

October 12, 1982

DMB016

Docket No. 50-302

Mr. J. A. Hancock
Vice President
Nuclear Operations
Florida Power Corporation
ATTN: Manager, Nuclear Operations
P. O. Box 14042; M.A.C. H-2
St. Petersburg, Florida 33733

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Dear Mr. Hancock:

The Commission has issued the enclosed Amendment No. 58 to Facility Operating License No. DPR-72 for the Crystal River Unit 3 Nuclear Generating Plant. The amendment consists of changes to the Technical Specifications in response to your application dated February 19, 1982. Changes to your proposed Technical Specifications were discussed with and accepted by your staff.

The amendment pertains to the Environmental Technical Specifications (Appendix B to the Facility Operating License). The amendment (1) deletes all water quality requirements, (2) upgrades and changes the section on administrative controls, and (3) divides the Appendix B Technical Specifications into two parts: Part I- Radiological Environmental Technical Specifications (RETS), and Part II- Environmental Protection Plan (Non-radiological) Technical Specifications (NRETS).

Your basis for the requested deletion of water quality limits and monitoring programs is that these aquatic requirements are now under the jurisdiction of the U.S. Environmental Protection Agency as established by the Atomic Safety and Licensing Appeal Board's ruling on December 27, 1978 (Yellow Creek, ALAB-515). The NRC has taken the position that water quality conditions in existing reactor operating licenses should be removed as a matter of law where the licensee holds, as you do, an effective National Pollutant Discharge Elimination System (NPDES) permit. In place of such requirements, an Environmental Protection Plan (EPP) has been adopted in a standard format for all new plants, and existing plants on a case-by-case basis. The EPP is designed to keep the NRC aware of environmental effects of plant operation, while recognizing that the regulation of non-radiological aspects of aquatic matters lies with the appropriate NPDES permitting agency.

We concur in the deletion of the aquatic requirements and will rely on the NPDES permit system which is administered by the U.S. Environmental Protection Agency for regulation and protection of the aquatic environment.

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Mr. J. A. Hancock

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We have determined that the deletion of these water quality requirements is a ministerial action required as a matter of law and that therefore no environmental impact statement or environmental impact appraisal and negative declaration need be prepared in connection with this action.

We have also reviewed your proposed changes to administrative controls and the Part I, RETS, section of Appendix B. We have determined that the requirements have not been changed or reduced in effectiveness by the administrative control changes or by restructuring the RETS section into a section separate from the NRETS. We have determined that the changes do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the changes involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR 51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the approval of the changes.

This amendment does not involve significant new safety information of a type not considered by a previous Commission review of the facility. It does not involve a significant increase in the probability or consequences of an accident previously evaluated, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant reduction in a margin of safety, and therefore does not involve a significant hazards consideration. We have also concluded that there is reasonable assurance that the health and safety of the public will not be endangered by this action.

A copy of the Notice of Issuance is also enclosed.

Sincerely,

"ORIGINAL SIGNED BY
JOHN F. STOLZ"

John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Enclosures:

- 1. Amendment No. 58 to DPR-72
- 2. Notice of Issuance

cc w/enclosures:
See next page

*no neg declaration
to F.R. notice and amendment*

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Crystal River Unit No. 3
Florida Power Corporation

50-302

cc w/enclosure(s):
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER CORPORATION
CITY OF ALACHUA
CITY OF BUSHNELL
CITY OF GAINESVILLE
CITY OF KISSIMMEE
CITY OF LEESBURG
CITY OF NEW SMYRNA BEACH AND UTILITIES COMMISSION, CITY OF NEW SMYRNA BEACH
CITY OF OCALA
ORLANDO UTILITIES COMMISSION AND CITY OF ORLANDO
SEBRING UTILITIES COMMISSION
SEMINOLE ELECTRIC COOPERATIVE, INC.
CITY OF TALLAHASSEE

DOCKET NO. 50-302

CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 58
License No. DPR-72

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power Corporation, et al (the licensees) dated February 19, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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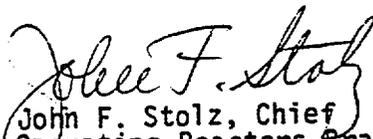
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-72 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 58, are hereby incorporated in the license. Florida Power Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 12, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 58

FACILITY OPERATING LICENSE NO. DPR-72

DOCKET NO. 50-302

Replace the following pages of the Appendix "B" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove

Appendix B-
Title Page

i
ii
iii
iv
1-1
1-2
1-3
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3-1
3-2
3-3
3-4
3-5
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Appendix B-Part I
Title Page

i
ii
iii*
iv
1-1
1-2
1-3
1-4
2-3
2-4
3-1
3-2
3-3
3-4
3-5
3-6*
3-8
4-1
4-2
5-1
5-2
5-3
5-4
5-4a
5-4b
5-5
5-5a
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Appendix B-Part II
Title Page

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Insert

1-1
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3-1
3-2
3-3
4-1
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*No change to these pages, Included for document completeness only.

