

NOVEMBER 7 1979

NOVEMBER 9
Docket No. 50-302

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Mr. W. P. Stewart
Director, Power Production
Florida Power Corporation
P. O. Box 14042, Mail Stop C-4
St. Petersburg, Florida 33733

Dear Mr. Stewart:

The Commission has issued the enclosed Amendment No. 25 to Facility Operating License No. DPR-72 for Crystal River Unit No. 3 Nuclear Generating Plant. This amendment consists of changes to the Technical Specifications in response to portions of your applications dated July 21, 1977 (Change Request Nos. 3a.16 and 3e), September 22, 1978 (Change Request Nos. 33t and u) and May 23, 1979 (Change Request No. 41).

This amendment modifies the Technical Specifications to indicate a modified offsite organization, clarify the functions of the Nuclear General Review Committee in environmental matters, and change the units for the high rate of power increase reporting requirement.

Your remaining requests contained in the above-mentioned applications will be handled separately.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by
Robert W. Reid

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Enclosures:

1. Amendment No. 25 to DPR-72
2. Safety Evaluation
3. Notice

cc w/enclosures:
See next page

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RWB
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OFFICE	ORB#4: DOR	ORB#4: DOR	C-ORB#4: DOR	A-AD-ORB: DOR	OELD
SURNAME	RIngram/cb	MFairtile	RReid	WGammill	JH Lewis
DATE	10/29/79	10/12/79	11/5/79	11/3/79	11/6/79



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

November 7, 1979

Docket No. 50-302

Mr. W. P. Stewart
Director, Power Production
Florida Power Corporation
P. O. Box 14042, Mail Stop C-4
St. Petersburg, Florida 33733

Dear Mr. Stewart:

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Your remaining requests contained in the above-mentioned applications will be handled separately.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Morton B. Fairtile for

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Enclosures:

1. Amendment No. 25 to DPR-72
2. Safety Evaluation
3. Notice

cc w/enclosures:
See next page

Florida Power Corporation

cc w/enclosure(s):

Mr. S. A. Brandimore
Vice President and General Counsel
P. O. Box 14042
St. Petersburg, Florida 33733

Mr. Wilbur Langely, Chairman
Board of County Commissioners
Citrus County
Iverness, Florida 36250

U. S. Environmental Protection Agency
Region IV Office
ATTN: EIS COORDINATOR
345 Courtland Street, N.E.
Atlanta, Georgia 30308

Director, Technical Assessment
Division
Office of Radiation Programs
(AW-459)
U. S. Environmental Protection Agency
Crystal Mall #2
Arlington, Virginia 20460

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Crystal River, Florida 32629

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Tallahassee, Florida 32304

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Department of Environmental Regulation
Power Plant Siting Section
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Department of Legal Affairs
The Capitol
Tallahassee, Florida 32304

Mr. Robert B. Borsum
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Nuclear Power Generation Division
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Bethesda, Maryland 20014

cc w/enclosures & incoming
dtd: 5/23/79*
Bureau of Intergovernmental
Relations
660 Apalachee Parkway
Tallahassee, Florida 32304

*Incoming dtd. 7/21/77 & 9/22/78
provided w/earlier actions.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER CORPORATION
CITY OF ALACHUA
CITY OF BUSHNELL
CITY OF GAINESVILLE
CITY OF KISSIMMEE
CITY OF LEESBURG
CITY OF NEW SMYRNA BEACH AND UTILITIES COMMISSION, CITY OF NEW SMYRNA BEACH
CITY OF OCALA
ORLANDO UTILITIES COMMISSION AND CITY OF ORLANDO
SEBRING UTILITIES COMMISSION
SEMINOLE ELECTRIC COOPERATIVE, INC.
CITY OF TALLAHASSEE

DOCKET NO. 50-302

CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 25
License No. DPR-72

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Florida Power Corporation, et al (the licensees) dated July 21, 1977, September 22, 1978, and May 23, 1979 comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-72 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 25, are hereby incorporated in the license. Florida Power Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Morton B. Fairlie for
Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 7, 1979

ATTACHMENT TO LICENSE AMENDMENT NO. 25

FACILITY OPERATING LICENSE NO. DPR-72

DOCKET NO. 50-302

Replace the following pages of the Appendices "A" and "B" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Appendix A Pages

6-2

6-6

6-7

6-11

6-12

6-16

Appendix B Pages

5-1

5-2

5-4

5-4a (new page)

5-4b (new page)

5-5

5-6

6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Nuclear Plant Manager shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

6.1.2 The Nuclear Plant Manager shall be responsible for an annual fire protection inspection which shall be performed utilizing either qualified offsite licensee personnel or an outside fire protection firm. The inspection shall consist of: a) an inspection of safety-related areas of the Plant to verify that they are in conformance with the fire hazards analysis; and b) a review of the Fire Brigade organization, training, and drills to verify their conformance with the requirements of Section 27 of the NFPA Code-1976.

