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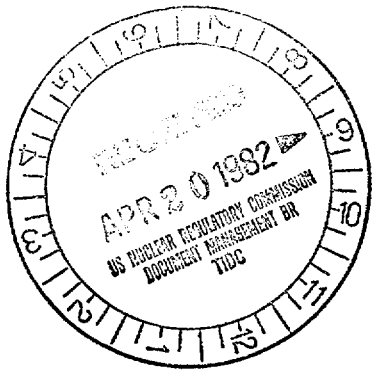

Mr. John A. Hancock
Vice President, Nuclear Operations
Florida Power Corporation
ATTN: Manager, Nuclear Licensing
P. O. Box 14042, M.A.C. H-2
St. Petersburg, Florida 33733

Dear Mr. Hancock:

This confirms our telephone authorization of April 6, 1982, with regard to the operation of the Crystal River Unit No. 3 Nuclear Generating Plant (CR-3) as requested by your telecopied letter dated April 6, 1982. On April 6, 1982, we authorized operation of CR-3 with the Reactor Coolant Pump Power Monitor (RCPPM) trip function bypassed provided four reactor coolant pumps were operating and the plant was operated at no more than 90% full power. You stated that you wish to operate with the RCPPM trip function bypassed until you can resolve the problem of the spurious reactor trip based on the RCPPMs.

Facility Operating License No. DPR-72 was amended April 6, 1982, by making the following Technical Specification change to Table 3.3.2:

Revise note "(c)" to "Table Notation" to read:

> "In addition to the limits noted in (b) above, interim operation of the reactor is limited to a maximum of 90% full power while the Reactor Coolant Pump Power Monitor (RCPPM) trip is manually bypassed to resolve the problem of spurious scrams from RCPPM operation."

Copies of the license amendment, our evaluation and Federal Register Notice for the TS change will be sent to you when completed.

Sincerely,

Original signed by:
Thomas M. Novak

Thomas M. Novak, Assistant Director
for Operating Reactors
Division of Licensing

cp
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cc: See next page

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NRC FORM 318 (10-80) NRCM 0240

OFFICIAL RECOR

Safety Evaluation Report

Crystal River 3 Technical Specification

Change Request No. 97

Introduction

Florida Power Corp. has requested a Technical Specification change to allow operation up to 90% of full power (2300 Mwt / 2544 Mwt = .90) with the Reactor Coolant Pump Power Monitors out-of-service. The RCPPM provides an instantaneous reactor trip signal on loss of power to the reactor coolant pumps. This system was added to CR-3 as part of the recent power level upgrade (2452 Mwt to 2544 Mwt). Use of the system has produced some unnecessary reactor trip due to transmission grid disturbances. Florida Power Corp. therefore intends to operate CR-3 at 90% power

without the RCPPM until the system can be modified to eliminate the unnecessary trips.

Discussion

Florida Power Corp. has submitted a summary of a B+W analysis of the four pump loss of coolant flow without taking credit for the RCPPM. This analysis relied on the Flux/Flow Trip. And shows that the minimum DNBR is 1.43 for a four pump loss of flow initiated from 2300 Mwt.

Evaluation

The analysis of the four pump loss of flow was done with the same codes, methods and correlations as previously used in CR-3 licensing analyses. The analysis results indicate that the minimum DNBR would be 1.43. The DNBR limit is 1.35. The analysis therefore

demonstrates acceptable results.

B+W was contacted (4/6/82 conversation with J. Costanes and others) to assure that the analyses had been performed in conformance with B+W quality assurance procedures. Technical aspects of the analysis were also discussed to assure that the results appeared reasonable and consistent with previous analyses.

Conclusion:

Based on the results of the analyses submitted by Florida Power Corp., we conclude that a four pump loss of coolant flow event initiated at 2300 MWt would not exceed the applicable

regulatory limits, therefore approval of the proposed Technical Specification change will not result in an undue risk to the health and safety of the public.

Gary M Holahan
Section Leader, System Section
ORAB, DL.

ok
jm

cc w/enclosure(s):
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