APPENDIX B PART I

TO

FACILITY OPERATING LICENSE NO. DPR-72

CYRSTAL RIVER UNIT 3

FLORIDA POWER CORPORATION

DOCKET NO. 50-302

RADIOLOGICAL ENVIRONMENTAL
TECHNICAL SPECIFICATIONS

Amendment No. 58

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TABLE OF CONTENTS

	<u>Page No.</u>
1.0 DEFINITIONS	
1.1 Frequency	1-1
1.2 Gross (β, γ) Analysis	1-1
1.3 Deleted	1-1
1.4 Deleted	1-1
1.5 Unit 3 Mixing zone	1-1
1.6 Deleted	1-1
1.7 Deleted	1-1
1.8 Known Radioactive Source	1-3
1.9 Deleted	1-3
1.10 Deleted	1-3
1.11 Deleted	1-3
1.12 Deleted	1-3
1.13 Channel Calibration	1-3
1.14 Channel Check	1-3
1.15 Channel Functional Test	1-3
1.16 Dose Equivalent I-131	1-3
2.0 LIMITING CONDITIONS FOR OPERATION	
2.1 Deleted	2-1
2.2 Deleted	2-3
2.3 Deleted	2-3
2.3.1 Deleted	2-3
2.3.2 Deleted	2-4
2.4 Radioactive Effluents	2-4
2.4.1 Liquid Waste Effluents	2-5
2.4.2 Gaseous Waste Effluents	2-9
2.4.3 Solid Waste Handling and Disposal	2-16
3.0 ENVIRONMENTAL SURVEILLANCE	
3.1 Deleted	3-1

TABLE OF CONTENTS (Cont'd)

	<u>Page No.</u>
3.1.1 Deleted	3-2
3.1.2 Deleted	3-3
3.1.3 Deleted	3-4
3.1.4 Deleted	3-4
3.1.5 Deleted	3-5
3.2 Radiological Environmental Monitoring	3-6
3.2.1 Milk and Green Leafy Vegetables Census	3-26
3.2.2 Media Other Than External Radiation	3-27
3.2.3 External Radiation	3-27
3.2.4 Milk and Green Leafy Vegetables	3-27
4.0 Deleted	
4.1 Deleted	4-1
4.2 Deleted	4-2
4.3 Deleted	4-2
5.0 ADMINISTRATIVE CONTROLS	
5.1 Organization	5-1
5.2 Responsibility	5-1
5.3 Review and Audit	5-3
5.4 Action to Be Taken if Limiting Condition for Operation is Exceeded	5-5
5.5 Procedures	5-5
5.6 Plant Reporting Requirements	5-6
5.6.1 Routine Reports	5-6
5.6.2 Non-routine Reports	5-8
5.6.3 Changes	5-11
5.7 Records Retention	5-11
5.8 Deleted	5-11

LIST OF TABLES

<u>No.</u>		<u>Page No.</u>
2.4-1	Radioactive Liquid Sampling and Analysis	2-17
2.4-2	Radioactive Gaseous Waste Sampling and Analysis Frequency	2-19
2.4-3	PWR-Liquid Waste System: Location of Process and Effluent Monitors and Samples Required by Technical Specifications	2-21
2.4-4	PWR-Gaseous Waste System: Location of Process and Effluent Monitors and Samples Required by Technical Specifications	2-22
2.4-5	Gamma and Beta Dose Factors for Crystal River Unit 3	2-23
3.2-1	Summary of Preoperational Environmental Surveillance Results 1971-1974	3-9
3.2-2	General Pathway Radiological Environmental Monitoring Program	3-15
3.2-3	Critical Pathway Radiological Environmental Monitoring Program	3-18
3.2-4	Operational Sample Station Locations	3-19
3.2-5A	Detection Capabilities for Environmental Sample Analysis	3-21
3.2-5B	Detection Capabilities for Environmental Sample Analysis	3-22
5.6-1	Environmental Radiological Monitoring Program Summary	5-9

List of Figures

<u>No.</u>		<u>Page No.</u>
1.1-1	Deleted	1-2
1.1-2	Deleted	1-4
3.1-1	Deleted	3-7
3.2-1	Environmental Media and Exposure Pathways	3-23
3.2-2	Offsite Operational Radiological Program Sampling Sites	3-24
3.2-3	Onsite Operational Radiological Program Sampling Sites	3-25
5.1-1	Organization for Implementing and Independent Review and Audit of Radiological Environmental Technical Specifications	5-2
5.3-1	Deleted	5-4
5.8-1	Deleted	5-12

1.0 Definitions

The following terms are defined for uniform interpretation of the Radiological Environmental Technical Specifications for Crystal River Unit 3.

1.1 Frequency - Terms used to specify frequency are defined as follows:

One per shift - At least once per 8 hours.

Daily - At least once per 24 hours.

Weekly - At least once per 7 days.

Monthly - At least once per 31 days.

Quarterly - At least once per 92 days.

Semiannually - At least once per 6 months.

A maximum allowable extension for each surveillance requirement shall not exceed 25% of the surveillance interval.

1.2 Gross (β, γ) Analysis - Radioactivity measurements of gross beta or gross beta in conjunction with gross gamma as defined in Regulatory Guide 1.21.

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1.5 Unit 3 Mixing Zone - The enclosed area of the discharge canal bounded by the eastern end of the canal and the cable chase from Units 1 and 2 by crossing the canal.

