plant Technical Support
Member: Nuclear Operations Superintendent
Member: Nuclear Maintenance Superintendent
Member: Asst. Mgr., Nuclear Plant Technical Support
Member: Nuclear Compliance Manager
Member: Nuclear Chemistry and Radiation Protection Supt.
Member: Mgr. Nuclear Mech./Struct. Engineering Services
Member: Mgr. Nuclear Elect./I&C Engineering Services
Member: Nuclear Safety and Reliability Superintendent
Member: At Large (Designated by Chairman)

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the PRC Chairman to serve on a temporary basis; no more than two alternates shall participate as voting members in PRC activities at any one time.

MEETING FREQUENCY

6.5.1.4 The PRC shall meet at least once per calendar month and as convened by the PRC Chairman or his designated alternate.

ADMINISTRATIVE CONTROLS

QUORUM

6.5.1.5 A quorum of the PRC shall consist of the Chairman or his designated alternate and five members including alternates.

RESPONSIBILITIES

6.5.1.6 The Plant Review Committee shall be responsible for:

- a. Review of 1) all procedures and changes thereto as required by Specification 6.8.2, 2) and other proposed procedures or changes thereto as determined by the Director, Nuclear Plant Operations to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to the Appendix "A" Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety, and changes to radwaste systems which could significantly alter their ability to meet Appendix I.
- e. Investigation of all violations of the Technical Specifications including the review and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Vice President, Nuclear Operations and to the Chairman of the Nuclear General Review Committee.
- f. Review of all REPORTABLE EVENTS.
- g. Review of facility operations to detect potential nuclear safety hazards.
- h. Performance of special reviews, investigations, or analyses and reports thereon as requested by the Chairman of the Nuclear General Review Committee.
- i. Review of the Plant Security Plan and implementing procedures.
- j. Review of the Emergency Plan and implementing procedures.

ADMINISTRATIVE CONTROLS

AUTHORITY

- 6.5.1.7 The Plant Review Committee shall:
- a. Recommend to the Director, Nuclear Plant Operations written approval or disapproval of items considered under 6.5.1.6 (a) through (d) above.
 - b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6 (a) through (e) above constitutes an unreviewed safety question.
 - c. Provide written notification within 24 hours to the Vice President, Nuclear Operations and the Nuclear General Review Committee of disagreement between the PRC and the Director, Nuclear Plant Operations; however, the Director, Nuclear Plant Operations shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

- 6.5.1.8 The Plant Review Committee shall maintain written minutes of each meeting and copies shall be provided to the Vice President, Nuclear Operations and Chairman of the Nuclear General Review Committee.

6.5.2 NUCLEAR GENERAL REVIEW COMMITTEE (NGRC)

FUNCTION

- 6.5.2.1 The Nuclear General Review Committee shall function to provide independent review and audit of designated activities in the areas of:
- a. Nuclear power plant operations
 - b. Nuclear engineering
 - c. Chemistry and radiochemistry
 - d. Metallurgy
 - e. Instrumentation and control
 - f. Radiological safety
 - g. Mechanical and electrical engineering
 - h. Quality assurance practices

ADMINISTRATIVE CONTROLS

COMPOSITION

6.5.2.2 The NGRC shall be composed of the Chairman, Vice Chairman, and at least 5 members. No more than a minority of the members shall have line responsibility for operation of the facility. The committee shall collectively have the experience and competence required to review problems in the following areas:

- a. Nuclear power plant operations
- b. Nuclear engineering
- c. Chemistry and radiochemistry
- d. Metallurgy
- e. Nondestructive testing
- f. Instrumentation and control
- g. Radiological safety
- h. Mechanical and electrical engineering
- i. Administrative controls
- j. Environmental
- k. Quality assurance practices

QUALIFICATIONS

6.5.2.3 The following minimum experience requirements shall be established for those persons involved in the independent off-site safety review and audit program:

- a. Chairman and Vice-Chairman-Bachelor of Science in engineering or related field or the equivalent and ten years related experience including five years involvement with operation and/or design of nuclear power plants.
- b. Member-Bachelor of Science in engineering or related field or the equivalent and five years related experience including three years involvement with operation and/or design of nuclear power plants.

