

September 13, 1982

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Docket No. 50-302

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Mr. J. A. Hancock
Vice President
Nuclear Operations
Florida Power Corporation
ATTN: Manager, Nuclear Operations
P. O. Box 14042; M.A.C. H-2
St. Petersburg, Florida 33733

Dear Mr. Hancock:

The Commission has issued the enclosed Amendment No. 57 to Facility Operating License No. DPR-72 for the Crystal River Unit No. 3 Nuclear Generating Plant (CR-3). This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated December 3, 1980 (Change Request 67), as revised January 19, 1982.

The amendment modifies the Administrative Controls Section of the TSs to reflect an expanded plant review committee membership, an expanded procedural review and current title designations for CR-3 management.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,



Sydney Miner, Project Manager
Operating Reactors Branch #4
Division of Licensing

Enclosures:

1. Amendment No. 57
2. Safety Evaluation
3. Notice

cc w/enclosures:
See next page

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IFR NOTICE
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AMENDMENT

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| OFFICE | OELD |
| SURNAME | M. KARMAK |
| DATE | 9/7/82 |

Crystal River Unit No. 3
Florida Power Corporation

50-302

cc w/enclosure(s):
Mr. S. A. Brandimore
Florida Power Corporation
Vice President and General Counsel
P. O. Box 14042
St. Petersburg, Florida 33733

Mr. Wilbur Langely, Chairman
Board of County Commissioners
Citrus County
Iverness, Florida 36250

Regional Radiation Representative
EPA Region IV
345 Courtland Street, N.E.
Atlanta, Georgia 30308

Crystal River Public Library
668 N. W. First Avenue
Crystal River, Florida 32629

Mr. Robert B. Borsum
Babcock & Wilcox
Nuclear Power Generation Division
Suite 220, 7910 Woodmont Avenue
Bethesda, Maryland 20814

Mr. Tom Stetka, Resident Inspector
U.S. Nuclear Regulatory Commission
Route #3, Box 717
Crystal River, Florida 32629

Mr. T. C. Lutkehaus
Nuclear Plant Manager
Florida Power Corporation
P. O. Box 219
Crystal River, Florida 32629

cc w/enclosure(s) & incoming dtd.:
12/3/80, 1/19/82
Bureau of Intergovernmental Relations
660 Apalachee Parkway
Tallahassee, Florida 32304

Administrator
Department of Environmental Regulation
Power Plant Siting Section
State of Florida
2600 Blair Stone Road
Tallahassee, Florida 32301

Attorney General
Department of Legal Affairs
The Capitol
Tallahassee, Florida 32304

Mr. James P. O'Reilly, Regional Administrator
U. S. Nuclear Regulatory Commission, Region II
101 Marietta Street, Suite 3100
Atlanta, Georgia 30303



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER CORPORATION
CITY OF ALACHUA
CITY OF BUSHNELL
CITY OF GAINESVILLE
CITY OF KISSIMMEE
CITY OF LEESBURG
CITY OF NEW SMYRNA BEACH AND UTILITIES COMMISSION, CITY OF NEW SMYRNA BEACH
CITY OF OCALA
ORLANDO UTILITIES COMMISSION AND CITY OF ORLANDO
SEBRING UTILITIES COMMISSION
SEMINOLE ELECTRIC COOPERATIVE, INC.
CITY OF TALLAHASSEE

DOCKET NO. 50-302

CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 57
License No. DPR-72

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power Corporation, et al (the licensees) dated December 3, 1980, as revised January 19, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

September 13, 1982

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RIngram

Docket No. 50-302

Docketing and Service Section
Office of the Secretary of the Commission

SUBJECT: CRYSTAL RIVER UNIT 3

Two signed originals of the Federal Register Notice identified below are enclosed for your transmittal to the Office of the Federal Register for publication. Additional conformed copies (12) of the Notice are enclosed for your use.

