

April 9, 1990

Docket No. 50-302

DISTRIBUTION
See attached sheet

Mr. Percy M. Beard, Jr.
Senior Vice President,
Nuclear Operations
Florida Power Corporation
ATTN: Manager, Nuclear Operations
Licensing
P. O. Box 219-NA-2I
Crystal River, Florida 32629

Dear Mr. Beard:

SUBJECT: CRYSTAL RIVER UNIT 3 - ISSUANCE OF EMERGENCY AMENDMENT RE:
DECAY HEAT REMOVAL (TAC NO. 76362)

The Commission has issued the enclosed Amendment No. 127 to Facility Operating License No. DPR-72 for the Crystal River Unit No. 3 Nuclear Generating Plant (CR-3). This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated March 26, 1990.

This amendment adds a footnote to TS 3.9.8.2 which allows either the normal or emergency power source for each decay heat loop to be inoperable when in Mode 6 with the water level above the top of the fuel but less than 23 feet. For purposes of clarity, the footnote has been changed slightly from that in your request. This change has been discussed with and agreed to by members of your staff. A Temporary Waiver of Compliance was issued on March 26, 1990 to permit utilization of this change until this amendment could be issued.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance and Final Determination of No Significant Hazards Consideration and Opportunity for Hearing will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Harley Silver, Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 127 to DPR-72
2. Safety Evaluation

cc w/enclosures:
See next page

*See previous concurrence

OFC :	LA: PD22	: PM: PD22	: PM: PD22	: D: PD22	: OGC *	: ADR2	:
NAME :	D Miller	: GWunder:sl	: HSilver	: HBerkow	:	: GLathas	:
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Mr. Percy M. Beard, Jr.
Florida Power Corporation

Crystal River Unit No. 3 Nuclear
Generating Plant

cc:

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DATED: April 9, 1990

AMENDMENT NO. 127 TO FACILITY OPERATING LICENSE NO. DPR-72-CRYSTAL RIVER UNIT 3

Docket File

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER CORPORATION
CITY OF ALACHUA
CITY OF BUSHNELL
CITY OF GAINESVILLE
CITY OF KISSIMMEE
CITY OF LEESBURG
CITY OF NEW SMYRNA BEACH AND UTILITIES COMMISSION, CITY OF NEW SMYRNA BEACH
CITY OF OCALA
ORLANDO UTILITIES COMMISSION AND CITY OF ORLANDO
SEBRING UTILITIES COMMISSION
SEMINOLE ELECTRIC COOPERATIVE, INC.
CITY OF TALLAHASSEE

DOCKET NO. 50-302

CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 127
License No. DPR-72

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power Corporation, et al. (the licensees) dated March 26, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-72 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 127, are hereby incorporated in the license. Florida Power Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Gus C. Lainas, Assistant Director
for Region II Reactors
Division of Reactor Projects-I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 9, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 127

FACILITY OPERATING LICENSE NO. DPR-72

DOCKET NO. 50-302

Replace the following page of the Appendix "A" Technical Specifications with the attached page. The revised page is identified by amendment number and contains vertical lines indicating the area of change.

Remove

3/4 9-8a

Insert

3/4 9-8a

REFUELING OPERATIONS

LOW WATER LEVEL

LIMITING CONDITION FOR OPERATION

3.9.8.2 Two decay heat removal (DHR) loops shall be OPERABLE.*

APPLICABILITY: MODE 6 when the water level above the top of the irradiated fuel assemblies seated within the reactor pressure vessel is less than 23 feet.

ACTION

- a. With less than the two required DHR loops OPERABLE, initiate corrective action to return the required loops to OPERABLE status.
- b. The provisions of Specification 3.0.3 are not applicable.

SURVEILLANCE REQUIREMENTS

4.9.8.2 No additional surveillance requirements other than those required by Specification 4.0.5.

*The normal or emergency power source may be inoperable for each DHR loop, as limited by Specification 3.8.1.2.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 127 TO FACILITY OPERATING LICENSE NO. DPR-72

FLORIDA POWER CORPORATION, ET AL.

CRYSTAL RIVER UNIT NO. 3 NUCLEAR GENERATING PLANT

DOCKET NO. 50-302

BACKGROUND

By letter dated March 26, 1990, Florida Power Corporation (FPC or the licensee) requested an amendment on an emergency basis to the Technical Specifications (TS) appended to Facility Operating License No. DPR-72 for the Crystal River Unit No. 3 Nuclear Generating Plant (CR-3). The proposed amendment would add a footnote to Technical Specification 3.9.8.2 that would allow either the normal or emergency power source for each decay heat loop to be inoperable when in Mode 6 with the water level above the top of the fuel but less than 23 feet. A Temporary Waiver of Compliance was issued on March 26, 1990 to permit utilization of this change until the amendment could be issued.

EVALUATION

By letter dated June 11, 1980, the Commission requested that licensees submit TS that would provide redundancy in decay heat removal capability in all modes. A Babcock & Wilcox Standard Technical Specification (STS) was enclosed for use by licensees in preparing their amendment requests. The STS included a footnote allowing either the normal or emergency power supply for each decay heat loop to be inoperable in Mode 6 (refueling) with the water level less than 23 feet above the top of the fuel.

An amendment request intended to make the Crystal River Unit 3 TS consistent with both the Commission's guidance on decay heat loop redundancy and the Babcock & Wilcox Standard Technical Specifications was submitted, and Amendment No. 117 was issued on May 31, 1989. The licensee, however, inadvertently omitted the footnote. The amendment request currently being considered would add such a footnote, which, for purposes of clarity, has been modified slightly to refer to the limitations of TS 3.8.1.2.

The footnote is consistent with the Commission's guidance on decay heat loop redundancy, was in the sample STS provided for licensee use, and would, therefore, have been approved in Amendment 117 had it been requested. By inadvertently omitting the footnote from their original amendment request, the licensee has placed an unnecessary restriction on itself. Therefore, the staff has concluded that the proposed change as modified is acceptable.

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SUMMARY

Based on our review, the change proposed in this request, as modified, is adequate and acceptable.

EMERGENCY CIRCUMSTANCES

On March 14, 1990, Crystal River Unit 3 commenced shutdown for Refuel 7. As a part of this refueling outage, extensive modifications to the diesel generators are to be performed. These modifications will require that one diesel generator be inoperable for much of the outage.

On March 22, 1990, with one diesel generator inoperable, the licensee realized that TS 3.9.8.2 would prevent the plant from entering Mode 6, as the specification did not have a note allowing an emergency power supply for a decay heat loop to be inoperable. This section of the TS was amended on May 31, 1989. This is the first time an entry into Mode 6 has been planned since the issuance of that amendment.

Since the licensee's original amendment submittal placed an unnecessary restriction on plant operations through an administrative error, and since this is the first attempt at entering Mode 6 since the issuance of the amendment, the staff believes that the licensee could not reasonably have identified the situation earlier. Therefore, the staff has determined that the licensee has not abused the emergency provisions of 10 CFR 50.91(a)(5) in this instance. Since the issuance of this amendment will prevent the unnecessary extension of the refueling outage, the Commission has determined that there are emergency circumstances warranting prompt approval.

FINAL NO SIGNIFICANT HAZARDS CONSIDERATION DETERMINATION

The Commission's regulations in 10 CFR 50.92 state that the Commission may make a final determination that a license amendment involves no significant hazards considerations if operability of the facility in accordance with the proposed changes would not:

1. involve a significant increase in the probability or consequences of any accident previously evaluated, or
2. create the possibility of a new or different kind of accident from any accident previously evaluated, or
3. involve a significant reduction in a margin of safety.

The licensee evaluated the request in light of these three criteria. In regard to the first criterion, the licensee determined that the proposed amendment would not involve a significant increase in the probability or consequences of any accident previously evaluated since redundant decay heat loops will be provided. Each loop will be provided with adequate assurance of a reliable power source.

In regard to the second criterion, the licensee determined that the proposed amendment will not create the possibility for a new or different kind of accident from any accident previously evaluated since adequate decay heat removal capability is maintained with redundant decay heat removal loop operability. Therefore, no new accident conditions are created as a result of this request.

In regard to the third criterion, the licensee determined that the proposed amendment will not involve a significant reduction in a margin of safety since there will be adequate assurance of a reliable power supply to each of the redundant decay heat removal loops. Furthermore, the proposed amendment is consistent with the Commission's guidance and with the Babcock & Wilcox Standard Technical Specifications.

The staff has evaluated the licensee's submittal and agrees that it satisfies the standards of 10 CFR 50.92. Therefore, the staff has made a final determination that the proposed amendment involves no significant hazards considerations.

ENVIRONMENTAL CONSIDERATION

This amendment involves the change in a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has made a final no significant hazards consideration determination with respect to this amendment. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) this emergency situation could not be avoided; (2) the licensee acted in a timely manner with respect to responding to this emergency; (3) the amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, does not create the possibility of a new or different type of accident from any evaluated previously, and does not involve a significant reduction in the margin of safety, (4) there is reasonable assurance that the

health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: April 9, 1990

Principal Contributors:

H. Silver
G. Wunder