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Mr. J. A. Hancock  
Director, Nuclear Operations  
Florida Power Corporation  
P. O. Box 14042, Mail Stop C-4  
St. Petersburg, Florida 33733



Dear Mr. Hancock:

The Commission has issued the enclosed Amendment No. 43 to Facility Operating License No. DPR-72 for the Crystal River Unit No. 3 Nuclear Generating Plant (CR-3). The amendment revises Technical Specification 4.1.3.1.2 in response to your application dated September 24, 1981. The amendment authorizes a delay of three days in the monthly operational tests of the control rods.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by

Peter B. Erickson, Project Manager  
Operating Reactors Branch #4  
Division of Licensing

Enclosures:

- 1. Amendment No. 43
- 2. Safety Evaluation
- 3. Notice

cp  
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cc w/enclosures:  
See next page

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FR NOTICE  
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AMENDMENT  
ONLY

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

FLORIDA POWER CORPORATION  
CITY OF ALACHUA  
CITY OF BUSHNELL  
CITY OF GAINESVILLE  
CITY OF KISSIMMEE  
CITY OF LEESBURG  
CITY OF NEW SMYRNA BEACH AND UTILITIES COMMISSION, CITY OF NEW SMYRNA BEACH  
CITY OF OCALA  
ORLANDO UTILITIES COMMISSION AND CITY OF ORLANDO  
SEBRING UTILITIES COMMISSION  
SEMINOLE ELECTRIC COOPERATIVE, INC.  
CITY OF TALLAHASSEE

DOCKET NO. 50-302

CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 43  
License No. DPR-72

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Florida Power Corporation, et al (the licensees) dated September 24, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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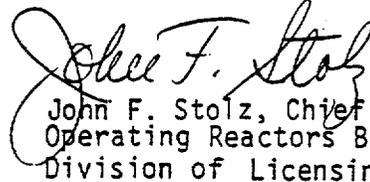
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-72 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 43, are hereby incorporated in the license. Florida Power Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Chief  
Operating Reactors Branch #4  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: September 25, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 43

FACILITY OPERATING LICENSE NO. DPR-72

DOCKET NO. 50-302

Replace the following page of Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains vertical lines indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

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## REACTIVITY CONTROL SYSTEMS

### ACTION: (Continued)

- c) A power distribution map is obtained from the incore detectors and  $F_0$  and  $F_{\Delta H}$  are verified to be within their limits within 72 hours, and
- d) The THERMAL POWER level is reduced to  $\leq 60\%$  of the THERMAL POWER allowable for the reactor coolant pump combination within one hour and within the next 4 hours the Nuclear Overpower Trip Setpoint is reduced to  $< 70\%$  of the THERMAL POWER allowable for the reactor coolant pump combination, or
- e) The remainder of the rods in the group with the inoperable rod are aligned to within  $\pm 6.5\%$  of the inoperable rod within one hour while maintaining the rod sequence, insertion and overlap limits of Figures 3.1-1, 3.1-2, 3.1-3, 3.1-4, 3.1-5 and 3.1-6; the THERMAL POWER level shall be restricted pursuant to Specification 3.1.3.6 during subsequent operation.

## SURVEILLANCE REQUIREMENTS

4.1.3.1.1 The position of each control rod shall be determined to be within the group average height limit by verifying the individual rod positions at least once per 12 hours except during time intervals when the Asymmetric Rod Monitor is inoperable, then verify the individual rod positions at least once per 4 hours.

4.1.3.1.2 Each control rod not fully inserted shall be determined to be OPERABLE by movement of at least 3% in any one direction at least once every 31 days. \*

\* This surveillance period is extended for 3 days for the September 1981 test only.

## REACTIVITY CONTROL SYSTEMS

### GROUP HEIGHT - AXIAL POWER SHAPING ROD GROUP

#### LIMITING CONDITION FOR OPERATION

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3.1.3.2 All axial power shaping rods (APSR) shall be OPERABLE, unless fully withdrawn, and shall be positioned within  $\pm 6.5\%$  (indicated position) of their group average height.

