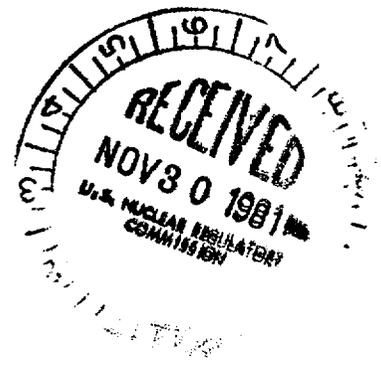


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Mr. J. A. Hancock  
 Assistant Vice President - Nuclear  
 Operations  
 Florida Power Corporation  
 P. O. Box 14042, M. A. C. H. 2.  
 St. Petersburg, Florida 33733

Dear Mr. Hancock:

The Commission has issued the enclosed Amendment No. 44 to Facility Operating License No. DPR-72 for the Crystal River Unit No. 3 Nuclear Generating Plant (CR-3). The amendment consists of changes to the Technical Specifications (TSs) in response to your application of February 15, 1980.

This change revises TS requirements for testing of Radiation Monitor RM-A5 to be consistent with Standard TSs.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by

Peter B. Erickson, Project Manager  
 Operating Reactors Branch #4  
 Division of Licensing

CP  
1

- Enclosures:  
 1. Amendment No. 44  
 2. Safety Evaluation  
 3. Notice

cc w/enclosures:  
 See next page

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 AMENDMENT ONLY

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DATE	10/9/81	10/9/81	10/16/81	10/17/81	10/16/81	10/19/81

Crystal River Unit No. 3  
Florida Power Corporation

50-302

cc w/enclosure(s):

Mr. S. A. Brandimore  
Vice President and General Counsel  
P. O. Box 14042  
St. Petersburg, Florida 33733

Mr. Wilbur Langely, Chairman  
Board of County Commissioners  
Citrus County  
Iverness, Florida 36250

Regional Radiation Representative  
EPA Region IV  
345 Courtland Street, N.E.  
Atlanta, Georgia 30308

Crystal River Public Library  
668 N. W. First Avenue  
Crystal River, Florida 32629

Mr. Robert B. Borsum  
Babcock & Wilcox  
Nuclear Power Generation Division  
Suite 420, 7735 Old Georgetown Road  
Bethesda, Maryland 20014

Mr. Tom Stetka, Resident Inspector  
U.S. Nuclear Regulatory Commission  
Route #3, Box 717  
Crystal River, Florida 32629

Mr. Dan C. Poole  
Nuclear Plant Manager  
Florida Power Corporation  
P. O. Box 219  
Crystal River, Florida 32629

cc w/enclosure(s) & incoming dtd.:

2/15/80  
Bureau of Intergovernmental Relations  
660 Apalachee Parkway  
Tallahassee, Florida 32304

Administrator  
Department of Environmental Regulation  
Power Plant Siting Section  
State of Florida  
2600 Blair Stone Road  
Tallahassee, Florida 32301

Attorney General  
Department of Legal Affairs  
The Capitol  
Tallahassee, Florida 32304



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

FLORIDA POWER CORPORATION  
CITY OF ALACHUA  
CITY OF BUSHNELL  
CITY OF GAINESVILLE  
CITY OF KISSIMMEE  
CITY OF LEESBURG  
CITY OF NEW SMYRNA BEACH AND UTILITIES COMMISSION, CITY OF NEW SMYRNA BEACH  
CITY OF OCALA  
ORLANDO UTILITIES COMMISSION AND CITY OF ORLANDO  
SEBRING UTILITIES COMMISSION  
SEMINOLE ELECTRIC COOPERATIVE, INC.  
CITY OF TALLAHASSEE

DOCKET NO. 50-302

CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 44  
License No. DPR-72

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Florida Power Corporation, et al (the licensees) dated February 15, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

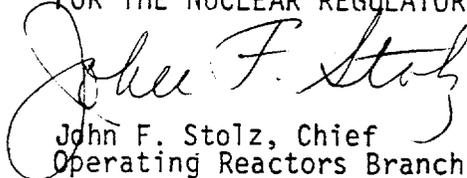
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-72 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 44, are hereby incorporated in the license. Florida Power Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Chief  
Operating Reactors Branch #4  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: November 24, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 44