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Figure 1.1-1

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- 1.8 Known Radioactive Source - A calibration source which is traceable to the National Bureau of Standards radiation measurement system and is capable of reproducible geometry.
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- 1.13 Channel Calibration - The adjustment, as necessary, of the channel output such that it responds with necessary range and accuracy to known values of the parameter which the channel monitors. The channel calibration shall encompass the entire channel including the sensor and alarm and/or trip functions, and shall include the channel functional test. Channel calibration may be performed by any series of sequential, overlapping or total channel steps such that the entire channel is calibrated.
- 1.14 Channel Check - The qualitative assessment of channel behavior during operation by observation. This determination shall include, where possible, comparison of the channel indication and/or status with other indications and/or status derived from independent instrument channels measuring the same parameter.
- 1.15 Channel Functional Test - The injection of a simulated signal into the channel as close to the primary sensor as practicable to verify operability including alarm and/or trip functions.
- 1.16 Dose Equivalent I-131 - That concentration of I-131 ($\mu\text{Ci}/\text{gram}$) which alone would produce the same thyroid dose as the quantity and isotopic mixture of I-131, I-132, I-133, I-134 and I-135 actually present. The thyroid dose conversion factors used for this calculation shall be those listed in TID-14844.

Figure 1.1-2

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2.4 RADIOACTIVE EFFLUENTS

Objective: To define the limits and conditions for the controlled release of radioactive materials in liquid and gaseous effluents to the environs to ensure that these releases are as low as reasonably achievable. These releases should not result in radiation exposures in unrestricted areas greater than a few percent of natural background exposures. The concentration of effluent discharges of radioactivities shall be within the limits specified in 10 CFR Part 20.

To ensure that the releases of radioactive material above background to unrestricted areas be as low as reasonably achievable, the following design objectives apply:

For liquid wastes:

- A. The annual dose above background to the total body or any organ of an individual from all reactors at a site should not exceed 5 mrem in an unrestricted area.
- B. The annual total quantity of radioactive materials in liquid waste, excluding tritium and dissolved gases, discharged from each reactor should not exceed 5 Ci.

For gaseous waste:

- C. The annual total quantity of noble gases above background discharged from the site should result in an air dose due to gamma radiation of less than 10 mrad, and air dose due to beta radiation of less than 20 mrad, at any location near ground level which could be occupied by individuals at or beyond the boundary of the site.
- D. The annual total quantity of radioiodines and radioactive material in particulate forms above background from all reactors

3.0 ENVIRONMENTAL SURVEILLANCE

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Amendment No. ~~12, 37, 58~~

1.0 Objectives of the Environmental Protection Plan

The Environmental Protection Plan (EPP) is to provide for protection of environmental values during operation and additional construction of the Crystal River Unit 3. The principal objectives of the EPP are as follows:

1. Verify that Crystal River Unit 3 is operated in an environmentally acceptable manner, as established by the Final Environmental Statement (FES) and other NRC environmental impact assessments.
2. Coordinate NRC requirements and maintain consistency with other Federal, State and local requirements for environmental protection.
3. Keep NRC informed of the environmental effects of Crystal River Unit 3 operation and additional construction, and of actions taken to control those effects.

Environmental concerns identified in the FES which relate to water quality matters are regulated by way of licensee's NPDES permit.

2.0 Environmental Protection Issues

In the FES-OL, dated May 1973, NRC staff considered the environmental impacts associated with the operation of Crystal River Unit 3. Certain environmental issues were identified which required study or license conditions to resolve environmental concerns and to assure adequate protection of the environment. The Appendix B Environmental Technical Specifications issued with the license included discharge restrictions and monitoring programs to resolve the issues. Prior to issuance of this EPP, the requirements remaining in the ETS were:

1. The need to control the release of heat (temperature) and chlorine within those discharge concentrations evaluated.
2. The need for aquatic monitoring programs to confirm that thermal mixing occurs as predicted, and that effects on aquatic biota and water quality due to plant operation are no greater than predicted.
3. The need for special studies to document levels of intake entrainment and impingement.

Aquatic issues are addressed by the effluent limitations, monitoring requirements and the Section 316(b) demonstration requirement contained in the effective NPDES permit issued by EPA-Region IV.

3.0 Consistency Requirements

3.1 Crystal River Unit 3 Design and Operation

The licensee may make changes in station design or operation or perform tests or experiments affecting the environment provided such changes, tests or experiments do not involve an unreviewed environmental question. Changes in plant design or operation or performance of tests or experiments which do not affect the environment are not subject to this requirement.

Before engaging in unauthorized construction or operational activities which may affect the environment, the licensee shall perform an environmental evaluation of such activity.* When the evaluation indicates that such activity involves an unreviewed environmental question, the licensee shall provide a written evaluation of such activities and obtain prior approval from the NRC.

A proposed change, test or experiment shall be deemed to involve an unreviewed environmental question if it concerns (1) a matter which may result in a significant increase in any adverse environmental impact previously evaluated in the FES, supplements to the FES, environmental impact appraisals, or in any decisions of the Atomic Safety and Licensing Board; or (2) a significant

*Activities are excluded from this requirement if all measurable nonradiological effects are confined to the on-site areas previously disturbed during site preparation and construction.

change in effluents or power level (in accordance with 10 CFR Part 51.5(b)(2)); or (3) a matter not previously reviewed and evaluated in the documents specified in (1) of this Subsection, which may have a significant adverse environmental impact.