APPLICABILITY: MODES 1\* and 2\*.

#### ACTION:

With a maximum of one APSR inoperable or misaligned from its group average height by more than  $\pm 6.5\%$  (indicated position), operation may continue provided that within 2 hours:

- a. The APSR group is positioned such that the misaligned rod is restored to within limits for the group average height, or
- b. It is determined that the imbalance limits of Specification 3.2.1 are satisfied and movement of the APSR group is prevented while the rod remains inoperable or misaligned.

#### SURVEILLANCE REQUIREMENTS

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4.1.3.2.1 The position of each APSR rod shall be determined to be within the group average height limit by verifying the individual rod positions at least once per 12 hours except during time intervals when the Asymmetric Control Rod Monitor is inoperable, then verify the individual rod positions at least once per 4 hours.

4.1.3.2.2 Unless all APSR are fully withdrawn, each APSR shall be determined to be OPERABLE by moving the individual rod at least 3% at least once every 31 days.

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\* See Special Test Exceptions 3.10.1 and 3.10.2.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
SUPPORTING AMENDMENT NO. 43 TO FACILITY OPERATING LICENSE NO. DPR-72

FLORIDA POWER CORPORATION, ET AL.

CRYSTAL RIVER UNIT NO. 3 NUCLEAR GENERATING PLANT

DOCKET NO. 50-302

Introduction

By letter dated September 24, 1981, Florida Power Corporation (FPC) proposed changes to the Crystal River Unit No. 3 (CR-3) Technical Specifications. The proposed change would extend the monthly control rod surveillance period for three days for the September 1981 test.

Discussion and Safety Evaluation

Technical Specification 4.1.3.1.2 requires control rods to be exercised once per month (31 days + 25%) to verify that they are movable and, therefore, free from mechanical interference. The next test of control rods would be required on or before noon on September 25, 1981, which is about 2 1/2 days before Cycle 4 refueling shutdown starts.

FPL has requested a delay of 3 days for this surveillance requirement because the test requires putting CR-3 into a manual control mode that may cause an increase in the leak rate of the seals of a reactor coolant pump which has somewhat degraded seals. Manual control removes the Integrated Control System from the reactor control system which in turn results in pressure oscillations of up to + 20 psi of the primary system due to the characteristics of the plant in that mode of operation. The primary system pressure oscillations would put more stress on the reactor coolant pump seals.

A delay of three days in the surveillance test is not considered to be a significant safety hazard because there has been no evidence of malfunction or sticking of the control rods during this cycle of operation or the previous rod tests, including the August 1981 test. In addition, the reactor has considerable excess shutdown margin so that failure of several rods to insert would not result in failure of the reactor to shutdown as required.

### Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

### Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: September 25, 1981

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-302FLORIDA POWER CORPORATION, ET ALNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY  
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 43 to Facility Operating License No. DPR-72, issued to the Florida Power Corporation, City of Alachua, City of Bushnell, City of Gainesville, City of Kissimmee, City of Leesburg, City of New Smyrna Beach and Utilities Commission, City of New Smyrna Beach, City of Ocala, Orlando Utilities Commission and City of Orlando, Sebring Utilities Commission, Seminole Electric Cooperative, Inc., and the City of Tallahassee (the licensees) which revised the Technical Specifications (TSs) for operation for the Crystal River Unit No. 3 Nuclear Generating Plant (the facility) located in Citrus County, Florida.

The amendment revises the Technical Specifications to authorize a delay of three days in the monthly operational tests of the control rods.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

-2-

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §1.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated September 24, 1981, (2) Amendment No. 43 to License No. DPR-72, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Crystal River Public Library, 668 N.W. First Avenue, Crystal River, Florida: A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 25th day of September 1981.

FOR THE NUCLEAR REGULATORY COMMISSION

  
John F. Stolz, Chief  
Operating Reactors Branch #4  
Division of Licensing