FACILITY OPERATING LICENSE NO. DPR-72

DOCKET NO. 50-302

replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages

3/4 3-23

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3/4 7-20

3/4 7-21



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

November 24, 1981

Docket No. 50-302

Mr. J. A. Hancock  
Assistant Vice President - Nuclear  
Operations  
Florida Power Corporation  
P. O. Box 14042, M. A. C. H. 2.  
St. Petersburg, Florida 33733

Dear Mr. Hancock:

The Commission has issued the enclosed Amendment No. 44 to Facility Operating License No. DPR-72 for the Crystal River Unit No. 3 Nuclear Generating Plant (CR-3). The amendment consists of changes to the Technical Specifications (TSs) in response to your application of February 15, 1980.

This change revises TS requirements for testing of Radiation Monitor RM-A5 to be consistent with Standard TSs.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

A handwritten signature in cursive script that reads "Peter B. Erickson".

Peter B. Erickson, Project Manager  
Operating Reactors Branch #4  
Division of Licensing

Enclosures:

1. Amendment No. 44
2. Safety Evaluation
3. Notice

cc w/enclosures:  
See next page

Crystal River Unit No. 3  
Florida Power Corporation

50-302

cc w/enclosure(s):

Mr. S. A. Brandimore  
Vice President and General Counsel  
P. O. Box 14042  
St. Petersburg, Florida 33733

Mr. Wilbur Langely, Chairman  
Board of County Commissioners  
Citrus County  
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345 Courtland Street, N.E.  
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668 N. W. First Avenue  
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Administrator  
Department of Environmental Regulation  
Power Plant Siting Section  
State of Florida  
2600 Blair Stone Road  
Tallahassee, Florida 32301

Attorney General  
Department of Legal Affairs  
The Capitol  
Tallahassee, Florida 32304

Mr. Robert B. Borsum  
Babcock & Wilcox  
Nuclear Power Generation Division  
Suite 420, 7735 Old Georgetown Road  
Bethesda, Maryland 20014

Mr. Tom Stetka, Resident Inspector  
U.S. Nuclear Regulatory Commission  
Route #3, Box 717  
Crystal River, Florida 32629

Mr. Dan C. Poole  
Nuclear Plant Manager  
Florida Power Corporation  
P. O. Box 219  
Crystal River, Florida 32629

cc w/enclosure(s) & incoming dtd.:

2/15/80

Bureau of Intergovernmental Relations  
660 Apalachee Parkway  
Tallahassee, Florida 32304



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

FLORIDA POWER CORPORATION  
CITY OF ALACHUA  
CITY OF BUSHNELL  
CITY OF GAINESVILLE  
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CITY OF NEW SMYRNA BEACH AND UTILITIES COMMISSION, CITY OF NEW SMYRNA BEACH  
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ORLANDO UTILITIES COMMISSION AND CITY OF ORLANDO  
SEBRING UTILITIES COMMISSION  
SEMINOLE ELECTRIC COOPERATIVE, INC.  
CITY OF TALLAHASSEE

DOCKET NO. 50-302

CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT  
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 44  
License No. DPR-72

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Florida Power Corporation, et al (the licensees) dated February 15, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

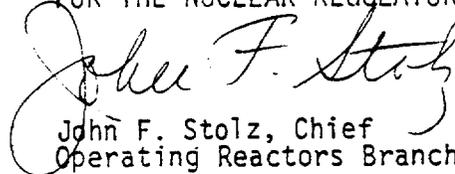
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-72 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 44, are hereby incorporated in the license. Florida Power Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Chief  
Operating Reactors Branch #4  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: November 24, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 44