The licensee shall maintain records of changes in facility design or operation and of tests and experiments carried out pursuant to this Subsection. These records shall include a written evaluation which provides bases for the determination that the change, test, or experiment does not involve an unreviewed environmental question.

Records of modifications to plant structures, systems and components determined to potentially affect the continued protection of the environment shall be retained for the life of the plant. All other records, data and logs relating to this EPP shall be retained for five years or, where applicable, in accordance with the requirements of other agencies.

Activities governed by Section 3.3 of this EPP are not subject to the requirements of this section.

3.2 Reporting Related to the NPDES Permit and State Certification

1. Violations of the NPDES Permit or the State 401 Certification Conditions shall be reported to the NRC by submittal of copies of the reports required by the NPDES Permit or State 401 Certification.

2. The licensee shall provide the NRC with a copy of any 316(a) or (b) studies and/or related documentation at the same time it is submitted to the permitting agency.
3. Changes and additions to the NPDES Permit or the State 401 Certification shall be reported to the NRC within 30 days following the date the change is approved. If a permit or certification, in part or in its entirety, is appealed and stayed, the NRC shall be notified within 30 days following the date the stay is granted.
4. The NRC shall be notified of changes to the effective NPDES Permit proposed by the licensee by providing NRC with a copy of the proposed change at the same time it is submitted to the permitting agency. The licensee shall provide the NRC a copy of the application for renewal of the NPDES Permit at the same time the application is submitted to the permitting agency.

3.3 Changes Required for Compliance with Other Environmental Regulations

Changes in Crystal River 3 design or operation and performance of tests or experiments which are required to achieve compliance with other Federal, State, or local environmental regulations are not subject to the requirements of Section 3.1.

4.0 Environmental Conditions

4.1 Significant Environmental Events

Any occurrence of a significant event that indicates or could result in significant environmental impact causally related to station operation shall be recorded and promptly reported to the NRC within 24 hours* followed by a written report within 30 days. No routine monitoring programs are required to implement this condition.

The written report shall (a) describe, analyze, and evaluate the event, including extent and magnitude of the impact and plant operating characteristics, (b) describe the probable cause of the event, (c) indicate the action taken to correct the reported event, (d) indicate the corrective action taken to preclude repetition of the event and to prevent similar occurrences involving similar components or systems, and (e) indicate the agencies notified and their preliminary responses.

Events reportable under this subsection which also require reports to other Federal, State or local agencies shall be reported in accordance with those reporting requirements in lieu of the requirements of this subsection. The NRC shall be provided a copy of such report at the same time it is submitted to the other agency.

*If a significant environmental event occurs over weekends or holidays the report shall be supplied within 24 hours of the first working day following the weekend or holiday.

The following are examples of significant environmental events: excessive bird impaction events; onsite plant or animal disease outbreaks; mortality or unusual occurrence of any species protected by the Endangered Species Act of 1973; unusual fish kills; and increase in nuisance organisms or conditions.

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-302FLORIDA POWER CORPORATION, ET ALNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 58 to Facility Operating License No. DPR-72, issued to the Florida Power Corporation, City of Alachua, City of Bushnell, City of Gainesville, City of Kissimmee, City of Leesburg, City of New Smyrna Beach and Utilities Commission, City of New Smyrna Beach, City of Ocala, Orlando Utilities Commission and City of Orlando, Sebring Utilities Commission, Seminole Electric Cooperative, Inc., and the City of Tallahassee (the licensees) which revised the Technical Specifications for operation of the Crystal River Unit No. 3 Nuclear Generating Plant (the facility) located in Citrus County, Florida. The amendment is effective as of the date of issuance.

The amendment pertains to the Environmental Technical Specifications (Appendix B to the Facility Operating License). The amendment (1) deletes all water quality requirements, (2) upgrades and changes the section on Administrative controls, and (3) divides the Appendix B Technical Specifications into two parts: Part I- Radiological Environmental Technical Specifications, and Part II- Environmental Protection Plan (Non-radiological) Technical Specifications.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act).

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and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

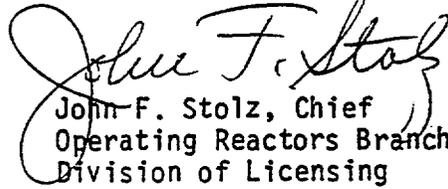
The Commission has determined that dividing the Appendix B Technical Specifications into two parts and the changes to administrative controls will not result in any significant environmental impact and pursuant to 10 CFR §1.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the changes. The Commission has also determined that the deletion of water quality requirements is a ministerial action required as a matter of law and that therefore no environmental impact statement or environmental impact appraisal and negative declaration need be prepared in connection with this action.

For further details with respect to this action, see (1) the application for amendment dated February 19, 1982, (2) Amendment No. 58 to License No. DPR-72, and (3) the Commission's letter to Florida Power Corporation dated October 12, 1982. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW, Washington, D.C., and at the Crystal River Public Library, 668 N.W. First Avenue, Crystal River, Florida. A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear

Regulatory Commission, Washington, D.C. 20555, Attention: Director,
Division of Licensing.

Dated at Bethesda, Maryland, this 12th day of October 1982.