- Notice of Receipt of Application for Construction Permit(s) and Operating License(s).
- Notice of Receipt of Partial Application for Construction Permit(s) and Facility License(s); Time for Submission of Views on Antitrust Matters.
- Notice of Availability of Applicant's Environmental Report.
- Notice of Proposed Issuance of Amendment to Facility Operating License.
- Notice of Receipt of Application for Facility License(s); Notice of Availability of Applicant's Environmental Report; and Notice of Consideration of Issuance of Facility License(s) and Notice of Opportunity for Hearing.
- Notice of Availability of NRC Draft/Final Environmental Statement.
- Notice of Limited Work Authorization.
- Notice of Availability of Safety Evaluation Report.
- Notice of Issuance of Construction Permit(s).
- Notice of Issuance of Facility Operating License(s) or Amendment(s).

Other: Amendment No. 57.
Referenced documents have been provided PDR.

Division of Licensing, ORB#4
Office of Nuclear Reactor Regulation

Enclosure:
As Stated

| | | | | | |
|-----------|------------|--|--|--|--|
| OFFICE → | ORB#4:DL/V | | | | |
| SURNAME → | RIngram;cf | | | | |
| DATE → | 9/14/82 | | | | |

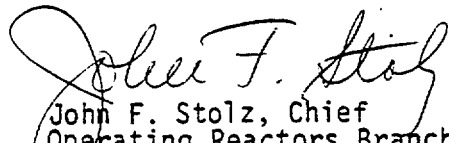
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-72 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 57, are hereby incorporated in the license. Florida Power Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment becomes effective 30 days after its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 13, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 57

FACILITY OPERATING LICENSE NO. DPR-72

DOCKET NO. 50-302

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

6-2

6-3

6-5

6-6

6-7

6-11

6-12

6-12a (new page)

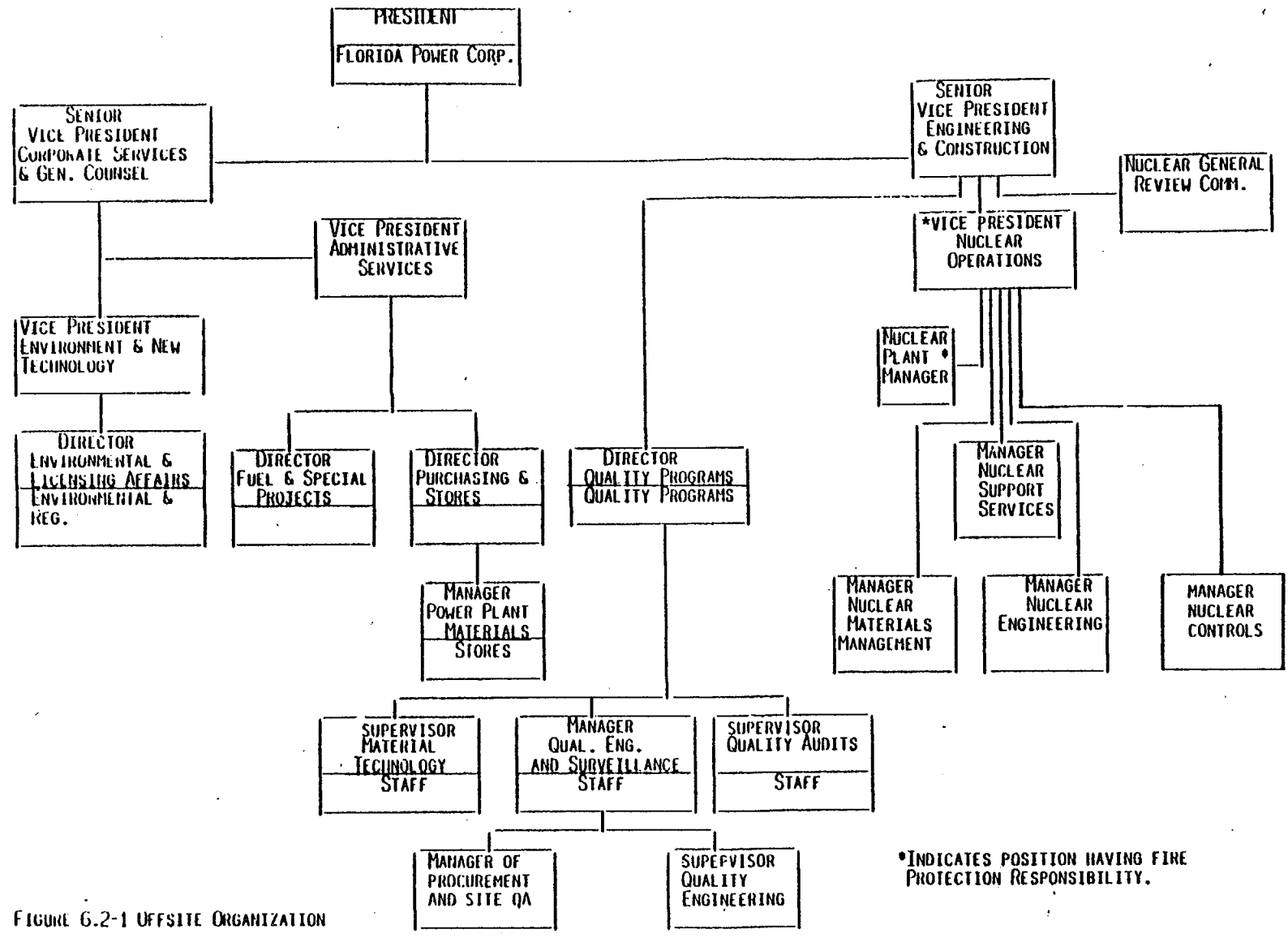
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CRYSTAL RIVER - UNIT 3