6.1.3 The Nuclear Plant Manager shall be responsible for an inspection of the fire protection program to be performed by a qualified outside fire consultant at least once per 36 months.

6.2 ORGANIZATION

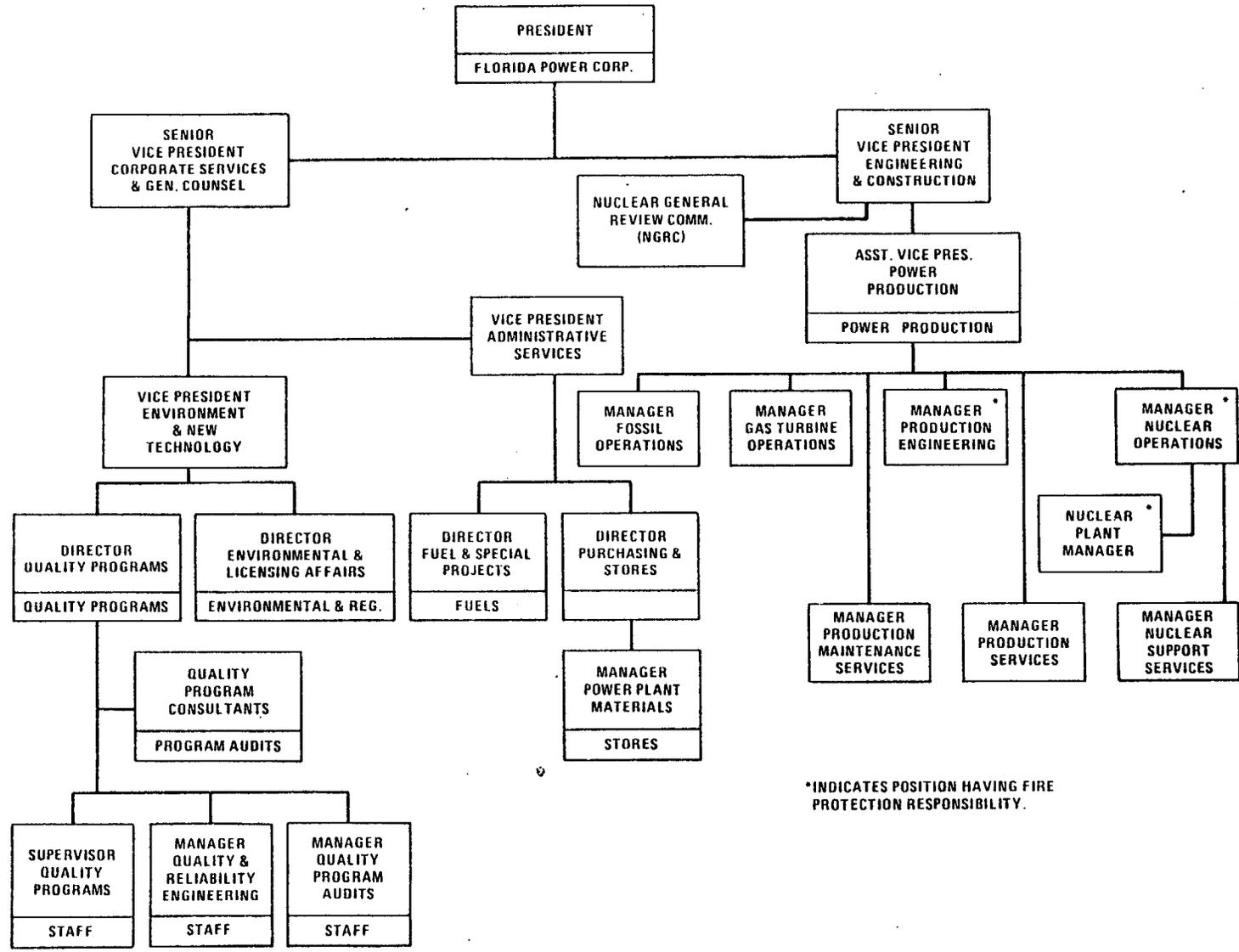
OFFSITE

6.2.1 The offsite organization for facility management and technical support shall be as shown on Figure 6.2-1.

FACILITY STAFF

6.2.2 The Facility organization shall be as shown on Figure 6.2-2 and:

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b. At least one licensed Operator shall be in the control room when fuel is in the reactor.
- c. At least two licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.
- d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
- e. All CORE ALTERATIONS after the initial fuel loading shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
- f. A Fire Brigade of at least 4 members shall be maintained onsite at all times. This excludes 5 members of the minimum shift crew necessary for safe shutdown of the plant and personnel required for other essential functions.



*INDICATES POSITION HAVING FIRE PROTECTION RESPONSIBILITY.

Figure 6.2-1 Offsite Organization

ADMINISTRATIVE CONTROLS

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the Chemistry and Radiation Protection Engineer who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Nuclear Plant Manager and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Nuclear Plant Manager and shall meet or exceed the requirements of Section 27 of the NFPA Code-1976, except for Fire Brigade training sessions which shall be held at least quarterly.

6.5 REVIEW AND AUDIT

6.5.1 PLANT REVIEW COMMITTEE (PRC)

FUNCTION

6.5.1.1 The Plant Review Committee shall function to advise the Nuclear Plant Manager on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The Plant Review Committee shall be composed of the:

Chairman:	Technical Services Superintendent
Member:	Operations Superintendent
Member:	Technical Support Engineer
Member:	Maintenance Superintendent
Member:	Chemistry and Radiation Protection Engineer
Member:	At large (Designated by Chairman)

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the PRC Chairman to serve on a temporary basis; no more than two alternates shall participate as voting members in PRC activities at any one time.

MEETING FREQUENCY

6.5.1.4 The PRC shall meet at least once per calendar month and as convened by the PRC Chairman or his designated alternate.

ADMINISTRATIVE CONTROLS

QUORUM

6.5.1.5 A quorum of the PRC shall consist of the Chairman or his designated alternate and four members including alternates.

RESPONSIBILITIES

6.5.1.6 The Plant Review Committee shall be responsible for:

- a. Review of 1) all procedures required by Specification 6.8 and changes thereto, 2) any other proposed procedures or changes thereto as determined by the Nuclear Plant Manager to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear Safety.
- c. Review of all proposed changes to the Appendix "A" Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Manager, Nuclear Operations and to the Chairman of the Nuclear General Review Committee.
- f. Review of events requiring 24 hour written notification to the Commission.
- g. Review of facility operations to detect potential nuclear safety hazards.
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman of the Nuclear General Review Committee.
- i. Review of the Plant Security Plan and implementing procedures and shall submit recommended changes to the Chairman of the Nuclear General Review Committee.
- j. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Chairman of the Nuclear General Review Committee.

ADMINISTRATIVE CONTROLS

AUTHORITY

6.5.1.7 The Plant Review Committee shall:

- a. Recommend to the Nuclear Plant Manager written approval or disapproval of items considered under 6.5.1(a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.
- c. Provide written notification within 24 hours to the Manager, Nuclear Operations and the Nuclear General Review Committee of disagreement between the PRC and the Nuclear Plant Manager; however, the Nuclear Plant Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.1.8 The Plant Review Committee shall maintain written minutes of each meeting and copies shall be provided to the Manager, Nuclear Operations and Chairman of the Nuclear General Review Committee.

6.5.2 NUCLEAR GENERAL REVIEW COMMITTEE (NGRC)

FUNCTION

6.5.2.1 The Nuclear General Review Committee shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations
- b. Nuclear engineering
- c. Chemistry and radiochemistry
- d. Metallurgy
- e. Instrumentation and control
- f. Radiological safety
- g. Mechanical and electrical engineering
- h. Quality assurance practices

ADMINISTRATIVE CONTROLS

COMPOSITION

6.5.2.2 The NGRC shall be composed of the Chairman, Vice Chairman, and at least 5 members. No more than a minority of the members shall have line responsibility for operation of the facility. The committee shall collectively have the experience and competence required to review problems in the following areas:

- a. Nuclear power plant operations
- b. Nuclear engineering
- c. Chemistry and radiochemistry
- d. Metallurgy
- e. Nondestructive testing
- f. Instrumentation and control
- g. Radiological safety
- h. Mechanical and electrical engineering
- i. Administrative controls
- j. Environmental
- k. Quality assurance practices

QUALIFICATIONS

6.5.2.3 The following minimum experience requirements shall be established for those persons involved in the independent off-site safety review and audit program:

- a. Chairman and Vice-Chairman-Bachelor of Science in engineering or related field and ten years related experience including five years involvement with operation and/or design of nuclear power plants.
- b. Member-Bachelor of Science in engineering or related field and five years related experience including three years involvement with operation and/or design of nuclear power plants.

ALTERNATES

6.5.2.4 All alternate members shall be appointed in writing by the NGRC Chairman to serve on a temporary basis; however no more than two alternates shall participate as voting members in NGRC activities at any one time.

ADMINISTRATIVE CONTROLS

RECORDS

6.5.2.11 Records of NGRC activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each NRG meeting shall be prepared, approved and forwarded to the Senior Vice President Engineering and Construction within 14 days following each meeting.
- b. Reports of reviews encompassed by Section 6.5.2.8 above, shall be prepared, approved and forwarded to the Senior Vice President-Engineering and Construction within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.9 above, shall be forwarded to the Senior Vice President Engineering and Construction and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.6 REPORTABLE OCCURRENCE ACTION

6.6.1 The following actions shall be taken for REPORTABLE OCCURRENCES:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.
- b. Each REPORTABLE OCCURRENCE requiring 24 hour notification to the Commission shall be reviewed by the PRC and submitted to the NGRC and the Manager, Nuclear Operations.