ALTERNATES

6.5.2.4 All alternate members shall be appointed in writing by the NGRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in NGRC activities at any one time.

ADMINISTRATIVE CONTROLS

CONSULTANTS

6.5.2.5 Consultants shall be utilized as determined by the NGRC Chairman to provide expert advice to the NGRC.

MEETING FREQUENCY

6.5.2.6 The NGRC shall meet at least once per calendar quarter during the initial year of facility operation following fuel loading and at least once per six months thereafter.

QUORUM

6.5.2.7 A quorum of NGRC shall consist of the Chairman or his designated alternate and five additional NGRC members, including alternates. No more than a minority of the quorum shall have line responsibility for operation of the facility.

REVIEW

6.5.2.8 The NGRC shall review:

- a. Safety evaluations of changes to the facility, changes in procedures and tests and experiments not described in the Final Safety Analysis Report which were completed without prior NRC approval.
- b. Proposed changes in the facility, changes in procedures or proposed tests or experiments which may involve an unreviewed safety question.
- c. Proposed changes in the Technical Specifications or license amendments relating to nuclear safety prior to implementation, except in those cases where the change is identical to a previously reviewed proposed change.
- d. Violations, deviations and REPORTABLE EVENTS. This review should include results of investigations and recommendations resulting from such investigations.
- e. Any other matter involving safe operation of the plant which an independent reviewer deems appropriate for consideration, or which is referred to the reviewers by members of Nuclear Operations staff or by other functional organizational units within Florida Power Corporation.
- f. Reports and meeting minutes of the Plant Review Committee.

ADMINISTRATIVE CONTROLS

REVIEW (Continued)

- g. Changes to the PROCESS CONTROL PROGRAM and to the OFFSITE DOSE CALCULATION MANUAL.

AUDITS

6.5.2.9 Audits of facility activities shall be performed under the cognizance of the NGRC. These Audits shall encompass:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The performance, training and qualifications of the entire facility staff at least once per 12 months.
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per 24 months.
- e. The Facility Emergency Plan and implementing procedures at least once per 12 months.
- f. The Facility Security Plan and implementing procedures at least once per 12 months.
- g. The Facility fire protection program and implementing procedures at least once per 12 months.
- h. The radiological environmental monitoring program and the results thereof at least once per 12 months.

ADMINISTRATIVE CONTROLS

AUDITS (Continued)

- i. The OFFSITE DOSE CALCULATION MANUAL and implementing procedures at least once per 24 months.
- j. The PROCESS CONTROL PROGRAM and implementing procedures for solidification of radioactive wastes at least once per 24 months.
- k. The performance of activities required by the Quality Assurance Program for effluent and environmental monitoring at least once per 12 months.
- l. Any other area of facility operation considered appropriate by the NGRC or the Executive Vice President.

AUTHORITY

- 6.5.2.10 The NGRC shall report to and advise the Executive Vice President on those areas of responsibility specified in Sections 6.5.2.8 and 6.5.2.9.

RECORDS

- 6.5.2.11 Records of NGRC activities shall be prepared, approved and distributed as indicated below:
- a. Minutes of each NGRC meeting shall be prepared, approved and forwarded to the Executive Vice President within 14 days following each meeting.
 - b. Reports of reviews encompassed by Section 6.5.2.8 above, shall be forwarded to the Executive Vice President within 14 days following completion of the review.
 - c. Audit reports encompassed by Section 6.5.2.9 above, shall be forwarded to the Executive Vice President and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.6 REPORTABLE EVENT ACTION

- 6.6.1 The following actions shall be taken for REPORTABLE EVENTS:
- a. The Commission shall be notified and a report submitted pursuant to the requirements of Section 50.73 to 10 CFR 50, and
 - b. Each REPORTABLE EVENT shall be reviewed by the PRC and submitted to the NGRC and the Vice President, Nuclear Operations.