6-2

Amendment No. 8, 78, 28, 57



*INDICATES POSITION HAVING FIRE PROTECTION RESPONSIBILITY.

FIGURE 6.2-1 OFFSITE ORGANIZATION

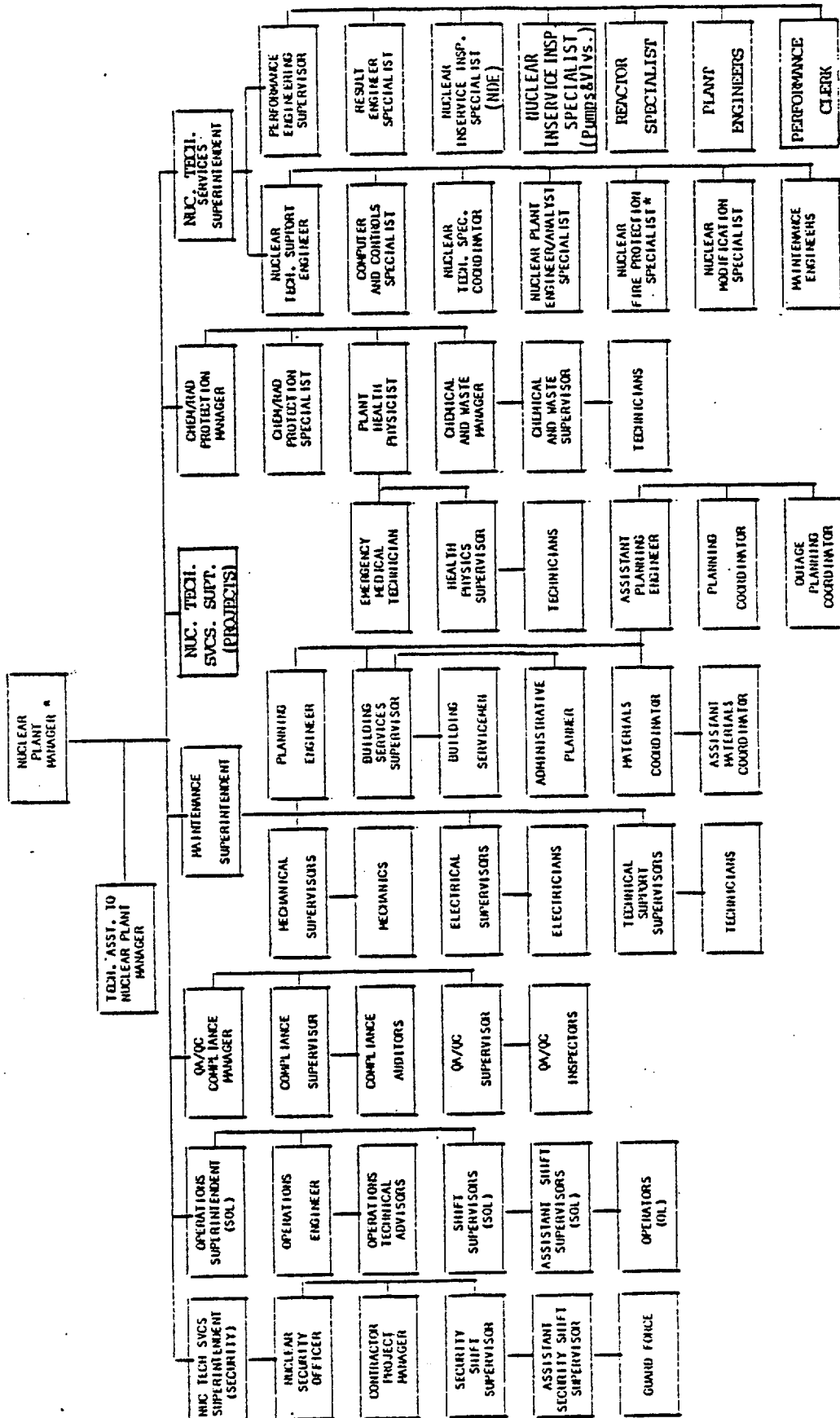


FIGURE 6.2-2 FACILITY ORGANIZATION

LEGEND:
 SOL Senior Operator License
 OL Operator License
 * Indicates position having fire protection responsibility

TABLE 6.2-1
MINIMUM SHIFT CREW COMPOSITION#

| LICENSE CATEGORY | APPLICABLE MODES | |
|--------------------------|------------------|-------|
| | 1, 2, 3, & 4 | 5 & 6 |
| SOL | 2 | 1* |
| OL | 2 | 1 |
| Non-Licensed | 3 | 1 |
| Operations Tech. Advisor | 1 | 0 |

*Does not include the licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling Individual supervising CORE ALTERATIONS after the initial fuel loading.

#Shift crew composition may be less than the minimum requirement for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements of Table 6.2-1.

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the Chemistry and Radiation Protection Manager who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975, and the Operations Technical Advisor, who shall have a Bachelor's degree, or the equivalent, in a scientific or engineering discipline with specific training in plant design and response and analysis of the plant for transients and accidents.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Nuclear Plant Manager and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Nuclear Plant Training Manager and shall meet or exceed the requirements of Section 27 of the NFPA Code-1976, except for Fire Brigade training sessions which shall be held at least quarterly.

6.5 REVIEW AND AUDIT

6.5.1 PLANT REVIEW COMMITTEE (PRC)

FUNCTION

6.5.1.1 The Plant Review Committee shall function to advise the Nuclear Plant Manager on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The Plant Review Committee shall be composed of the:

- Chairman: Technical Services Superintendent
- Member: Operations Superintendent
- Member: Maintenance Superintendent
- Member: Nuclear Technical Services Superintendent (Security)
- Member: QA/QC Compliance Manager
- Member: Chem/Rad Protection Manager
- Member: Technical Support Engineer
- Member: Performance Engineering Supervisor
- Member: At Large (Designated by Chairman)
- Member: At Large (Designated by Chairman)

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the PRC Chairman to serve on a temporary basis; no more than two alternates shall participate as voting members in PRC activities at any one time.