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

3.1.5 Deleted

3.2 RADIOLOGICAL ENVIRONMENTAL MONITORING

Objective

The radiological environmental monitoring program will provide information which can be used to assist in assessing the type and quantity of radiation exposure in unrestricted areas resulting from plant operation.

Background

Preoperational radiological environmental monitoring programs, to establish baseline environmental concentration values, were initiated in mid-1970. One program was operated by the State of Florida Department of Health and Rehabilitative Services; another program was operated by the University of Florida.

A summary of the preoperational surveillance results is shown in Table 3.2-1. This summary includes median values of the observed environmental concentrations and 95 percentile values (i.e., values which exceed 95 percent of all the comparable measured values). These values will be taken as the preoperational baseline concentrations. In some cases the values listed are smaller than the Lower Limit of Detection (LLD).

The 95 percentile values indicate the random frequency of high measured values during the operation of the plant contributes negligibly to the environmental radioactivity. These 95 percentile values will be used during operation to assess the probability that any observed high concentration value is due to random fluctuations in measurements rather than to a true increase in environmental concentrations.

Specification (Program)

Environmental media which are sampled and analyzed for radioactivity are shown by the two diagrams on Figure 3.2-1. Each box in the diagrams contains the name of an environmental media which is sampled. The upper diagram shows the critical pathways; the lower diagram shows the other monitored pathways.

The operational radiological monitoring program shall consist of a continuation of the preoperational program of measurements of radioactivity in environmental media which is outlined in Table 3.2-2.

The critical pathway monitoring program which is included in Table 3.2-2 is also shown in Table 3.2-3. Sample station locations are described on Table 3.2-4 and shown on maps on Figures 3.2-2 and 3.2-3. Lower Limit of Detection (LLD) values are given on Tables 3.2-5A and 3.2-5B.

Deviations are permitted from the required sampling schedule if specimens are unobtainable due to hazardous conditions, vandalism, seasonal unavailability or to malfunction of automatic sampling equipment. If the latter, every effort shall be made to complete corrective action prior to the end of the next sampling period. All deviations from the sampling schedule shall be described in the annual report.

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5.0 ADMINISTRATIVE CONTROLS

Objective

To define the organization, assign responsibilities, describe the environmental surveillance procedures, provide for a review and audit function, and prescribe the reporting requirements in order to insure continuing protection of the environment and implement the Environmental Technical Specifications.

5.1 ORGANIZATION

The organization responsible for radiological environmental protection, radiological environmental monitoring and the implementation of the Radiological Environmental Technical Specifications for Crystal River Unit 3, is shown on Figure 5.1-1.

5.2 RESPONSIBILITY

The responsibility for the conduct of the operational radiological environmental monitoring program described in Section 3 is that of the Nuclear Support Services Department under the direction of the Vice President, Nuclear Operations.

The plant organization is responsible for the development of Operating and Surveillance Procedures described generally in Section 5.5 and supplying field data to the Manager, Nuclear Support Services as required by Sections 2 and 3 of the Radiological Environmental Technical Specifications.

The Manager, Nuclear Support Services is responsible for consultant contracts, State and local regulatory agreements, assembly of data, preparation and review of reports required by Section 5.6 of these Radiological Environmental Technical Specifications, and making recommendations to improve radiological environmental protection practices.