6.7 SAFETY LIMIT VIOLATION

- 6.7.1 The following actions shall be taken in the event a Safety Limit is violated:
- a. The facility shall be placed in at least HOT STANDBY within one hour.
 - b. The Safety Limit violation shall be reported to the Commission, the Vice President, Nuclear Operations, and to the NGRC within 24 hours.
 - c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PRC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures and (3) corrective action taken to prevent recurrence.
 - d. The Safety Limit Violation Report shall be submitted to the Commission, the NGRC and the Vice President, Nuclear Operations within 14 days of the violation. A separate Licensee Event Report need not be submitted if the Safety Limit Violation Report meets the requirements of 10 CFR 50.73 (b) in addition to the requirements above.

6.8 PROCEDURES

6.8.1 SCOPE

Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, November, 1972.
- b. Refueling operations.
- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.
- g. Systems Integrity Program implementation.
- h. Iodine Monitoring Program implementation.
- i. PROCESS CONTROL PROGRAM implementation.
- j. OFFSITE DOSE CALCULATION MANUAL implementation.
- k. Quality Assurance Program for effluent and environmental monitoring.

ADMINISTRATIVE CONTROLS

6.8 PROCEDURES (Continued)

6.8.2 REVIEW PROCESS

6.8.2.1 Each procedure and administrative policy of 6.8.1 above, and changes thereto, shall be reviewed and approved prior to implementation as follows:

- a. The Emergency Plan, Security Plan, Fire Protection Plan and implementing procedures, Administrative Instructions and those test procedures associated with plant modifications that affect nuclear safety shall be reviewed and approved by the PRC and the Director, Nuclear Plant Operations prior to implementation.
- b. For all other procedures, the review cycle shall consist of: an intradepartmental review by a Qualified Reviewer, and interdisciplinary review by Qualified Reviewer(s) in interfacing departments, as specified in administrative procedures, and approval by the responsible Superintendent or Manager, as specified by administrative procedures. The PRC shall then review the 10 CFR 50.59 evaluation within 14 days of approval.

6.8.2.2 The training and qualification of Qualified Reviewers shall be governed by administrative procedures, with final certification by the Director, Nuclear Plant Operations. Recertification will be required on a periodic basis and upon transfer between departments. As a minimum, all Qualified Reviewers shall meet the requirements of ANSI N18.1-1971, Sections 4.2, 4.3, 4.4, or 4.6, or the equivalent.

6.8.2.3 Each procedure and administrative policy of 6.8.1 shall be reviewed on a periodic basis as set forth in administrative procedures.

ADMINISTRATIVE CONTROLS

MONTHLY OPERATING REPORT

- 6.9.1.6 Routine reports of operating statistics and shutdown experience shall be submitted on a monthly basis to the Document Control Desk, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, with a copy to the Regional Office, submitted no later than the 15th of each month following the calendar month covered by the report.

ADMINISTRATIVE CONTROLS

DELETED



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 111 TO FACILITY OPERATING LICENSE NO. DPR-72
FLORIDA POWER CORPORATION, ET AL.
CRYSTAL RIVER UNIT NO. 3 NUCLEAR GENERATING PLANT
DOCKET NO. 50-302

INTRODUCTION

By letter dated April 14, 1983, as modified December 13, 1983, July 25, 1984, January 24 and October 16, 1986 and August 18, 1987, Florida Power Corporation (FPC or the licensee) requested an amendment to the Technical Specifications (TS) appended to Facility Operating License No. DPR-72 for the Crystal River Unit No. 3 Nuclear Generating Plant (CR-3). The proposed amendment would clarify portions of Section 6 of the TSs, and modify audit frequencies of the Security, Emergency, and Fire Protection plans to be consistent with 10 CFR 73.40(d). Also, in the December 13, 1983 letter, FPC proposed changes to TS Table 3.7-3, "Safety-Related Hydraulic Snubbers," which would have added six new safety-related hydraulic snubbers to the Table and clarify others. However, Amendment 88 to the license deleted this list from the TSs, and therefore, this change is not addressed in this evaluation.

SECTION 6.0 ADMINISTRATIVE CONTROLS

General

FPC proposed changes throughout this section in titles of various positions to agree with the revised organization charts, as well as renumbering certain sections. These changes are administrative in nature, reflect the position functions, and are acceptable.