FACILITY OPERATING LICENSE NO. DPR-72

DOCKET NO. 50-302

replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages

3/4 3-23

3/4 3-24

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3/4 7-21

TABLE 3.3-6

RADIATION MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ALARM/TRIP SETPOINT</u>	<u>MEASUREMENT RANGE</u>	<u>ACTION</u>
1. AREA MONITORS					
a. Fuel Storage Pool Area					
i. Criticality Monitor	1	*	≤15 mr/hr	10 <sup>-1</sup> - 10 <sup>4</sup> mr/hr	14
2. PROCESS MONITORS					
a. Fuel Storage Pool Area					
i. Gaseous Activity - Ventilation System Isolation	1	**	≤2 x background	10 <sup>1</sup> - 10 <sup>6</sup> cpm	16
b. Reactor Building					
i. Gaseous Activity -					
a) Purge Exhaust Duct Isolation	1	6	***	10 <sup>1</sup> - 10 <sup>6</sup> cpm	17
b) RCS Leakage Detection	1	1, 2, 3 & 4	Not Applicable	10 <sup>1</sup> - 10 <sup>6</sup> cpm	15
ii. Iodine Activity - RCS Leakage Detection	1	1, 2, 3 & 4	Not Applicable	10 <sup>1</sup> - 10 <sup>6</sup> cpm	15
c. Control Room					
i. Iodine Activity - Ventilation System Isolation/ Recirculation	1	All Modes	≤2 x background	10 <sup>1</sup> - 10 <sup>6</sup> cpm	18

\*With fuel in the storage pool or building

\*\*With irradiated fuel in the storage pool

\*\*\*Determined by requirements of Appendix "B", Tech. Specs., Section 2.4.2 - Crystal River 3 Operating License No. DPR-72.

TABLE 3.3-6 (Continued)

TABLE NOTATION

- ACTION 14 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, perform area surveys of the monitored area with portable monitoring instrumentation at least once per 24 hours.
- ACTION 15 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, comply with the ACTION requirements of Specification 3.4.6.1.
- ACTION 16 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, comply with the ACTION requirements of Specification 3.9.12.
- ACTION 17 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, comply with the ACTION requirements of Specification 3.9.9.
- ACTION 18 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement; within 1 hour initiate and maintain operation of the control room emergency ventilation system in the recirculation mode of operation.

CRYSTAL RIVER - UNIT 3

3/4 3-25

Amendment No. 17, 44

TABLE 4.3-3

RADIATION MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>MODES IN WHICH SURVEILLANCE REQUIRED</u>
1. AREA MONITORS				
a. Fuel Storage Pool Area				*
i. Criticality Monitor	S	R	M	
2. PROCESS MONITORS				
a. Fuel Storage Pool Area				**
i. Gaseous Activity - Ventilation System Isolation	S	R	M	
b. Reactor Building				
i. Gaseous Activity -				
a) Purge Exhaust Duct Isolation	S	Q	M	6
b) RCS Leakage Detection	S	R	M	1, 2, 3, & 4
ii. Iodine Activity - RCS Leakage Detection	S	R	M	1, 2, 3, & 4
c. Control Room				
i. Iodine Activity - Ventilation System Isolation/ Recirculation	S	R	M	All Modes

\*With fuel in the storage pool or building

\*\*With irradiated fuel in the storage pool

## INSTRUMENTATION

### INCORE DETECTORS

#### LIMITING CONDITION FOR OPERATION

---

3.3.3.2 As a minimum, the incore detectors shall be OPERABLE as specified below.

- a. For AXIAL POWER IMBALANCE measurements:
  1. Nine detectors shall be arranged such that there are three detectors in each of three strings and there are three detectors lying in the same axial plane with one plane at the core mid-plane and one plane in each axial core half.
  2. The axial planes in each core half shall be symmetrical about the core mid-plane.
  3. The detector strings shall not have radial symmetry.
- b. For QUADRANT POWER TILT measurements with the Minimum Incore Detector System:
  1. Two sets of 4 detectors shall lie in each core half. Each set of detectors shall lie in the same axial plane. The two sets in the same core half may lie in the same axial plane.
  2. Detectors in the same plane shall have quarter core radial symmetry.
- c. For QUADRANT POWER TILT measurements with the Symmetric Incore Detector System at least 75% of the detectors in each core quadrant shall be OPERABLE.

APPLICABILITY: When the incore detection system is used for surveillance of:

- a. The AXIAL POWER IMBALANCE, or
- b. The QUADRANT POWER TILT.

#### ACTION:

With less than the specified minimum incore detector arrangement OPERABLE, do not use incore detector measurements to determine AXIAL POWER IMBALANCE or QUADRANT POWER TILT. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

PLANT SYSTEMS

3/4.7.6 FLOOD PROTECTION

LIMITING CONDITION FOR OPERATION

---

3.7.6.1 Flood protection shall be provided for all safety related systems, components and structures when:

- a. The water level of the Gulf of Mexico exceeds 98 feet Plant Datum, at the intake structure, and
- b. A Hurricane Warning is in effect.

APPLICABILITY: At all times.

ACTION:

With the water level at the intake structure above elevation 98 feet Plant Datum and with a Hurricane Warning in effect:

- a. Close all watertight doors providing flood protection for safety related systems, components and structures within 2 hours, and
- b. Be in at least HOT SHUTDOWN within 6 hours; be in at least COLD SHUTDOWN within the next 30 hours.

SURVEILLANCE REQUIREMENTS

---

4.7.6.1.1 When a hurricane Watch or Warning is in effect, the water level at the intake structure shall be determined to be within the limit by:

- a. Measurement at least once per 12 hours when the water level is below elevation 92 feet Plant Datum,
- b. Measurement at least once per 4 hours when the water level is equal to or above elevation 92 feet Plant Datum, and
- c. Measurement at least once per 30 minutes when the water level is equal to or above elevation 94 feet Plant Datum.

4.7.6.1.2 Meteorological forecasts shall be obtained from the National Hurricane Center in Miami, Florida at least once per 2 hours when a Hurricane Warning is in effect.

## PLANT SYSTEMS

### 3/4.7.7 CONTROL ROOM EMERGENCY VENTILATION SYSTEM

#### LIMITING CONDITION FOR OPERATION

---

3.7.7.1 Two independent control room emergency ventilation systems shall be OPERABLE.\*

APPLICABILITY: MODES 1, 2, 3 and 4.

#### ACTION:

With one control room emergency ventilation system inoperable, restore the inoperable system to OPERABLE status within 7 days or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

#### SURVEILLANCE REQUIREMENTS

---

4.7.7.1 Each control room emergency ventilation system shall be demonstrated OPERABLE:

- a. At least once per 12 hours by verifying that the control room air temperature is  $\leq 120^{\circ}\text{F}$ .
- b. At least once per 31 days on a STAGGERED TEST BASIS by initiating, from the control room, flow through the HEPA filters and charcoal adsorbers and verifying that the system operates for at least 15 minutes.
- c. At least once per 18 months or (1) after any structural maintenance on the HEPA filter or charcoal adsorber housings, or (2) following painting, fire or chemical release in any ventilation zone communicating with the system by:

---

\*When the ventilation systems are determined to be inoperable solely because the control complex return duct radiation monitor (RM-A5) is inoperable, the systems may be considered OPERABLE for the purpose of satisfying the requirements of the LCO provided the requirements of Action 18 of Technical Specification 3.3.3.1 are complied with.

## PLANT SYSTEMS

### SURVEILLANCE REQUIREMENTS (Continued)

---

1. Verifying that with the system operating at a flow rate of 43,500 cfm  $\pm$  10% and exhausting through the HEPA filters and charcoal adsorbers, the total bypass flow of the system to the facility vent, including leakage through the system diverting valves, is  $<$  1% when the system is tested by admitting cold DOP at the system intake.
  2. Verifying that the ventilation system satisfies the in-place testing acceptance criteria and uses the test procedures of Regulatory Positions C.5.a, C.5.c\* and C.5.d\* of Regulatory Guide 1.52, Revision 1, July 1976, and the system flow rate is 43,500 cfm  $\pm$  10%.
  3. Verifying within 31 days after removal that a laboratory analysis of a representative carbon sample obtained in accordance with Regulatory Position C.6.b of Regulatory Guide 1.52, Revision 1, July 1976, meets the laboratory testing criteria of Regulatory Position C.6.a of Regulatory Guide 1.52, Revision 1, July 1976.
  4. Verifying a system flow rate of 43,500 cfm  $\pm$  10% during system operation when tested in accordance with ANSI N510-1975.
- d. After every 720 hours of charcoal adsorber operation by verifying within 31 days after removal that a laboratory analysis of a representative carbon sample obtained in accordance with Regulatory Position C.6.b of Regulatory Guide 1.52, Revision 1, July 1976, meets the laboratory testing criteria of Regulatory Position C.6.a of Regulatory Guide 1.52, Revision 1, July 1976.
- e. At least once per 18 months by:
1. Verifying that the pressure drop across the combined HEPA filters and charcoal adsorber banks is  $<$  6 inches Water Gauge while operating the system at a flow rate of 43,500 cfm  $\pm$  10%.
  2. Verifying that on a containment isolation test signal, the system automatically switches into a recirculation mode of operation with flow through the HEPA filters and charcoal adsorber banks.

