

April 3, 1990

Docket No. 50-302

Mr. Percy M. Beard, Jr.
Senior Vice President,
Nuclear Operations
Florida Power Corporation
ATTN: Manager, Nuclear Operations
Licensing
P.O. Box 219-NA-2I
Crystal River, Florida 32629

Dear Mr. Beard:

SUBJECT: CRYSTAL RIVER UNIT 3 - CORRECTION TO AMENDMENT NO. 125
(TAC NO. 73356)

On January 31, 1990, the Commission issued Amendment No. 125 for the Crystal River Unit 3 Nuclear Generating Plant. The amendment replaced the cycle-specific parameter limits with a reference to the Core Operating Limits Report in the Crystal River Technical Specifications.

Due to an administrative error, page 6-15, item 9 did not contain the date of the Safety Evaluation Report issued with the amendment. Enclosed is the corrected page 6-15, as well as the corresponding overleaf page.

Sincerely,

Original Signed By

Harley Silver, Project Manager
Project Directorate II-2
Division of Reactor Projects-I/II
Office of Nuclear Reactor Regulation

Enclosure:
As stated

cc w/enclosure:
See next page

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Mr. Percy M. Beard, Jr.
Florida Power Corporation

Crystal River Unit No. 3 Nuclear
Generating Plant

cc:

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cc: Plant Service List

ADMINISTRATIVE CONTROLS

MONTHLY OPERATING REPORT

6.9.1.6 Routine reports of operating statistics and shutdown experience shall be submitted on a monthly basis to the Document Control Desk, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, with a copy to the Regional Office, submitted no later than the 15th of each month following the calendar month covered by the report.

CORE OPERATING LIMITS REPORT

6.9.1.7 Core operating limits shall be established and documented in the CORE OPERATING LIMITS REPORT before each reload cycle and any remaining part of a reload cycle for the following:

- 3.1.3.6 Regulating Rod Insertion Limits
- 3.1.3.7 Rod Program
- 3.1.3.9 Axial Power Shaping Rod Insertion Limits
- 3.2.1 AXIAL POWER IMBALANCE
- 3.2.4 QUADRANT POWER TILT

The analytical methods used to determine the core operating limits addressed by the individual Technical Specifications shall be those previously reviewed and approved by the NRC, specifically:

- 1) BAW-10122A Rev. 1, "Normal Operating Controls", May 1984
- 2) BAW-10116A, "Assembly Calculations and Fitted Nuclear Data", May 1977
- 3) BAW-10117P-A, "Babcock & Wilcox Version of PDQ User's Manual", January 1977
- 4) BAW-10118A, "Core Computational Techniques and Procedures", December 1979.
- 5) BAW-10124A, "FLAME 3 - A Three-Dimensional Nodal Code for Calculating Core Reactivity and Power Distributions", August 1976
- 6) BAW-10125A, "Verification of Three-Dimensional FLAME Code", August 1976
- 7) BAW-10152A, "NOODLE - A Multi-Dimensional Two-Group Reactor Simulator", June 1985
- 8) BAW-10119, "Power Peaking Nuclear Reliability Factors", June 1977
- 9) The methodology for Rod Program received NRC approval in the Safety Evaluation Report dated January 31, 1990.

The core operating limits shall be determined so that all applicable limits (e.g., fuel thermal-mechanical limits, core thermal-hydraulic limits, ECCS

ADMINISTRATIVE CONTROLS

limits, nuclear limits such as shutdown margin, and transient and accident analysis limits) of the safety analysis are met.

The CORE OPERATING LIMITS REPORT, including any mid-cycle revision or supplements thereto, shall be provided upon issuance for each reload cycle to the NRC Document Control Desk with copies to the Regional Administrator and Resident Inspector.