PRESENT CORPORATE ORGANIZATIONAL STRUCTURE

Discussion and Evaluation

FPC proposed changes to Figure 6.2-1, "Corporate Organization," to indicate the present corporate organizational structure and to add a footnote stating that the corporate organization may be changed "without prior License Amendment provided that a revision to Figure 6.2-1 is included with the next License Amendment request."

Except for the footnote, the proposed change is administrative in nature, and is acceptable. The proposed footnote is not acceptable, even though the NRC staff does not disagree with its intent.

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The staff recognizes the addition of the footnote as an attempt by the licensee to address the problem of burdensome administrative effort and delays in implementation of desired organizational changes. However, the proposed footnote would permit unilateral licensee changes to the TS without prior NRC review and approval. This would violate the regulations. A more appropriate solution is presented in Generic Letter 88-06, which permits removal of organization charts from the TS with appropriate changes to the administrative control requirements of the TS. In FPC letter dated April 27, 1989, it was stated that FPC will proceed expeditiously to request an amendment removing the organization charts in accordance with Generic Letter 88-06. This will satisfactorily accomplish the intended goal of the proposed footnote.

FACILITY ORGANIZATION

Discussion and Evaluation

FPC proposed changes to Figure 6.2-2, "Facility Organization," to show the general responsibilities of each department rather than every position in each department and to add a footnote similar to that added to Figure 6.2-1.

Except for the footnote, the proposed changes are administrative in nature. Listing departmental responsibilities rather than the positions within each department will help clarify the responsibilities of each group and the inter-departmental relationships. The proposed changes are acceptable, except for the footnote, as discussed in Present Corporate Organizational Structure, above.

PLANT REVIEW COMMITTEE RESPONSIBILITIES

Discussion and Evaluation

FPC proposed a change to TS 6.5.1.6.e by replacing "preparation and forwarding of reports" with "review of reports."

Changing PRC responsibility to review TS violations clarifies the role of the PRC in the submittal process concerning TS violations and should continue to assure that proper evaluations and recommendations concerning these violations are performed. Therefore, the proposed change is acceptable.

NUCLEAR GENERAL REVIEW COMMITTEE, QUALIFICATIONS

Discussion and Evaluation

FPC proposed a change to TS 6.5.2.3, which would add "or the equivalent" after "Bachelor of Science in Engineering or related field" in both parts "a" and "b."

The proposed Nuclear General Review Committee (NGRC) membership qualifications will allow FPC to equate adequate experience to a degree; otherwise, highly qualified individuals may be excluded from NGRC membership. Including these non-degreed individuals with substantial expertise in the NGRC can potentially increase the effectiveness and quality of this committee. Therefore, the proposed change is acceptable.

NUCLEAR GENERAL REVIEW COMMITTEE, REVIEW REQUIREMENTS

Discussion and Evaluation

FPC proposed changes to TS 6.5.2.8, which would update the NGRC review requirements to concur with ANSI N18.7-1976, so that the TS accurately reflect the ANSI standard. ANSI N18.7-1976, which was endorsed by the NRC in Regulatory Guide 1.33, Revision 2, dated February 1978, upgraded their Quality Assurance program and FPC stated compliance to this standard in their FSAR. Therefore, this change is acceptable.

NUCLEAR GENERAL REVIEW COMMITTEE, AUDITS

Discussion and Evaluation

FPC proposed changes to TS 6.5.2.9, which would revise the Emergency Plan, Security Plan, and Fire Protection Plan auditing frequency from 24 months to 12 months. This change is required by 10 CFR 73.40(d). Therefore, this change is acceptable.

PROCEDURES

Discussion and Evaluation

FPC proposed changes to TS 6.8.2.1.a, which would require the PRC and the Director, Nuclear Plant Operations to review "those test procedures associated with plant modifications that affect nuclear safety" only, rather than those associated with all plant modifications as previously required. This change would be consistent with the scope of the TS, namely, matters dealing with nuclear safety. It would relieve the PRC of the burden of reviewing test procedures for modifications not affecting nuclear safety, which could detract from nuclear safety matters. In addition, the proposed change would be consistent with the responsibilities of the PRC as defined in TS 6.5.1.6. Therefore, we conclude that this change is acceptable.