MEETING FREQUENCY

6.5.1.4 The PRC shall meet at least once per calendar month and as convened by the PRC Chairman or his designated alternate.

ADMINISTRATIVE CONTROLS

QUORUM

6.5.1.5 A quorum of the PRC shall consist of the Chairman or his designated alternate and five members including alternates.

RESPONSIBILITIES

6.5.1.6 The Plant Review Committee shall be responsible for:

- a. Review of 1) all procedures and changes thereto as required by Specification 6.8(2, 2) any other proposed procedures or changes thereto as determined by the Nuclear Plant Manager to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to the Appendix "A" Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Vice-President, Nuclear Operations and to the Chairman of the Nuclear General Review Committee.
- f. Review of events requiring 24-hour written notification to the Commission.
- g. Review of facility operations to detect potential nuclear safety hazards.
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman of the Nuclear General Review Committee.
- i. Review of the Plant Security Plan and implementing procedures.
- j. Review of the Emergency Plan and implementing procedures.

ADMINISTRATIVE CONTROLS

AUTHORITY

- 6.5.1.7 The Plant Review Committee shall:
- a. Recommend to the Nuclear Plant Manager written approval or disapproval of items considered under 6.5.1.6 (a) through (d) above.
 - b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6 (a) through (e) above constitutes an unreviewed safety question.
 - c. Provide written notification within 24 hours to the Vice President, Nuclear Operations and the Nuclear General Review Committee of disagreement between the PRC and the Nuclear Plant Manager; however, the Nuclear Plant Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

- 6.5.1.8 The Plant Review Committee shall maintain written minutes of each meeting and copies shall be provided to the Vice President, Nuclear Operations and Chairman of the Nuclear General Review Committee.

6.5.2 NUCLEAR GENERAL REVIEW COMMITTEE (NGRC)

FUNCTION

- 6.5.2.1 The Nuclear General Review Committee shall function to provide independent review and audit of designated activities in the areas of:
- a. Nuclear power plant operations
 - b. Nuclear engineering
 - c. Chemistry and radiochemistry
 - d. Metallurgy
 - e. Instrumentation and control
 - f. Radiological safety
 - g. Mechanical and electrical engineering
 - h. Quality assurance practices

ADMINISTRATIVE CONTROLS

COMPOSITION

6.5.2.2 The NGRC shall be composed of the Chairman, Vice Chairman, and at least 5 members. No more than a minority of the members shall have line responsibility for operation of the facility. The committee shall collectively have the experience and competence required to review problems in the following areas:

- a. Nuclear power plant operations
- b. Nuclear engineering
- c. Chemistry and radiochemistry
- d. Metallurgy
- e. Nondestructive testing
- f. Instrumentation and control
- g. Radiological safety
- h. Mechanical and electrical engineering
- i. Administrative controls
- j. Environmental
- k. Quality assurance practices

QUALIFICATIONS

6.5.2.3 The following minimum experience requirements shall be established for those persons involved in the independent off-site safety review and audit program:

- a. Chairman and Vice-Chairman-Bachelor of Science in engineering or related field and ten years related experience including five years involvement with operation and/or design of nuclear power plants.
- b. Member-Bachelor of Science in engineering or related field and five years related experience including three years involvement with operation and/or design of nuclear power plants.

ALTERNATES

6.5.2.4 All alternate members shall be appointed in writing by the NGRC Chairman to serve on a temporary basis; however no more than two alternates shall participate as voting members in NGRC activities at any one time.

ADMINISTRATIVE CONTROLS

RECORDS

6.5.2.11 Records of NGRC activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each NRGC meeting shall be prepared, approved and forwarded to the Senior Vice President, Engineering and Construction, within 14 days following each meeting.
- b. Reports of reviews encompassed by Section 6.5.2.8 above, shall be prepared, approved and forwarded to the Senior Vice President, Engineering and Construction, within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.9 above, shall be forwarded to the Senior Vice President, Engineering and Construction, and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.6 REPORTABLE OCCURRENCE ACTION

6.6.1 The following actions shall be taken for REPORTABLE OCCURRENCES:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.
- b. Each REPORTABLE OCCURRENCE requiring 24-hour notification to the Commission shall be reviewed by the PRC and submitted to the NGRC and the Vice President, Nuclear Operations.

6.7 SAFETY LIMIT VIOLATION

- 6.7.1 The following actions shall be taken in the event a Safety Limit is violated:
- a. The facility shall be placed in at least HOT STANDBY within one hour.
 - b. The Safety Limit violation shall be reported to the Commission, the Vice President, Nuclear Operations and to the NGRC within 24 hours.
 - c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PRC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures and (3) corrective action taken to prevent recurrence.
 - d. The Safety Limit Violation Report shall be submitted to the Commission, the NGRC and the Vice President, Nuclear Operations within 14 days of the violation.

6.8 PROCEDURES

- 6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:
- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, November, 1972.
 - b. Refueling operations.
 - c. Surveillance and test activities of safety related equipment.
 - d. Security Plan implementation.
 - e. Emergency Plan implementation.
 - f. Fire Protection Program implementation.
 - g. Systems Integrity Program implementation.
 - h. Iodine Monitoring Program implementation.
- 6.8.2 Each procedure and administrative policy of 6.8.1 above, and changes thereto, shall be reviewed and approved prior to implementation as follows:
- a. The Emergency Plan, Security Plan, Fire Protection Plan and implementing procedures, Administrative Instructions and those test procedures associated with plant modifications shall be reviewed and approved by the PRC and the Nuclear Plant Manager prior to implementation.

ADMINISTRATIVE CONTROLS

- b. For all other procedures, the review cycle shall consist of: an intradepartmental review by a Qualified Reviewer, an interdisciplinary review by Qualified Reviewer(s) in interfacing departments, as specified in Administrative Procedures, and approval by the responsible Superintendent or Manager, as specified by Administrative Procedures. The PRC shall then review the 10CFR 50.59 evaluation within 14 days of approval.
- c. The training and qualification of Qualified Reviewers shall be governed by Administrative Procedures, with final certification by the Nuclear Plant Manager. Recertification will be required on a periodic basis and upon transfer between departments. As a minimum, all Qualified Reviewers shall meet the requirements of ANSI N18.1-1971, Sections 4.2, 4.3, 4.4, or 4.6, or the equivalent.
- d. Each procedure and administrative policy of 6.8.1 shall be reviewed on a periodic basis as set forth in Administrative Procedures.

ADMINISTRATIVE CONTROLS

- 6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:
- a. The intent of the original procedure is not altered.
 - b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License.
 - c. The change is documented and subsequently reviewed and approved within 14 days of implementation, in accordance with the requirements of Specification 6.8.2.

6.9 REPORTING REQUIREMENTS

ROUTINE REPORTS AND REPORTABLE OCCURRENCES

- 6.9.1 In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following reports shall be submitted to the Director of the Regional Office of Inspection and Enforcement unless otherwise noted.

STARTUP REPORTS

- 6.9.1.1 A summary report of plant startup and power escalation testing will be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant.
- 6.9.1.2 The startup report shall address each of the tests identified in the FSAR and shall include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details requested in license conditions based on other commitments shall be included in this report.
- 6.9.1.3 Startup reports shall be submitted within (1) 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the Startup Report does not cover all three events (i.e., initial criticality, completion of startup test program, and the resumption or commencement of commercial power operation), supplementary reports shall be submitted at least every three months until all three events have been completed.

ADMINISTRATIVE CONTROLS

ANNUAL REPORTS

- 6.9.1.4 Annual reports covering the activities of the unit as described below for the previous calendar year shall be submitted prior to March 1 of each year. The initial report shall be submitted prior to March 1 of the year following initial criticality.
- 6.9.1.5 Reports required on an annual basis shall include:
- a. A tabulation of the number of station, utility, and other personnel (including contractors) receiving exposures greater than 100 mrem/yr and their associated man-rem exposure according to work and job functions,¹ e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. The dose assignments to various duty functions may be estimated based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20 percent of the individual total dose need not be accounted for. In the aggregate, at least 80 percent of the total whole body dose received from external sources should be assigned to specific major work functions.
 - b. A list of the reactor vessel material surveillance capsules installed in the reactor at the end of the report period and a summary of any withdrawals or insertions of capsules during the report period. In supplying this information, the ownership of each capsule shall be indicated and the irradiation location in the vessel of each capsule which was inserted during the report period shall be identified.

¹ This tabulation supplements the requirements of 20.407 of 10CFR Part 20.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 57 TO FACILITY OPERATING LICENSE NO. DPR-72
FLORIDA POWER CORPORATION, ET AL
CRYSTAL RIVER UNIT NO. 3 NUCLEAR GENERATING PLANT
DOCKET NO. 50-302

Introduction

By letter dated December 3, 1980, as revised January 19, 1982, Florida Power Corporation (FPC) proposed a change to the Crystal River Unit 3 (CR-3) Technical Specifications (TSs) to reflect an expanded Plant Review Committee (PRC) membership, an expanded procedural review and current title designations for CR-3 management.

Evaluation

The changes proposed by FPC will expand the PRC by adding the Nuclear Technical Services Superintendent, the QA/QC Compliance Manager and the Performance Engineering Supervisor. The proposed number of PRC members required for a quorum is expanded from four plus the chairman to five plus the chairman. Expanding the membership of the PRC to more technical areas and departments will increase the technical expertise of the PRC and the participation by all FPC departments. The proposed increase in the number of members required for a quorum will assure that issues are more completely considered with respect to impact on all CR-3 operations. Expanding the number of members on the committee will also assure greater continuity if individual members are absent. Therefore, this change is in the direction of improving the capability of the PRC.

The proposed TSs expand the procedural review requirements of the PRC to specifically require PRC review of the Emergency Plan, Security Plan, Fire Protection Plan and test procedures associated with plant modification. The proposed TSs also specify the review requirements for other procedures including the qualification of reviewers. These changes assure that each procedure is reviewed adequately by FPC management and technically qualified reviewers prior to implementation.

The revised review process clarifies the review process for procedures and places more review responsibility on lower level management at CR-3. The procedures do, however, retain adequate higher management participation in the review and approval process.

The proposed changes in titles of CR-3 management are administrative only and do not involve any safety issues.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant reduction in a margin of safety, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: September 13, 1982

The following NRC personnel have contributed to this Safety Evaluation:
P. Erickson.

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-302FLORIDA POWER CORPORATION, ET ALNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

!

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No.57 to Facility Operating License No. DPR-72, issued to the Florida Power Corporation, City of Alachua, City of Bushnell, City of Gainesville, City of Kissimmee, City of Leesburg, City of New Smyrna Beach and Utilities Commission, City of New Smyrna Beach, City of Ocala, Orlando Utilities Commission and City of Orlando, Sebring Utilities Commission, Seminole Electric Cooperative, Inc., and the City of Tallahassee (the licensees) which revised the Technical Specifications (TSs) for operation of the Crystal River Unit No. 3 Nuclear Generating Plant (the facility) located in Citrus County, Florida. The amendment becomes effective 30 days after its date of issuance.

This amendment modifies the Technical Specifications to reflect an expanded Plant Review Committee membership, an expanded procedural review and current title designations for CR-3 management.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in

-2-

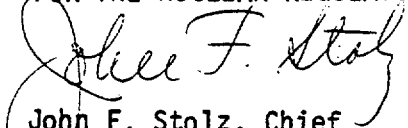
the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated December 3, 1980, as revised January 19, 1982, (2) Amendment No. 57 to License No. DPR-72, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, DC 20555, and at the Crystal River Public Library, 668 N.W. First Avenue, Crystal River, Florida. A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 13th day of September 1982.

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing