

October 10, 1989

Docket No. 50-302

DISTRIBUTION
See attached sheet

Mr. W. S. Wilgus
Vice President, Nuclear Operations
Florida Power Corporation
ATTN: Manager, Nuclear Licensing
P. O. Box 219
Crystal River, Florida 32629

Dear Mr. Wilgus:

SUBJECT: CRYSTAL RIVER UNIT 3 - ISSUANCE OF AMENDMENT RE: REMOVAL OF ORGANIZATIONAL CHARTS (TAC NO. 74194)

The Commission has issued the enclosed Amendment No. 122 to Facility Operating License No. DPR-72 for the Crystal River Unit No. 3 Nuclear Generating Plant (CR-3). This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated July 26, 1989.

This amendment removes Figures 6.2-1, Corporate Organization, and 6.2-2, Facility Organization, from the TS and replaces them with narrative descriptions. It also removes the requirement that members of the plant review committee have specific job titles and replaces it with a requirement that they be supervisors from various disciplines.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

15/

Harley Silver, Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 122 to DPR-72
- 2. Safety Evaluation

cc w/enclosures:
See next page

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LA: ~~PDII-2~~
DMiller
09/15/89

PE: ~~PDII-2~~
GWunder:jd
09/15/89

PM: ~~PDII-2~~
HSilver
9/18/89

D: ~~PDII-2~~
HBenkow
9/21/89

OTSB
JCalvo
9/19/89

OGC
subject to changes as marked

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DLangford
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

October 10, 1989

Docket No. 50-302

Mr. W. S. Wilgus
Vice President, Nuclear Operations
Florida Power Corporation
ATTN: Manager, Nuclear Operations
Licensing
P. O. Box 219-NA-2I
Crystal River, Florida 32629

Dear Mr. Wilgus:

SUBJECT: CRYSTAL RIVER UNIT 3 - ISSUANCE OF AMENDMENT RE: REMOVAL OF
ORGANIZATIONAL CHARTS (TAC NO. 74194)

The Commission has issued the enclosed Amendment No. 122 to Facility Operating License No. DPR-72 for the Crystal River Unit No. 3 Nuclear Generating Plant (CR-3). This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated July 26, 1989.

This amendment removes Figures 6.2-1, Corporate Organization, and 6.2-2, Facility Organization, from the TS and replaces them with narrative descriptions. It also removes the requirement that members of the plant review committee have specific job titles and replaces it with a requirement that they be supervisors from various disciplines.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely

A handwritten signature in black ink, appearing to read "Harley Styer", written over a large, stylized flourish.

Harley Styer, Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 122 to DPR-72
2. Safety Evaluation

cc w/enclosures:
See next page

DATED: October 10, 1989

AMENDMENT NO. 122 TO FACILITY OPERATING LICENSE NO. DPR-72-CRYSTAL RIVER UNIT 3

Docket File - 50-302 ←

- NRC & Local PDRs
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- S. Varga, 14/E/4 *||*
- G. Lainas, 14/H/3
- H. Berkow
- D. Miller
- H. Silver
- G. Wunder
- OGC-WF
- D. Hagan, 3302 MNBB
- E. Jordan, 3302 MNBB
- B. Grimes, 9/A/2
- T. Meek(4), P1-137
- Wanda Jones, P-130A
- J. Calvo, 11/F/23
- D. Langford, 14/B/2
- J. Miller, 11/F/23
- ACRS (10)
- GPA/PA
- OC/LFMB
- M. Sinkule, R-II

cc: Plant Service list

Mr. W. S. Wilgus
Florida Power Corporation

Crystal River Unit No. 3 Nuclear
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER CORPORATION
CITY OF ALACHUA
CITY OF BUSHNELL
CITY OF GAINESVILLE
CITY OF KISSIMMEE
CITY OF LEESBURG
CITY OF NEW SMYRNA BEACH AND UTILITIES COMMISSION, CITY OF NEW SMYRNA BEACH
CITY OF OCALA
ORLANDO UTILITIES COMMISSION AND CITY OF ORLANDO
SEBRING UTILITIES COMMISSION
SEMINOLE ELECTRIC COOPERATIVE, INC.
CITY OF TALLAHASSEE

DOCKET NO. 50-302

CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 122
License No. DPR-72

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power Corporation, et al. (the licensees) dated July 26, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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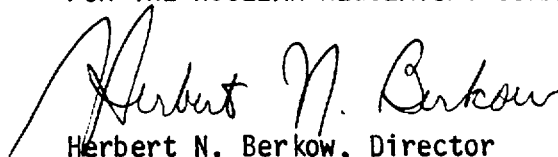
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-72 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 122, are hereby incorporated in the license. Florida Power Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 10, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 122

FACILITY OPERATING LICENSE NO. DPR-72

DOCKET NO. 50-302

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Remove

6-1
6-1a
6-2
6-3
6-5

Insert

6-1
6-1a

6-5

6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Director, Nuclear Plant Operations shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

6.1.2 The Director, Nuclear Plant Operations shall be responsible for an annual fire protection inspection which shall be performed utilizing either qualified offsite licensee personnel or an outside fire protection firm. The inspection shall consist of: a) an inspection of safety-related areas of the Plant to verify that they are in conformance with the fire hazards analysis; and b) a review of the Fire Brigade organization, training, and drills to verify their conformance with the requirements of Section 27 of the NFPA Code-1976.

6.1.3 The Director, Nuclear Plant Operations shall be responsible for an inspection of the fire protection program to be performed by a qualified outside fire consultant at least once per 36 months.

6.2 ORGANIZATION

OFFSITE

6.2.1 OFFSITE AND ONSITE ORGANIZATIONS

Onsite and offsite organizations shall be established for facility operation and corporate management, respectively. The onsite and offsite organizations shall include the positions for activities affecting the safety of the nuclear power plant.

- a. Lines of authority, responsibility, and communication shall be established and defined from the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of department responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the Final Safety Analysis Report.
- b. The Director, Nuclear Plant Operations shall be responsible for overall unit safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.

- c. The Vice President, Nuclear Operations shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.
- d. The individuals who train the operating staff and those who carry out health physics and quality assurance functions may report to the appropriate onsite manager; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.

6.2.2 FACILITY STAFF

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b. At least one licensed Operator shall be in the control room when fuel is in the reactor.
- c. At least two licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.
- d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
- e. All CORE ALTERATIONS after the initial fuel loading shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
- f. A site Fire Brigade of at least 5 members shall be maintained onsite at all times. # The Fire Brigade shall not include 5 members of the minimum shift crew necessary for safe shutdown of the unit and any personnel required for other essential functions during a fire emergency.
- g. The shift supervisors shall hold a senior reactor operator license. The operators required to have a license shall hold at least a reactor operator license.

Fire Brigade composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of Fire Brigade members provided immediate action is taken to restore the Fire Brigade to within the minimum requirements.

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TABLE 6.2-1**MINIMUM SHIFT CREW COMPOSITION**

LICENSE CATEGORY	APPLICABLE MODES	
	1, 2, 3, & 4	5 & 6
SOL	2	1*
OL	2	1
Non-Licensed	3	1
Operations Tech. Advisor	1	0

*Does not include the licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling Individual supervising CORE ALTERATIONS after the initial fuel loading.

#Shift crew composition may be less than the minimum requirement for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements of Table 6.2-1.

ADMINISTRATIVE CONTROLS

6.3 FACILITY STAFF QUALIFICATIONS

6.3 FACILITY STAFF QUALIFICATIONS

- 6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the Chemistry and Radiation Protection Superintendent who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975, and the Operations Technical Advisor, who shall have a Bachelor's degree, or the equivalent, in a scientific or engineering discipline with specific training in plant design and response and analysis of the plant for transients and accidents.

6.4 TRAINING

- 6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Vice President, Nuclear Operations and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and 10 CFR Part 55.
- 6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Vice President, Nuclear Operations and shall meet or exceed the requirements of Section 27 of the NFPA Code-1976. except for Fire Brigade training sessions which shall be held at least quarterly.

6.5 REVIEW AND AUDIT

6.5.1 PLANT REVIEW COMMITTEE (PRC)

FUNCTION

- 6.5.1.1 The Plant Review Committee shall function to advise the Director, Nuclear Plant Operations (DNPO) on all matters related to nuclear safety.

COMPOSITION

- 6.5.1.2 The Plant Review Committee shall be composed of nine members and one chairman from Nuclear Operations Supervisory Personnel responsible for the following areas: operations, health physics, security, maintenance, modifications, engineering, nuclear safety, and quality. These positions will be designated by the DNPO in Administrative Procedures.

ALTERNATES

- 6.5.1.3 All alternate members shall be appointed in writing by the PRC Chairman to serve on a temporary basis; no more than two alternates shall participate as voting members in PRC activities at any one time.

MEETING FREQUENCY

- 6.5.1.4 The PRC shall meet at least once per calendar month and as convened by the PRC Chairman or his designated alternate.

ADMINISTRATIVE CONTROLS

QUORUM

6.5.1.5 A quorum of the PRC shall consist of the Chairman or his designated alternate and five members including alternates.

RESPONSIBILITIES

6.5.1.6 The Plant Review Committee shall be responsible for:

- a. Review of 1) all procedures and changes thereto as required by Specification 6.8.2, 2) and other proposed procedures or changes thereto as determined by the Director, Nuclear Plant Operations to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to the Appendix "A" Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety, and changes to radwaste systems which could significantly alter their ability to meet Appendix I.
- e. Investigation of all violations of the Technical Specifications including the review and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Vice President, Nuclear Operations and to the Chairman of the Nuclear General Review Committee.
- f. Review of all REPORTABLE EVENTS.
- g. Review of facility operations to detect potential nuclear safety hazards.
- h. Performance of special reviews, investigations, or analyses and reports thereon as requested by the Chairman of the Nuclear General Review Committee.
- i. Review of the Plant Security Plan and implementing procedures.
- j. Review of the Emergency Plan and implementing procedures.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 122 TO FACILITY OPERATING LICENSE NO. DPR-72
FLORIDA POWER CORPORATION, ET AL.
CRYSTAL RIVER UNIT NO. 3 NUCLEAR GENERATING PLANT
DOCKET NO. 50-302