MONTHLY OPERATING REPORT

Discussion and Evaluation

FPC proposed to update TS 6.9.1.6 to specify the correct NRC address for submittal of the required monthly reports. Since submittal of the proposed change, the NRC address has again been changed and the proposed change has been modified accordingly. This revision is acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment relates to changes in administrative requirements. The Commission has previously published a proposed finding that the amendment involves no significant hazards considerations and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that:
(1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: May 5, 1989

Principal Contributors:

G. Pick
H. Silver

UNITED STATES NUCLEAR REGULATORY COMMISSION
DENIAL OF PORTION OF APPLICATION FOR AMENDMENT
TO FACILITY OPERATING LICENSE
FLORIDA POWER CORPORATION, ET AL.
DOCKET NO. 50-302

The United States Nuclear Regulatory Commission (the Commission) has denied a portion of a request by Florida Power Corporation (the licensee) for an amendment to Facility Operating License No. DPR-72, issued to the licensee for operation of the Crystal River Unit 3 Nuclear Generating Plant (the facility), located in Citrus County, Florida.

The proposed amendment requested changes to Section 6, Administrative Controls, and requested modification of the audit frequencies for the Security, Emergency and Fire Protection plans to be consistent with 10 CFR 73.40(d). Notice of Consideration of Issuance of this amendment was published in the FEDERAL REGISTER on February 24, 1984 (49 FR 7035).

The licensee's application for amendment was dated April 14, 1983, as modified December 13, 1983, July 25, 1984, January 24, 1986, October 16, 1986 and August 18, 1987.

One of the requested changes to Section 6, Administrative Controls, involved the addition of a footnote to Figure 6.2-1, "Corporate Organization," and Figure 6.2-2, "Facility Organization," which would have stated that the organizations may be changed "without prior License Amendment provided that a revision to Figure 6.2-1 [and Figure 6.2-2] is included with the next License Amendment request." The staff determined that the proposed footnote was not acceptable, even though the staff does not disagree with its intent. The staff recognizes

the addition of the footnote as an attempt by the licensee to address the problem of burdensome administrative effort and delays in implementation of desired organizational changes. However, the proposed footnote would permit unilateral licensee changes to the Technical Specifications (TS) without prior NRC review and approval. This would violate the regulations. A more appropriate solution is presented in NRC Generic Letter 88-06, which permits removal of the organization charts from the TS with appropriate changes to the administrative control requirements of the TS.

By letter dated April 27, 1989, the licensee committed to proceed expeditiously to request an amendment removing the organization charts from the TS in accordance with Generic Letter 88-06.

The licensee was notified of the Commission's denial of this request by letter dated May 5, 1989.

By June 14, 1989, the licensee may demand a hearing with respect to the denial described above and any person whose interest may be affected by this proceeding may file a written petition for leave to intervene.

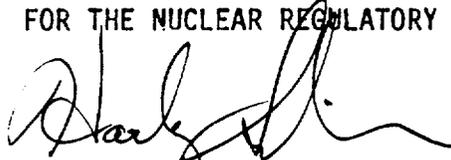
A request for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Docketing and Services Branch, or may be delivered to the Commission's Public Document Room, 2120 L Street, N.W., Washington, D.C. by the above date.

A copy of this petition should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, and to A. H. Stephens, General Counsel, Florida Power Corporation, MAC-A5D, P. O. Box 14042, St. Petersburg, Florida 33733.

For further details with respect to this action, see (1) the application for amendment dated April 14, 1983, as modified December 13, 1983, July 25, 1984, January 24, 1986, October 16, 1986 and August 18, 1987, and (2) the Commission's letter to Florida Power Corporation dated May 5, 1989, which are available for public inspection at the Commission's Public Document Room, 2120 L Street, N.W., Washington, D.C., and at the Crystal River Public Library, 668 N.W. First Avenue, Crystal River, Florida 32629. A copy of item (2) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C., 20555, Attention: Director, Division of Reactor Projects.

Dated at Rockville, Maryland, this 5th day of May 1989.

FOR THE NUCLEAR REGULATORY COMMISSION



Harley Silver, Project Manager
Project Directorate II-2
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation