Docket No. 50-302

Mr. W. S. Wilgus

Vice President, Nuclear Operations

Florida Power Corporation

ATTN: Manager, Nuclear Licensing

P.O. Box 219

Crystal River, Florida 32629

Dear Mr. Wilgus:

SUBJECT: CRYSTAL RIVER UNIT 3 - CORRECTION

TO AMENDMENT NO. 106

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On April 22, 1988, the Commission issued Amendment No. 106 to Facility Operating License No. DPR-72 for the Crystal River Unit No. 3 Nuclear Generating Plant. The amendment revised the Technical Specifications to provide specific actions to be taken when one of the batteries supplying D.C. control power to the 230 kv switchyard breakers is inoperable, and revised the time the plant may operate with one inoperable battery. The amendment also more explicitly defined the surveillances to be performed in Modes 5 and 6.

Due to an administrative error, page 3/4 8-6c did not incorporate changes from a previously issued amendment (Amendment No. 104 issued February 19, 1988). Enclosed is the corrected page to be inserted into the Technical Specifications.

Please accept our apologies for any inconvenience this error may have caused.

Sincerely,

Harley Silver, Sr. Project Manager Project Directorate II-2 Division of Reactor Projects-I/II Office of Nuclear Reactor Regulation

Enclosure: As stated

cc w/enclosure: See next page

L/APPIN -2 DM Pier 5/0/88 PM:PBII-2 HS11ver:bg 5/**9**/88

D: PD(11-2 HBerkov 5/9/88 Mr. W. S. Wilgus Florida Power Corporation

cc: Mr. R. W. Neiser Senior Vice President and General Counsel Florida Power Corporation P. O. Box 14042 St. Petersburg, Florida 33733

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Crystal River Unit No. 3 Nuclear Generating Plant

State Planning and Development Clearinghouse Office of Planning and Budget Executive Office of the Governor The Capitol Building Tallahassee, Florida 32301

Mr. F. Alex Griffin, Chairman Board of County Commissioners Citrus County 110 North Apopka Avenue Inverness, Florida 36250

Mr. E. C. Simpson Director, Nuclear Site Florida Power Corporation Support P.O. Box 219 Crystal River, Florida 32629

ELECTRICAL POWER SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- *3. Simulating a loss of offsite power in conjunction with Reactor Building high pressure and Reactor Building high-high pressure tests signals, and
 - a) Verifying de-energization of the emergency buses and load shedding from the emergency buses,
 - b) Verifying that the 4160 v. emergency bus tie breakers open,
 - c) Verifying the diesel starts from ambient condition on the auto-start signal, energizes the emergency buses with permanently connected loads, energizes the auto-connected emergency loads through the load sequencer, and operates for >5 minutes while its generator is loaded with the emergency loads.
- **4. Verifying the diesel generator operates for at least 60 minutes. During the first 5 minutes but no greater than 6 minutes of this test the diesel generator shall be loaded to greater than or equal to 3248 kw but less than 3300 kw and during the remaining time of this 60 minute test, the diesel generator shall be loaded to greater than or equal to 2750 kw but less than 3000 kw,
- **5. Verifying that the auto-connected loads to each diesel generator for the worst case diesel generator operating condition do not exceed 3248 kw, and
 - 6. Verifying that the automatic load sequence timers are OPERABLE with each load sequence time interval within + 10%.

^{*}This test shall be performed in MODE 3.

^{**}These revised requirements shall become effective upon approval of the licensee's final test report and supporting documentation and shall apply only until the end of Cycle VII.