INTRODUCTION

By letter dated July 26, 1989, Florida Power Corporation (FPC or the licensee) proposed changes to Technical Specifications (TS) 6.2.1, Offsite Organizations, and 6.2.2, Unit Staff. The proposed changes would remove Figure 6.2-1, Corporate Organization, and Figure 6.2-2, Facility Organization, and replace them with a narrative description of the offsite and onsite organizations' functional requirements in TS 6.2.1 and 6.2.2, respectively. Guidance for these proposed changes to the TS was provided to licensees and applicants by Generic Letter 88-06 dated March 22, 1988.

The amendment would also allow for revision of the composition of the Plant Review Committee by replacing the requirement that its members have specific job titles with a more general requirement that they be supervisors from various disciplines. This issue was not addressed in the Generic Letter.

BACKGROUND

Consistent with the guidance provided in the Standard Technical Specifications, Specifications 6.2.1 and 6.2.2 of the administrative control requirements have referenced offsite and unit onsite organization charts that are provided as figures to these sections. On a plant-specific basis, these organization charts have been provided by applicants and included in the TS issued with the operating license. Subsequent restructuring of either the offsite or unit onsite organizations, following the issuance of an operating license, has required licensees to submit a license amendment for NRC approval to reflect the desired changes in these organizations.

Because of these limitations on organizational structure, the nuclear industry has highlighted this as an area for improvement in the TS. The Shearon Harris licensee proposed changes to remove organization charts from its TS under the lead-plant concept. The proposed changes were endorsed by the Westinghouse Owners Group. In its review of the Shearon Harris proposal, the staff concluded that most of the essential elements of offsite and onsite organization charts are captured by other regulatory requirements, notably Appendix B to 10 CFR 50; however, there were aspects of the organizational structure that are important to ensure that the administrative control requirements of 10 CFR 50.36 would be met and that would not be retained with the removal of the organizational

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charts. The applicable regulatory requirements are those administrative controls that are necessary to ensure safe operation of the facility. Therefore, those aspects of organization charts for Shearon Harris that were essential for conformance with regulatory requirements were added (1) to Specification 6.2.1 to define functional requirements for the offsite and onsite organizations and (2) to Specification 6.2.2 to define qualification requirements of the unit staff.

By letter dated January 27, 1988, the staff issued Amendment No. 3 to Facility Operating License NFP-63 for the Shearon Harris Nuclear Power Plant that incorporated these changes to their TS. Subsequently the staff developed guidance on an acceptable format for license amendment requests to remove the organization charts from the TS. Generic Letter 88-06 provided this guidance to all power reactor licensees.

EVALUATION

The licensee's proposed changes to its TS are in accordance with the guidance provided by Generic Letter 88-06 and address the items listed below.

- (1) Specifications 6.2.1 and 6.2.2 were revised to delete the references to Figures 6.2-1 and 6.2-2, which were removed from the TS.
- (2) Functional requirements of the offsite and onsite organizations were defined and added to Specification 6.2.1, and they are consistent with the guidance provided in Generic Letter 88-06. The specification notes that implementation of these requirements is documented in the Final Safety Analysis Report.
- (3) The senior reactor operator and reactor operator license qualified positions of the unit staff were added to Specification 6.2.2. Therefore, this requirement, which was identified on the organization chart for the unit staff, will be retained.
- (4) Consistent with requirements to document the offsite and onsite organization relationships in the form of organization charts, the licensee has confirmed that this documentation currently exists in the Final Safety Analysis Report.
- (5) The licensee has confirmed that no specifications, other than those noted in item (1) above, include references to the figures of the organization charts that are being removed from TS for their plant. Hence, this is not an applicable consideration with regard to the need to redefine referenced requirements as a result of the removal of these figures.

The deletion of the specific titles for members of the Plant Review Committee is considered to be administrative in nature, as a requirement for supervisors from various disciplines to compose this Committee has been added to the TS. The details of the specific job titles of the Plant Review Committee members will be located in administrative procedures. Changes to these procedures are made in accordance with 10 CFR 50.59.

On the basis of its review of the above items, the staff concludes that the licensee has provided an acceptable response to these items as addressed in the NRC guidance on removing organization charts from the administrative control requirements of the TS. Furthermore, the staff finds that these changes are consistent with the staff's generic finding on the acceptability of such changes as noted in Generic Letter 88-06. Accordingly, the staff finds the proposed changes to be acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment relates to changes in record keeping, or administrative procedures or requirements. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

CONCLUSION

We have concluded, based on the considerations discussed above, that:
(1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors:

T. Dunning
G. Wunder

Dated: October 10, 1989