

April 13, 1988

Docket No. 50-302

DISTRIBUTION

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Vice President, Nuclear Operations
Florida Power Corporation
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Dear Mr. Wilgus:

SUBJECT: CRYSTAL RIVER UNIT 3 - ISSUANCE OF AMENDMENT RE: CONTAINMENT
AIR LOCK SURVEILLANCE REQUIREMENTS (TAC NO. 64804)

The Commission has issued the enclosed Amendment No. 105 to Facility Operating License No. DPR-72 for the Crystal River Unit No. 3 Nuclear Generating Plant (CR-3). This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated February 17, 1987, as revised March 9, 1988.

This amendment revises the containment air lock surveillance requirements so that the TSs are now in conformance with 10 CFR Part 50, Appendix J and the exemption previously issued by the staff on December 9, 1986.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by

Harley Silver, Project Manager
Project Directorate II-2
Division of Reactor Projects-I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 105 to DPR-72
2. Safety Evaluation

cc w/enclosures:

See next page

LA:PDII-2
DA:Silver
03/29/88

PM:PDII-2
HSilver:bd
03/29/88

D:PDII-2
HBerkow
03/29/88

OGC-WF
SH Lewis
03/28/88

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Mr. W. S. Wilgus
Florida Power Corporation

Crystal River Unit No. 3 Nuclear
Generating Plant

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER CORPORATION
CITY OF ALACHUA
CITY OF BUSHNELL
CITY OF GAINESVILLE
CITY OF KISSIMMEE
CITY OF LEESBURG
CITY OF NEW SMYRNA BEACH AND UTILITIES COMMISSION, CITY OF NEW SMYRNA BEACH
CITY OF OCALA
ORLANDO UTILITIES COMMISSION AND CITY OF ORLANDO
SEBRING UTILITIES COMMISSION
SEMINOLE ELECTRIC COOPERATIVE, INC.
CITY OF TALLAHASSEE

DOCKET NO. 50-302

CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 105
License No. DPR-72

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power Corporation, et al. (the licensees) dated February 17, 1987, as revised March 9, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

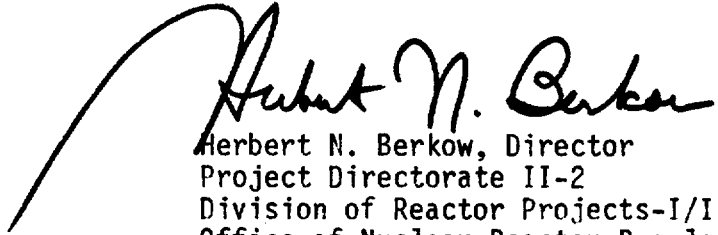
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-72 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 105, are hereby incorporated in the license. Florida Power Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects-I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 13, 1988

ATTACHMENT TO LICENSE AMENDMENT NO.105

FACILITY OPERATING LICENSE NO. DPR-72

DOCKET NO. 50-302

Replace the following page of the Appendix "A" Technical Specifications with the attached page. The revised page is identified by amendment number and contains vertical lines indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

Remove

3/4 6-5

Insert

3/4 6-5

CONTAINMENT SYSTEMS

CONTAINMENT AIR LOCKS

LIMITING CONDITION FOR OPERATION

3.6.1.3 Each containment air lock shall be OPERABLE with:

- a. Both doors closed except when the air lock is being used for normal transit entry and exit through the containment, then at least one air lock door shall be closed, and
- b. An overall air lock leakage rate of $\leq 0.05 L_a$ at P_a , 49.6 psig.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With an air lock inoperable, restore the air lock to OPERABLE status within 24 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.3 Each containment air lock shall be demonstrated OPERABLE:

- a. By verifying seal leakage $\leq 0.01 L_a$ when the volume between the door seals is pressurized to ≥ 8 psig for at least 30 seconds after stabilizing pressure for at least 15 minutes:
 1. After each opening, (in Modes 1, 2, 3, or 4) except when the air lock is being used for multiple entries, then at least once per 72 hours.
 2. *Prior to establishing CONTAINMENT INTEGRITY when maintenance has not been performed on the air lock. (Reperformance of this test is not required prior to entering Mode 4 if the air lock has not been opened since the previous test.)
- b. By conducting overall air lock leakage tests at not less than P_a (49.6 psig), and verifying the overall air lock leakage rate is within its limit:
 1. - #At least once per 6 months, and
 2. *Prior to establishing CONTAINMENT INTEGRITY when maintenance has been performed on the air lock that could affect the air lock sealing capability.
- c. At least once per 6 months by verifying that only one door in each air lock can be opened at a time.

*Exemption to Appendix "J" of 10 CFR 50.

#The provisions of Specification 4.0.2 are not applicable.

CONTAINMENT SYSTEMS

INTERNAL PRESSURE

LIMITING CONDITION FOR OPERATION

3.6.1.4 Primary containment internal pressure shall be maintained between 17.7 and 12.7 psia.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With the containment internal pressure outside of the limits above, restore the internal pressure to within the limits within 1 hour or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.4 The primary containment internal pressure shall be determined to within the limits at least once per 12 hours.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 105 TO FACILITY OPERATING LICENSE NO. DPR-72
FLORIDA POWER CORPORATION, ET AL.
CRYSTAL RIVER UNIT NO. 3 NUCLEAR GENERATING PLANT
DOCKET NO. 50-302

INTRODUCTION

By letter dated February 17, 1987, Florida Power Corporation (the licensee) proposed certain changes to the Crystal River Unit 3 Technical Specifications (TSs) concerning the containment air lock surveillance requirements (TS 4.6.1.3). These proposed changes update the plant Technical Specifications to reflect the exemption from the requirements of 10 CFR 50, Appendix J, paragraph III.D.1(b)(ii), which the staff granted the licensee on December 9, 1986. The exemption provided the licensee relief from the requirements to conduct a full pressure air lock leakage test by permitting substitution of an air lock seal test when the reactor is in cold shutdown (Mode 5) or refueling (Mode 6), and when no maintenance has been performed that could affect air lock sealing capabilities. A full pressure air lock leakage test will be performed at least once per 6 months and prior to establishing containment integrity following any maintenance that affects the air lock sealing capability.

In response to discussions with the staff (as discussed below), the licensee submitted a revised change to TS 4.6.1.3 by letter dated March 9, 1988. This letter did not alter, in any way, the staff's initial determination of no significant hazards considerations.

EVALUATION

In the February 17, 1987 letter, the licensee proposed to update the TS for containment air lock surveillance testing by requiring the performance of an air lock seal test in Modes 5 and 6 if the air lock doors had been opened. If the doors had not been opened, the licensee stated that the performance of the 6-month full pressure test was sufficient to demonstrate the overall sealing capabilities of the air lock. In addition, to ensure that air lock leakage continues to be within specified limits, an air lock leakage test at full pressure will be performed prior to establishing primary containment integrity whenever maintenance has been performed on the doors.

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However, the TS surveillance requirements as represented in the licensee's February 17, 1987 letter did not specify that a seal leakage test will be performed on each door prior to ascending to Mode 4 when no maintenance has been performed. Appendix J of 10 CFR 50, paragraph III.D.2.(b)(ii) explicitly states that "air locks opened during periods when containment integrity is not required by the plant's technical specifications shall be tested at the end of such periods at not less than Pa." The exemption granted by the staff did not address this point, and the staff maintained that performance of a seal test before ascending to Mode 4 was required even when no maintenance was done that could affect the sealing capability. This issue was discussed with the licensee and they subsequently committed to add a paragraph to the TS incorporating this requirement, which was submitted by letter dated March 9, 1988.

The staff also raised the concern that paragraph "a" in the TS was not very clear. We requested that the licensee add a statement identifying that the seal leakage test be done after they have assured that the pressure has stabilized. The licensee agreed to add a clarifying sentence to that effect.

Based on the above, the staff concludes that the proposed TS is in conformance with the requirements of 10 CFR 50, Appendix J and the approved exemption, and will adequately ensure containment air lock leak tightness. The staff therefore finds the proposed changes acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or changes to a surveillance requirement. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: April 13, 1988

Principal Contributor:

A. Gill