

May 8, 1989

DISTRIBUTION  
See attached sheet

Mr. W. S. Wilgus  
Vice President, Nuclear Operations  
Florida Power Corporation  
ATTN: Manager, Nuclear Licensing  
P. O. Box 219  
Crystal River, Florida 32629

Dear Mr. Wilgus:

SUBJECT: CRYSTAL RIVER UNIT 3 - ISSUANCE OF AMENDMENT RE: REACTOR COOLANT SYSTEM HIGH POINT VENTS (TAC NOS. 54387)

The Commission has issued the enclosed Amendment No. 112 to Facility Operating License No. DPR-72 for the Crystal River Unit No. 3 Nuclear Generating Plant (CR-3). This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated March 31, 1983, as supplemented June 22, 1983 and revised February 24, 1984, May 31, 1984 and December 31, 1984.

This amendment adds requirements to the TS for the reactor coolant system high point vents.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by

Harley Silver, Project Manager  
Project Directorate II-2  
Division of Reactor Projects-I/II  
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 112 to DPR-72
- 2. Safety Evaluation

cc w/enclosures:  
See next page

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D Silver  
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Mr. W. S. Wilgus  
Florida Power Corporation

Crystal River Unit No. 3 Nuclear  
Generating Plant

cc:

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555  
May 8, 1988

FLORIDA POWER CORPORATION  
CITY OF ALACHUA  
CITY OF BUSHNELL  
CITY OF GAINESVILLE  
CITY OF KISSIMMEE  
CITY OF LEESBURG  
CITY OF NEW SMYRNA BEACH AND UTILITIES COMMISSION, CITY OF NEW SMYRNA BEACH  
CITY OF OCALA  
ORLANDO UTILITIES COMMISSION AND CITY OF ORLANDO  
SEBRING UTILITIES COMMISSION  
SEMINOLE ELECTRIC COOPERATIVE, INC.  
CITY OF TALLAHASSEE

DOCKET NO. 50-302

CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 112  
License No. DPR-72

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Florida Power Corporation, et al. (the licensees) dated March 31, 1983, as supplemented June 22, 1983 and revised February 24, 1984, May 31, 1984 and December 31, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations.
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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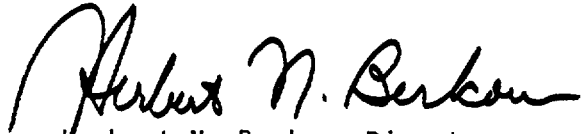
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-72 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 112, are hereby incorporated in the license. Florida Power Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance and shall be implemented within 30 days of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director  
Project Directorate II-2  
Division of Reactor Projects-I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: May 8, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 112

FACILITY OPERATING LICENSE NO. DPR-72

DOCKET NO. 50-302

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

Remove

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Insert

3/4 4-33  
3/4 4-34  
B3/4 4-14

## REACTOR COOLANT SYSTEM

### 3/4.4.11 REACTOR COOLANT SYSTEM VENTS

#### LIMITING CONDITION FOR OPERATION

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3.4.11 At least one reactor coolant system vent path consisting of one vent valve and one block valve capable of being powered from emergency buses shall be OPERABLE and closed at each of the following locations:

- a. Pressurizer Steam Space
- b. Reactor Coolant Loop A High Point
- c. Reactor Coolant Loop B High Point

APPLICABILITY: MODES 1, 2, 3 and 4

#### ACTION:

- a. With the pressurizer steam space vent path inoperable, maintain the inoperable vent path closed with power removed from the valve actuator of the vent valve and block valve in the vent path and provided an alternate vent path is available; with no alternate vent path available, restore the pressurizer steam space vent path to OPERABLE status within 30 days, or prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within the next 30 days describing the reasons for inoperability and a schedule for corrective action.
- b. With one of the two reactor coolant loop vent paths inoperable, maintain the inoperable vent path closed with power removed from the valve actuator of the vent valve and block valve in the inoperable vent path; restore the inoperable vent path to OPERABLE status within 30 days, or prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within the next 30 days describing the reasons for inoperability and a schedule for corrective action.
- c. With two reactor coolant loop vent paths inoperable; maintain the inoperable vent paths closed with power removed from the valve actuators of all the vent valves and block valves in the inoperable vent paths, and restore at least one of two of the vent paths to OPERABLE status within 72 hours or be in HOT STANDBY within 6 hours and in COLD SHUTDOWN within the following 30 hours.
- d. The provisions of Specification 3.0.4 are not applicable.

## SURVEILLANCE REQUIREMENTS

4.4.11. Each reactor coolant system vent path shall be demonstrated OPERABLE at least once per 18 months by:

1. Verifying all manual isolation valves in each vent path are locked in the open position.
2. Cycling each vent valve and block valve through at least one complete cycle of full travel from the Control Room.
3. Verifying flow through the Reactor Coolant Vent System vent paths.

## **REACTOR COOLANT SYSTEM (continued)**

### **BASES**

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#### **3/4.4.11 Reactor Coolant System Vents**

The operability and surveillance requirements for the Reactor Coolant System (RCS) Vents ensure that gases which could inhibit core cooling during natural circulation may be vented from the RCS. This system was installed as a result of NUREG-0737, Item II.B.1.





UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
SUPPORTING AMENDMENT NO. 112 TO FACILITY OPERATING LICENSE NO. DPR-72  
FLORIDA POWER CORPORATION, ET AL.  
CRYSTAL RIVER UNIT NO. 3 NUCLEAR GENERATING PLANT  
DOCKET NO. 50-302

INTRODUCTION

By letter dated March 31, 1983, as supplemented June 22, 1983 and revised February 24, 1984, May 31, 1984, and December 31, 1984, Florida Power Corporation (FPC or the licensee) submitted Technical Specifications for the Reactor Coolant System (RCS) high point vents. The proposed TS add action requirements should one or more of the vents become inoperable. They also define surveillance requirements for the RCS vents.

EVALUATION

Generic Letter 83-37 provided guidance on several of the NUREG-0737 requirements, including RCS high point vents. The TS proposed by the licensee conform to the model TS provided in Generic Letter 83-37 in all but two areas.

As an alternative to the requirement to shut down if a vent path cannot be restored within 30 days, the licensee proposed that if the vent cannot be restored to service within 30 days, FPC would submit a special report within the next 30 days outlining the plan for restoring the vent to service. As far as the requirement to be in either mode five or six to verify flow through the vents, the licensee proposed that this verification could best be done in other modes.

In the event that the pressurizer steam space vent were to be inoperable, the pressurizer could still be vented via the power operated relief valve. If one of the coolant loop high point vents were to be inoperable, the vent in the opposite loop and the pressurizer steam space vent would still be available. For these reasons the staff has concluded that providing a 30-day report rather than shutting down if the RCS vents are not restored to operable status within 30 days does not pose any undue risk to the public health and safety, and that this deviation from the model TS is therefore acceptable, providing that the 30-day report includes a commitment to repair the vent system at the next suitable outage of sufficient length. The purpose of the flow verification surveillance is to verify flow, regardless of the mode the plant is in. The objective of this surveillance can best be met in modes other than five or six since these modes involve reduced RCS pressure. For this reason, the staff concludes that this deviation from the model TS is also acceptable.

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SUMMARY

The proposed TS dealing with RCS vents are in accordance with the guidance of Generic Letter 83-37 in all but two areas. The staff has concluded that the proposed deviations from the guidance are acceptable. Therefore, the staff concludes that the TS changes are acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: May 8, 1989

Principal Contributors:

H. Silver  
G. Wunder

DATED: May 8, 1989

AMENDMENT NO. 112 TO FACILITY OPERATING LICENSE NO. DPR-72-CRYSTAL RIVER UNIT 3

Docket File

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ACRS (10)

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ARM/LFMB

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