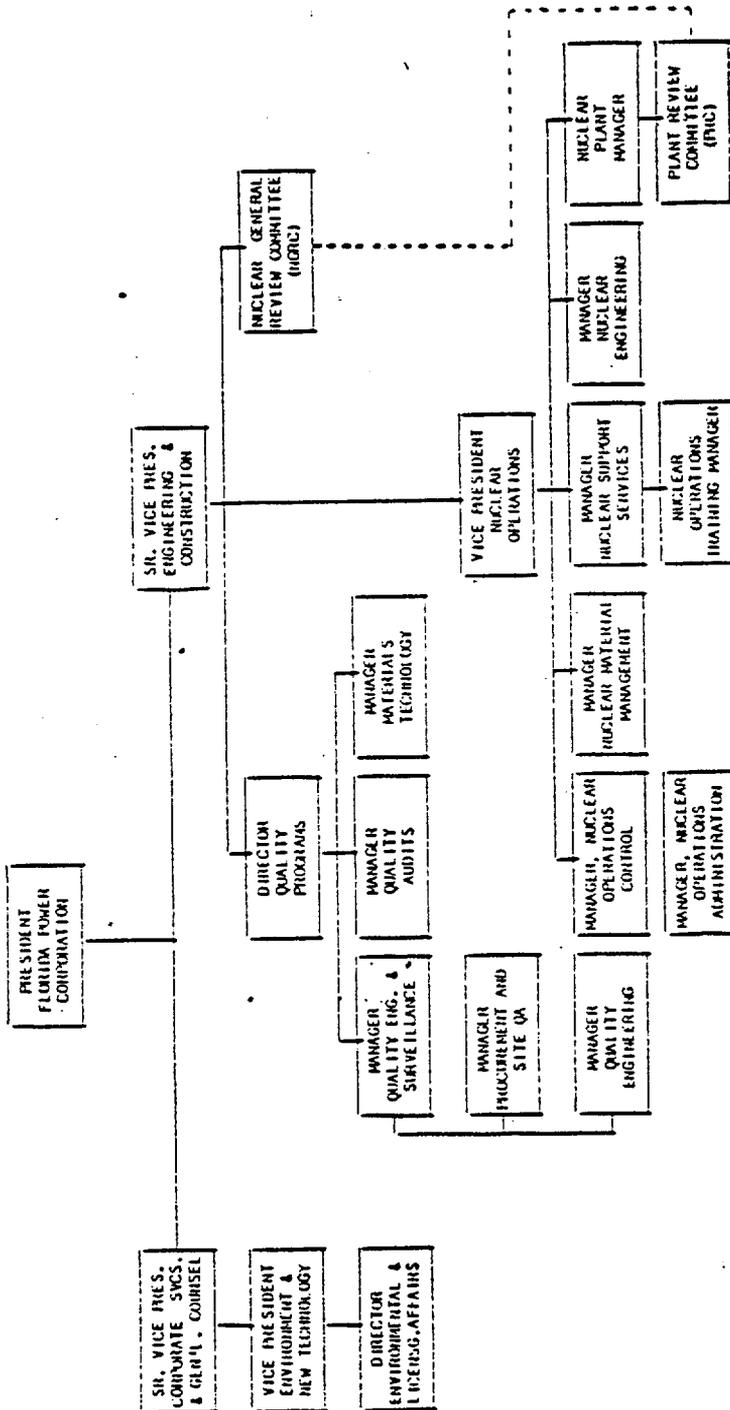


FIGURE 5.1-1 ORGANIZATION FOR IMPLEMENTING AND INDEPENDENT REVIEW AND AUDIT OF RADIOLOGICAL ENVIRONMENTAL TECHNICAL SPECIFICATIONS

5.3 REVIEW AND AUDIT

5.3.1 Function

The Nuclear General Review Committee shall function to provide independent review and audit of designated activities in the areas of radiological environmental monitoring and surveillance.

5.3.2 Composition

The NGRC shall be composed as described in the Crystal River Unit 3 Technical Specifications, Appendix A, Section 6.5.2.2.

5.3.3 Qualifications

The Nuclear General Review Committee members' qualifications shall be as described in the Crystal River Unit 3 Technical Specifications, Appendix A, Section 6.5.2.3.

FIGURE 5.3-1 DELETED

5.3.4 Alternates

All alternate members shall be appointed in accordance with the Crystal River Unit 3 Technical Specifications, Appendix A, Section 6.5.2.4.

5.3.5 Consultants

Consultants shall be utilized as provided in the Crystal River Unit 3 Technical Specifications, Appendix A, Section 6.5.2.5.

5.3.6 Meeting Frequency

The NGRC shall meet as required by the Crystal River Unit 3 Technical Specifications, Appendix A, Section 6.5.2.6.

5.3.7 Quorum

A quorum of NGRC shall be as designated in the Crystal River Unit 3 Technical Specifications, Appendix A, Section 6.5.2.7.

5.3.8 Review

The NGRC shall review:

- a. Proposed changes to the Radiological Environmental Technical Specifications and the evaluated impact of the changes.
- b. Proposed changes or modifications to plant systems or equipment and the evaluated impact which would require a change in the procedures described in 5.5.1 below (or which would affect the evaluation of the plant's radiological environmental impact) as determined by the Plant Review Committee.
- c. Reported instances of violations of Radiological Environmental Technical Specifications, the reaching of specified reporting levels, and reportable radiological environmental occurrences.
- d. Proposed special tests or experiments which might involve a change in the Radiological Environmental Technical Specifications or involve an unreviewed increase in radiological effluents.
- e. Events requiring 24 hour notification to the Commission.
- f. Descriptions of changes, tests or experiments, and the results thereof, as described in 5.5.3.a.

- h. Audits of the radiological environmental monitoring and surveillance program.

5.3.9 Audits

Audits of facility activities shall be performed under the cognizance of the NGRC. These audits shall encompass:

- a. Implementation of the radiological environmental monitoring and surveillance programs at least once per twelve months.
- b. Conformance to procedures and RETS requirements at least once per twelve months.
- c. Contractor radiological environmental monitoring and surveillance activities at least once per twelve months.

5.3.10 Records

Records of NGRC activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each NGRC meeting shall be prepared, approved and forwarded to the Senior Vice President Engineering and Construction within 14 days following each meeting.
- b. Reports of reviews encompassed by Section 5.3.8 above, shall be prepared, approved and forwarded to the Senior Vice President Engineering and Construction within 14 days following the completion of the review.
- c. Audit reports encompassed by Section 5.3.9 above, shall be forwarded to the Senior Vice President Engineering and Construction and to the management positions responsible for the areas audited within 30 days after completion of the audit.

5.3.11 Authority

The NGRC shall report to and advise the Senior Vice President Engineering and Construction on those areas of responsibility specified in Sections 5.3.8, 5.3.9 and 5.3.10 (see Figure 5.1-1).

5.4 ACTION TO BE TAKEN IF LIMITING CONDITION FOR OPERATION IS EXCEEDED.

- 5.4.1 Immediate remedial actions as permitted by these Radiological Environmental Technical Specifications shall be implemented until such time as the limiting condition for operation is met.
- 5.4.2 The occurrence shall be promptly reported to the Chairman of the Nuclear General Review Committee and investigated as specified in Section 5.3.
- 5.4.3 The Nuclear General Review Committee shall prepare and submit a report for each occurrence in accordance with Section 5.3.10.
- 5.4.4 The Vice President, Nuclear Operations, or his designee, shall report the occurrence to the NRC as specified in Section 5.6.2.