---

\* The air flow distribution test of Section 8 of ANSI N510-1975 may be performed downstream of the HEPA filters.

PLANT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

---

- f. After each complete or partial replacement of a HEPA filter bank by verifying that the HEPA filter banks remove  $\geq 99\%$  of the DOP when they are tested in-place in accordance with ANSI N510-1975\* while operating the system at a flow rate of 43,500 cfm  $\pm 10\%$ .
- g. After each complete or partial replacement of a charcoal adsorber bank by verifying that the charcoal adsorbers remove  $\geq 99\%$  of a halogenated hydrocarbon refrigerant test gas when they are tested in-place in accordance with ANSI N510-1975\* while operating the system at a flow rate of 43,500 cfm  $\pm 10\%$ .



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
SUPPORTING AMENDMENT NO. 44 TO FACILITY OPERATING LICENSE NO. DPR-72  
FLORIDA POWER CORPORATION, ET AL  
CRYSTAL RIVER UNIT NO. 3 NUCLEAR GENERATING PLANT  
DOCKET NO. 50-302

Introduction

By letter dated February 15, 1980, Florida Power Corporation (FPC) submitted Technical Specification (TS) Change Request No. 58 to revise requirements for testing of Radiation Monitor RM-A5 to be consistent with Standard Technical Specifications (STS). The change would also reduce the potential for unnecessary reactor shutdown.

Evaluation

TS 4.7.7.1.e.2 presently requires the use of a high radiation test signal for the control room ventilation system tests, whereas the operability and surveillance requirements for other radiation monitoring instruments are specified in Tables 3.3-6 and 4.3-3 of the TSs. With the surveillance requirements for Radiation Monitor RM-A5 specified as part of the control room ventilation TSs, the ventilation system may be declared inoperable if RM-A5 is found to be defective during the ventilation test.

The function of the RM-A5 monitor is to cause control room ventilation to switch to the recirculation mode if the radiation setpoint is reached. An adequate compensation for having RM-A5 temporarily out of service is to manually place the control room ventilation system in a recirculation mode. This corrective action is not clearly permitted by the present TSs, however.

The proposed change would (1) put surveillance requirements for RM-A5 in the proper section of the TSs, (2) allow the corrective action discussed above if RM-A5 is determined to be out of service, (3) require more frequent testing of the RM-A5 monitor, and (4) make the operability and surveillance requirements for RM-A5 consistent with STS.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: November 24, 1981

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-302FLORIDA POWER CORPORATION, ET ALNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY  
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 44 to Facility Operating License No. DPR-72, issued to the Florida Power Corporation, City of Alachua, City of Bushnell, City of Gainesville, City of Kissimmee, City of Leesburg, City of New Smyrna Beach and Utilities Commission, City of New Smyrna Beach, City of Ocala, Orlando Utilities Commission and City of Orlando, Sebring Utilities Commission, Seminole Electric Cooperative, Inc., and the City of Tallahassee (the licensees) which revised the Technical Specifications for operation for the Crystal River Unit No. 3 Nuclear Generating Plant (the facility) located in Citrus County, Florida. The amendment is effective as of the date of issuance.

This amendment revises the Technical Specification requirements for testing of Radiation Monitor RM-A5 to be consistent with Standard Technical Specifications.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior

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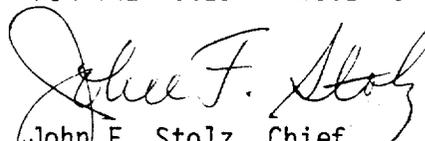
public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated February 15, 1980, (2) Amendment No. 44 to License No. DPR-72, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW, Washington, D.C., and at the Crystal River Public Library, 668 N.W. First Avenue, Crystal River, Florida. A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 24th day of November 1981.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Chief  
Operating Reactors Branch #4  
Division of Licensing