ADMINISTRATIVE CONTROLS

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The facility shall be placed in at least HOT STANDBY within one hour.
- b. The Safety Limit violation shall be reported to the Commission, the Manager, Nuclear Operations and to the NRC within 24 hours.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PRC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the NRC and the Manager, Nuclear Operations within 14 days of the violation.

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, November, 1972.
- b. Refueling operations.
- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.

6.8.2 Each procedure and administrative policy of 6.8.1 above, and changes thereto, shall be reviewed by the PRC and approved by the Nuclear Plant Manager prior to implementation and reviewed periodically as set forth in administrative procedures.

ADMINISTRATIVE CONTROLS

MONTHLY OPERATING REPORT

6.9.1.6 Routine reports of operating statistics and shutdown experience shall be submitted on a monthly basis to the Director, Office of Management Information and Program Control, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, with a copy to the Regional Office, submitted no later than the 15th of each month following the calendar month covered by the report.

REPORTABLE OCCURRENCES

6.9.1.7 The REPORTABLE OCCURRENCES of Specifications 6.9.1.8 and 6.9.1.9 below, including corrective actions and measures to prevent recurrence, shall be reported to the NRC. Supplemental reports may be required to fully describe final resolution of occurrence. In case of corrected or supplemental reports, a licensee event report shall be completed and reference shall be made to the original report date.

PROMPT NOTIFICATION WITH WRITTEN FOLLOWUP

6.9.1.8 The types of events listed below shall be reported within 24 hours by telephone and confirmed by telegraph, mailgram, or facsimile transmission to the Director of the Regional Office, or his designate no later than the first working day following the event, with a written followup report within 14 days. The written followup report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

- a. Failure of the reactor protection system or other systems subject to limiting safety-system settings to initiate the required protective function by the time a monitored parameter reaches the setpoint specified as the limiting safety-system setting in the technical specifications or failure to complete the required protective function.
- b. Operation of the unit or affected systems when any parameter or operation subject to a limiting condition for operation is less conservative than the least conservative aspect of the limiting condition for operation established in the technical specifications.
- c. Abnormal degradation discovered in fuel cladding, reactor coolant pressure boundary, or primary containment.

ADMINISTRATIVE CONTROLS

- d. Reactivity anomalies involving disagreement with the predicted value of reactivity balance under steady-state conditions during power operation greater than or equal to 1% $\Delta k/k$; a calculated reactivity balance indicating a shutdown margin less conservative than specified in the technical specifications; short-term reactivity increases that correspond to a reactor startup rate of greater than 5.2 DPM or, if subcritical, an unplanned reactivity insertion of more than 0.5% $\Delta k/k$; or occurrence of any unplanned criticality.
- e. Failure or malfunction of one or more components which prevents or could prevent, by itself, the fulfillment of the functional requirements of system(s) used to cope with accidents analyzed in the SAR.
- f. Personnel error or procedural inadequacy which prevents or could prevent, by itself, the fulfillment of the functional requirements of systems required to cope with accidents analyzed in the SAR.
- g. Conditions arising from natural or man-made events that, as a direct result of the event, require plant shutdown, operation of safety systems, or other protective measures required by technical specifications.
- h. Errors discovered in the transient or accident analyses or in the methods used for such analyses as described in the safety analysis report or in the bases for the technical specifications that have or could have permitted reactor operation in a manner less conservative than assumed in the analyses.
- i. Performance of structures, systems, or components that requires remedial action or corrective measures to prevent operation in a manner less conservative than that assumed in the accident analyses in the safety analysis report or technical specifications bases; or discovery during plant life of conditions not specifically considered in the safety analysis report or technical specifications that require remedial action or corrective measures to prevent the existence or development of an unsafe condition.

THIRTY-DAY WRITTEN REPORT

6.9.1.9 The types of events listed below shall be the subject of written reports to the Director of the Regional Office within 30 days of

5.0 ADMINISTRATIVE CONTROLS

Objective

To define the organization, assign responsibilities, describe the environmental surveillance procedures, provide for a review and audit function, and prescribe the reporting requirements in order to insure continuing protection of the environment and implement the Environmental Technical Specifications.

5.1 ORGANIZATION

The organization responsible for environmental protection, environmental monitoring and the implementation of the Environmental Technical Specifications for Crystal River Unit 3, is shown on Figure 5.1-1.

5.2 RESPONSIBILITY

The responsibility for the conduct of the preoperational environmental monitoring program described in Section 3 and special studies described in Section 4 is that of the Quality and Environmental Department under the direction of the Director of Environmental and Licensing Affairs. The responsibility for the conduct of the operational environmental monitoring program and the implementation of Environmental Technical Specifications is the responsibility of the Power Production Department.

The plant organization is responsible for the development of Operating and Surveillance Procedures described generally in Section 5.5 and supplying field data to the Manager, Nuclear Support Services as required by Sections 2, 3 and 4 of the Environmental Technical Specifications.

The Manager, Nuclear Support Services is responsible for consultant contracts, State and local regulatory agreements, assembly of data, preparation and review of reports required by Section 5.6 of these Environmental Technical Specifications, and making recommendations to improve environmental protection practices.

All reports and correspondence with the NRC regarding the Environmental Technical Specifications shall be approved and signed by the Manager, Nuclear Operations. The Nuclear Plant Manager shall, however, make reports by telephone and telegraph of any incident or occurrence requiring reporting within 24 hours or less, as required in Section 5.6.

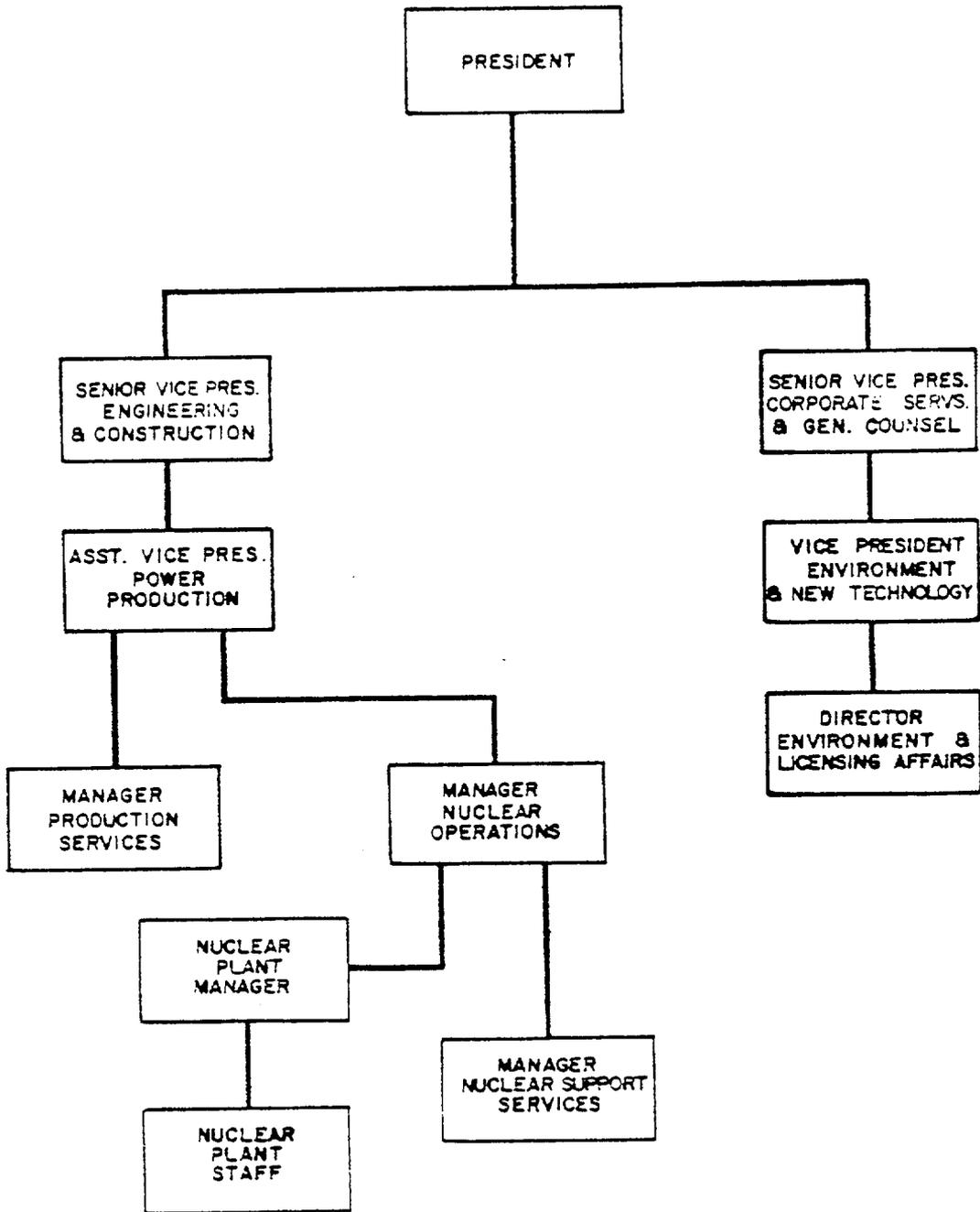


FIGURE 5.1-1 ORGANIZATION FOR IMPLEMENTING ENVIRONMENTAL TECHNICAL SPECIFICATIONS .

5.3 REVIEW AND AUDIT5.3.1 Function

The Nuclear General Review Committee shall function to provide independent review and audit of designated activities in the areas of environmental monitoring and surveillance.

5.3.2 Composition

The NGRC shall be composed of the Chairman, Vice Chairman, and at least 5 members. No more than a minority of the members shall have line responsibility for operation of the facility. The committee shall collectively have the experience and competence required to review problems in the following areas:

- a. Nuclear power plant operations
- b. Nuclear engineering
- c. Chemistry and radiochemistry
- d. Metallurgy
- e. Nondestructive testing
- f. Instrumentation and control
- g. Radiological safety
- h. Mechanical and electrical engineering
- i. Administrative controls
- j. Environmental
- k. Quality assurance practices

5.3.3 Qualifications

The following minimum experience requirements shall be established for those persons involved in the independent off-site safety review and audit program:

- a. Chairman and Vice-Chairman-Bachelor of Science in engineering or related field and ten years related experience including five years involvement with operation and/or design of nuclear power plants.
- b. Member-Bachelor of Science in engineering or related field and five years related experience including three years involvement with operation and/or design of nuclear power plants.

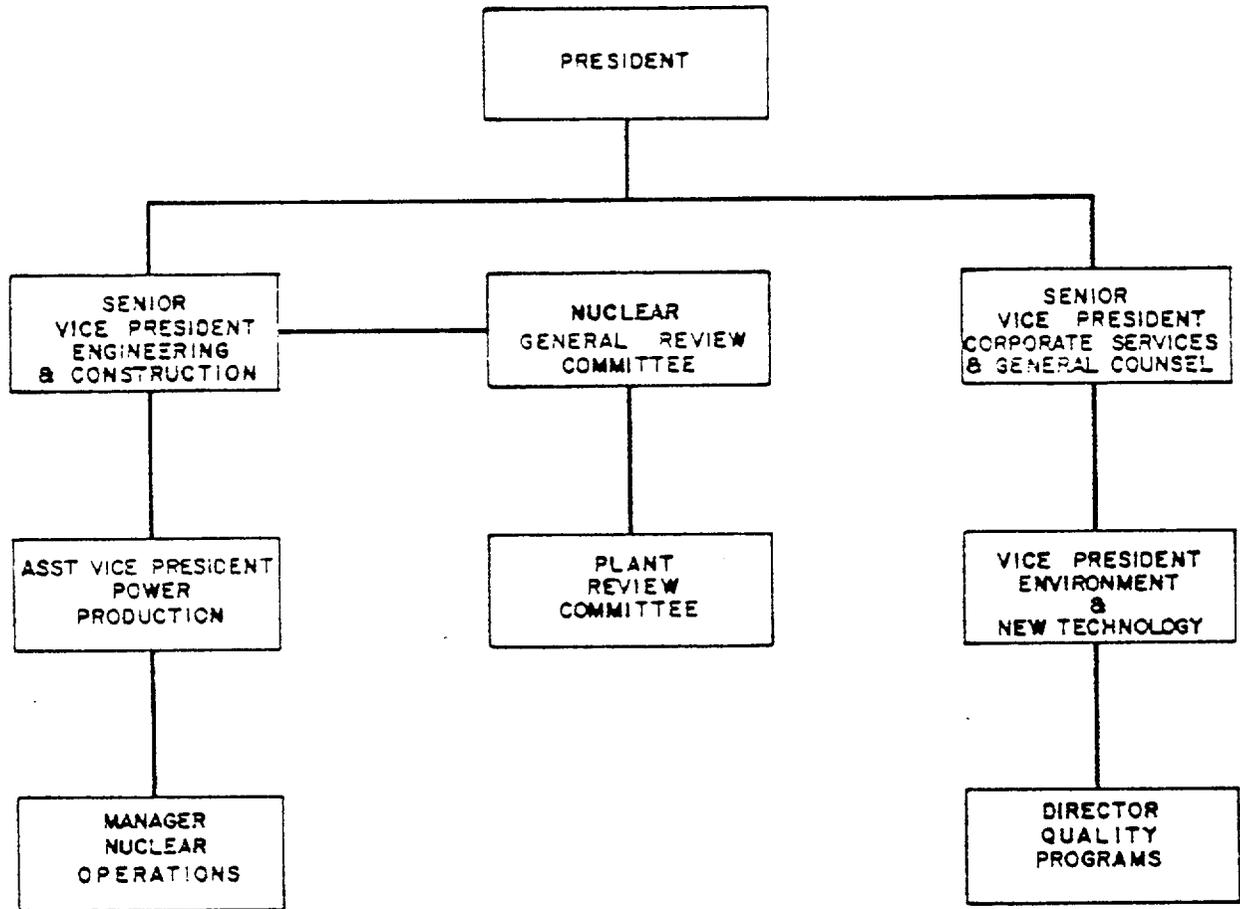


FIG. 5.3-1 ORGANIZATION FOR INDEPENDENT REVIEW AND AUDIT.

5.3.4 Alternates

All alternate members shall be appointed in writing by the NGRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in NGRC activities at any one time.

5.3.5 Consultants

Consultants shall be utilized as determined by the NGRC Chairman to provide expert advice to the NGRC.

5.3.6 Meeting Frequency

The NGRC shall meet at least once per calendar quarter during the initial year of facility operation following fuel loading and at least once per six months thereafter.

5.3.7 Quorum

A quorum of NGRC shall consist of the Chairman or his designated alternate and five additional NGRC members, including alternates. No more than a minority of the quorum shall have line responsibility for operation of the facility.

5.3.8 Review

The NGRC shall review:

- a. Proposed changes to the Environmental Technical Specifications and the evaluated impact of the changes.
- b. Proposed changes or modifications to plant systems or equipment and the evaluated impact which would require a change in the procedures described in 5.5.1 below (or which would affect the evaluation of the plant's environmental impact) as determined by the Plant Review Committee.
- c. Reported instances of violations of Environmental Technical Specifications, the reaching of specified reporting levels, and reportable environmental occurrences.
- d. Proposed special tests or experiments which might involve a change in the Environmental Technical Specifications or involve an unreviewed environmental impact question.
- e. Temporary changes to procedures as described in 5.5.2.
- f. Events requiring 24 hour notification to the Commission.
- g. Descriptions of changes, tests or experiments, and the results thereof, as described in 5.5.3.a.

- h. Audits of the environmental monitoring and surveillance program.

5.3.9 Audits

Audits of facility activities shall be performed under the cognizance of the NGRC. These audits shall encompass:

- a. Implementation of the environmental monitoring and surveillance programs at least once per twelve months.
- b. Conformance to procedures and ETS requirements at least once per twelve months.
- c. Contractor environmental monitoring and surveillance activities at least once per twelve months.

5.3.10 Records

Records of NGRC activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each NGRC meeting shall be prepared, approved and forwarded to the Senior Vice President Engineering and Construction within 14 days following each meeting.
- b. Reports of reviews encompassed by Section 5.3.8 above, shall be prepared, approved and forwarded to the Senior Vice President Engineering and Construction within 14 days following completion of the review.
- c. Audit reports encompassed by Section 5.3.9 above, shall be forwarded to the Senior Vice President Engineering and Construction and to the management positions responsible for the areas audited within 30 days after completion of the audit.

5.3.11 Authority

The NGRC shall report to and advise the Senior Vice President Engineering and Construction on those areas of responsibility specified in Sections 5.3.8, 5.3.9 and 5.3.10 (see Figure 5.3-1).

5.4 ACTION TO BE TAKEN IF LIMITING CONDITION FOR OPERATION IS EXCEEDED

- 5.4.1 Immediate remedial actions as permitted by these environmental technical specifications shall be implemented until such time as the limiting condition for operation is met.
- 5.4.2 The occurrence shall be promptly reported to the Chairman of the Nuclear General Review Committee and investigated as specified in Section 5.3.
- 5.4.3 The Nuclear General Review Committee shall prepare and submit a report for each occurrence in accordance with Section 5.3.10.
- 5.4.4 The Manager, Nuclear Operations, shall report the occurrence to the NRC as specified in Section 5.6.2.

5.5 PROCEDURES

- 5.5.1 Explicit written procedures, including applicable check-off lists and instructions, shall be prepared for the implementation of the monitoring requirements described in Sections 2 and 3, approved as specified in Section 5.5.2, and adhered to for operation of all systems and components involved in carrying out the effluent release and environmental monitoring programs. Procedures shall include sampling, instrument calibration, analysis, and action to be taken when limits are approached or exceeded. Calibration frequencies and standards for instruments used in performing the measurements shall be included. Testing frequency of alarms shall be included. These frequencies shall be determined from experience with similar instruments in similar environments and from manufacturers' technical manuals.
- 5.5.2 All procedures implemented by plant staff personnel described in Section 5.5.1 above, and changes thereto, shall be reviewed as specified in Section 5.3 and approved by the Nuclear Plant Manager prior to implementation. Temporary changes to procedures which do not change the intent of the original procedure may be made, provided such changes are approved by two members of the plant management staff, one of whom holds a senior operator's license. Such changes shall be documented, subsequently reviewed and approved by the Plant and Nuclear General Review Committees. All procedures and changes to procedures utilized by contractors to implement the environmental monitoring programs described in Section 3 shall be reviewed and approved by the Manager, Nuclear Support Services.

5.5.3 Prior to special tests or changes:

- a. If the Nuclear Plant Manger decides to make a change in the facility or Operating Procedures, or to conduct a test or experiment, and concludes that the proposed change, test, or experiment does not involve a change in the Environmental Technical Specifications or an unreviewed environmental impact question, he may order the change, test or experiment to be made, shall enter a description thereof in the operating records of the facility, and shall send a copy of the instructions pertinent thereto to the Chairman of the Nuclear General Review Committee for review per Section 5.3.8.
- b. If the Nuclear Plant Manager desires to make a change in the facility or Operating Procedures, or to conduct a test or experiment which in his opinion might involve a change in the Environmental Technical Specifications, or involve an unreviewed environmental impact question, he shall not order such change, test or experiment until he has referred the matter to the Nuclear General Review Committee for review and report to the Senior Vice President Engineering and Construction for resolution.

5.6 PLANT REPORTING REQUIREMENTS

5.6.1 Routine Reports

A. Annual Environmental Operating Report

(1) Non radiological Volume

A report on the nonradiological environmental surveillance programs for the previous 12 months of operation shall be



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 25 TO FACILITY OPERATING

LICENSE NO. DPR-72

FLORIDA POWER CORPORATION, ET AL

CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT

DOCKET NO. 50-302

Introduction

By letter dated May 23, 1979, Florida Power Corporation (FPC) proposed changes to the Technical Specifications (TS) for Crystal River Unit No. 3 (CR-3). These changes would revise the FPC offsite organization and the unit in which maximum rate of change of power is specified. Also by letter dated July 21, 1977, the licensee proposed to modify the format of a portion of the Environmental TS for clarity. We have evaluated the proposed changes.

Evaluation

The revised offsite organization will establish a management position at the corporate offices of FPC which deals solely with operation of nuclear plants. This position is the Manager-Nuclear Operations. Previously the Nuclear Plant Manager and the Manager-Nuclear Support Services reported to the Director-Power Production whose office was also responsible for non-nuclear related power production operations. In the modified organization, the Nuclear Plant Manager and the Manager-Nuclear Support Services report to the Manager-Nuclear Operations.

The revised offsite organization should enhance corporate level management attention to the operation of CR-3 and is acceptable.

By letter dated July 21, 1977, the licensee proposed a change to the format of the Environmental TS to more clearly define the functions and responsibilities of the Nuclear General Review Committee in environmental matters. This change clarifies the Environmental TS and is acceptable.

Currently FPC is required to provide prompt notification to NRC of short term reactivity increases corresponding to a reactor period of less than 5 seconds. Reactor period is the time required for neutron density (flux) to change by a factor of about 2.7.

FPC has proposed to change the units for rate of change of power from seconds (reactor period) to decades per minute (startup rate) since startup rate meters, instead of reactor period meters, are used at the plant. The revised TS would require reporting a startup rate greater than 5.2 decades per minute.

Reactor period (T) and startup rate (SUR) are directly related by the formula:

$$\text{SUR} = \frac{26.06}{T}$$

We have determined that the proposed change does not alter the rate of change of power required to be reported and is therefore acceptable.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: November 7, 1979

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-302FLORIDA POWER CORPORATIONCITY OF ALACHUACITY OF BUSHNELLCITY OF GAINESVILLECITY OF KISSIMMEECITY OF LEESBURGCITY OF NEW SMYRNA BEACH AND UTILITIES COMMISSION, CITY OF NEW SMYRNA BEACHCITY OF OCALAORLANDO UTILITIES COMMISSION AND CITY OF ORLANDOSEBRING UTILITIES COMMISSIONSEMINOLE ELECTRIC COOPERATIVE, INC.CITY OF TALLAHASSEENOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 25 to Facility Operating License No. DPR-72, issued to the Florida Power Corporation, City of Alachua, City of Bushnell, City of Gainesville, City of Kissimmee, City of Leesburg, City of New Smyrna Beach and Utilities Commission, City of New Smyrna Beach, City of Ocala, Orlando Utilities Commission and City of Orlando, Sebring Utilities Commission, Seminole Electric Cooperative, Inc., and the City of Tallahassee (the licensees) which revised the Technical Specifications for operation for the Crystal River Unit No. 3 Nuclear Generating Plant (the facility) located in Citrus County, Florida. The amendment is effective as of the date of issuance.

This amendment modifies the Technical Specifications to indicate a modified offsite organization, clarify the functions of the Nuclear General Review Committee in environmental matters, and change the units for the high rate of power increase reporting requirement.

The applications for the amendment comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR § 51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the applications for amendment dated July 21, 1977, September 22, 1978, and May 23, 1979, (2) Amendment No. 25 to License No. DPR-72, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C., and at the Crystal River Public Library, Crystal River, Florida. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 7th day of November 1979.

FOR THE NUCLEAR REGULATORY COMMISSION

Morton B. Fairtile

Morton B. Fairtile, Acting Chief
Operating Reactors Branch #4
Division of Operating Reactors