5.5 PROCEDURES

- 5.5.1 Explicit written procedures, including applicable check-off lists and instructions, shall be prepared for the implementation of the monitoring requirements described in Sections 2 and 3, approved as specified in Section 5.5.2, and adhered to for operation of all systems and components involved in carrying out the effluent release and environmental radiological monitoring programs. Procedures shall include sampling, instrument calibration, analysis, and action to be taken when limits are approached or exceeded. Calibration frequencies and standards for instruments used in performing the measurements shall be included. Testing frequency of alarms shall be included. These frequencies shall be determined from experience with similar instruments in similar environments and from manufacturers' technical manuals.
- 5.5.2 Each procedure in 5.5.1 above, and changes thereto, shall be reviewed and approved prior to implementation as follows:

a. Except as noted in b:

1. All procedures and changes thereto shall be subjected to a review by a Qualified Reviewer within the responsible department, interdisciplinary review by Qualified Reviewers within interfacing departments as specified in Administrative Procedures, and the responsible superintendent or Plant Manager as specified in Administrative Procedures.

2. The training and qualification of reviewers shall be governed by Administrative Procedures with final certification by the Nuclear Plant Manager. Recertification will be required on a periodic basis and upon transfer between departments. As a minimum, all Qualified Reviewers shall meet the requirements of ANSI N18.1-1971, Section 4.2, 4.4, or 4.6, or the equivalent.
 3. All procedure revision records, including the reviews required by 10 CFR 50.59, shall be reviewed by the Plant Review Committee within 14 days, and forwarded to the Nuclear General Review Committee.
- b. Temporary changes may be made to procedures required by Section 5.5.1 provided:
1. The intent of the original procedure is not altered.
 2. The change is approved by two members of the plant management staff, at least one of which holds a Senior Reactor Operators (SRO) License.
 3. The change is documented and reviewed per 5.5.2.a.3, above.

5.5.3 Special Tests or Changes:

The licensee may make changes in station design or operation or perform tests or experiments provided such changes, tests or experiments do not involve an unevaluated radiological impact question or involve a change in these Radiological Environmental Technical Specifications.

Any change in station design or operation or test or experiment which could involve an unevaluated radiological impact question or involve a change in these Radiological Environmental Technical Specifications shall be submitted to the Nuclear General Review Committee for review and report to the Vice President, Nuclear Operations for resolution prior to implementation.

5.6 PLANT REPORTING REQUIREMENTS

5.6.1 Routine Reports

Radiological Volume

A report on the radiological environmental surveillance programs for the previous 12 months of operation shall be submitted to the Director of Inspection and Enforcement (with copy to Director, Office of Nuclear Reactor Regulation) by April 1, each year. The report shall include summaries, interpretations, and statistical evaluation of the results of the radiological environmental surveillance activities for the report period, including a comparison with preoperational studies, operational controls (as appropriate) and previous environmental surveillance reports, and an assessment of the observed impacts of the plant operation on the environment. If harmful effects or evidence of irreversible damage are detected by the monitoring, the licensee shall provide an analysis of the problem and a proposed course of action to alleviate the problem.

Results of all radiological environmental samples taken shall be summarized on an annual basis following the format of Table 5.6-1. In the event that some results are not available by April 1, the report shall be submitted, noting and explaining the reasons for the missing results. The missing data shall be submitted as soon as possible in a supplementary report.

B. Semiannual Operating Report - Radioactive Effluents

A report on the radioactive discharges released from the site during the previous 6 months of operation shall be submitted to the Director of the Office of Inspection and Enforcement (with a copy to Director, Office of Nuclear Reactor Regulation) as part of the Semiannual Operating Report by March 1 and September 1 each year. The report shall include a summary of the quantities of radioactive liquid and gaseous effluents and solid waste released from the plant as outlined in USNRC Regulatory Guide 1.21, with data summarized on a quarterly basis following the format of Appendix 3 thereof. Any unplanned releases of radioactive material from the site will be reported in accordance with Appendix B Section A.6, "Abnormal Release" of Regulatory Guide 1.21, with the cause of each release identified.

The report shall include a summary of the meteorological conditions concurrent with the release of gaseous effluents during each quarter as outlined in USNRC Regulatory Guide 1.21, with data summarized on a quarterly basis following the format of Appendix B thereof. Calculated offsite dose to humans resulting from the release of effluents and their subsequent dispersion in the atmosphere (Regulatory Guide 1.109) shall be reported in accordance with Regulatory Guide 1.21.

5.6.2 Non-Routine Reports

A. Limiting Condition for Operation Exceeded

In the event that: (1) a limiting condition for operation is exceeded, (2) an unplanned release of radioactive material from the site occurs in quantities such that the release rate is greater than 10% of the allowed instantaneous release rate specified in 2.4.1A or 2.4.2A or (3) an event involving a significant adverse radiological environmental impact occurs, a report will be made within 24 hours by telephone and telegraph to the Director of the Office of Inspection and Enforcement followed by a written report with a copy to the Director, Office of Nuclear Reactor Regulation within 15 days. The telegraph report will quantify the occurrence, its causes and, if aspects of the Crystal River Unit 3 operation are among the

causes, planned remedial action to the extent possible. The written report will fully describe the occurrence and will describe its causes and corrective action as fully as possible.

TABLE 5.6-1

ENVIRONMENTAL RADIOLOGICAL MONITORING PROGRAM SUMMARY

Name of Facility _____ Docket No. _____

Location of Facility _____ Reporting Period _____
(County, State)

Medium or Pathway Sampled (Unit of Measurement)	Type and Total Number of Analyses Performed	Lower Limit of Detection ^a (LLD)	All Indicator Locations Mean (f) ^b Range ^b	Location with Highest Annual Mean		Control Locations Mean (f) ^b Range ^b	Number of Nonroutine Reported Measurements ^c
				Name Distance and Direction	Mean (f) ^b Range ^b		
Air Particulates (pCi/m ³)	Gross β 416	0.003	0.08 (200/312) (0.05-2.0)	Middletown 5 miles 340°	0.10 (5/52) (0.08-2.0)	0.08 (8/104) (0.05-1.40)	1
	γ-Spec. 32 ¹³⁷ Cs	0.003	0.05 (4/24) (0.03-0.13)	Smithville 2.5 miles 160°	0.08 (2/4) (0.03-0.13)	<LLD	4
	¹⁴⁰ Ba	0.003	0.03 (2/24) (0.01-0.08)	Podunk 4.0 miles 270°	0.05 (2/4) (0.01-0.08)	0.02 (1/8)	1
	⁸⁹ Sr 40	0.002	<LLD	-	-	<LLD	0
	⁹⁰ Sr 40	0.0003	<LLD	-	-	<LLD	0
	Fish pCi/kg (dry weight)	γ-Spec. 8 ¹³⁷ Cs	80	<LLD	-	<LLD	90 (1/4)
¹³⁴ Cs		80	<LLD	-	<LLD	<LLD	0
⁶⁰ Co		80	120 (3/4) (90-200)	River Mile 35 Podunk River	See column 4	<LLD	0

Example Data Presentation^d

5-9

^aNominal Lower Limit of Detection (LLD) as defined in HASL-300 (Rev. 8/73), pp. D-08-01, 02, 03.

^bMean and range based upon detectable measurements only. Fraction of detectable measurements at specified locations is indicated in parentheses. (f)

^cNonroutine reported measurements are defined in Section 5.6.2.h.

^dNote: The example data are provided for illustrative purposes only.

B. Deleted

C. Radiological Reporting Levels

In the event a report level specified below is reached, a report shall be made within the designed time period to the Director of Inspection and Enforcement with a copy to the Director of Office of Nuclear Reactor Regulation.

(1) Radioactive Discharge

If measured rates of release of radioactivity in the environment, averaged over a calendar quarter, exceed the design objective rates as specified in specifications 2.4.1.H for liquid effluents and in 2.4.2.C for airborne effluents, a report of the causes of the release rates and of a proposed program of action to reduce the release rates will be submitted within 30 days from the end of the quarter during which the release occurred.

(2) Radiological Environmental Monitoring

If a single measured value of radioactivity concentrations in critical pathway environmental medium samples identified in Section 3.2 exceeds ten times the control station value as defined in Section 3.2, a written notification including an evaluation of any release conditions, environmental factors, or other aspects necessary to explain the anomalous result shall be submitted to the Director of the NRC Regional Office (with a copy to the Director, Office of Nuclear Reactor Regulation) within 10 days after confirmation.*

*Confirmation is defined in Regulatory Guide 4.8.

5.6.3 Changes

- A. A report shall be made to the Director of Office of Nuclear Reactor Regulation prior to implementation of a change in plant design, in plant operation, or in procedures described in Section 5.5 if the change would have, in the judgment of the licensee, a significant adverse effect on the environment or involves a radiological environmental matter or question not previously reviewed and evaluated by the USNRC. The report shall include a description and evaluation of the change and a supporting benefit-cost analysis.
- B. Request for changes in Radiological Environmental Technical Specifications shall be submitted to the Director of Office of Nuclear Reactor Regulation for review and authorization. The request shall include an evaluation of the impact on the change and a supporting benefit-cost analysis.

5.7 RECORDS RETENTION

- 5.7.1 Records and logs relative to the following areas shall be retained for the life of the plant:
 - a. Records and drawing changes reflecting plant design modifications made to systems and equipment as described in Section 5.6.3.
 - b. Records of radiological environmental surveillance data.
 - c. Records to demonstrate compliance with the limiting conditions for operation in Section 2.0.
- 5.7.2 All other records and logs relating to the radiological environmental technical specifications shall be retained for five years.
- 5.8 Deleted

5-12

Figure 5.8-1 Deleted

APPENDIX B - PART II

TO FACILITY OPERATING LICENSE NO. DPR-72
CRYSTAL RIVER UNIT 3

FLORIDA POWER CORPORATION
DOCKET NO. 50-302

ENVIRONMENTAL PROTECTION PLAN
(NON-RADIOLOGICAL)
TECHNICAL SPECIFICATIONS