

NOV 23 1982

DOD-016

DISTRIBUTION:
 Docket File
 NRC PDR
 L PDR
 ORB#4 Rdg
 Docket No. 50-302

Gray File +4
 DEisenhut
 RIngram
 SMiner
 Hornstein
 EBlackwood
 OELD
 SECY w/NRC 102

LJHarmon-2
 TBarnhart-4
 LSchneider
 DBrinkman
 ACRS-10
 CMiles
 ASLAB
 RDiggs

Mr. John A. Hancock
 Vice President, Nuclear Operations
 Florida Power Corporation
 ATTN: Manager, Nuclear Operations
 P. O. Box 14042, M.A.C. H-2
 St. Petersburg, Florida 33733

Dear Mr. Hancock:

The Commission has issued the enclosed Amendment No. 60 to Facility Operating License No. DPR-72 for the Crystal River Unit No. 3 Nuclear Generating Plant (CR-3). This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated October 28, 1981.

This amendment (1) clarifies which post-accident monitoring instrument readouts are supplemented with recorders and (2) changes the measurement units for the specification of the Reactor Coolant Total Flow measurement range from percent to lb/hr.

In your letter of October 28, 1981, you proposed a TS change which would decrease from two to one the "minimum channels operable" requirement for the Power Range Nuclear Flux instrumentation in TS Table 3.3-10. We did not approve this TS change for the reasons outlined in the enclosed Safety Evaluation. Should you desire to proceed with this proposed change, you should resubmit TS 3.3.3.6 and Table 3.3-10 in the format and conditions of Babcock and Wilcox Standard Technical Specifications, Rev. 4.

A copy of the Notice of Issuance is also enclosed.

Sincerely,

"ORIGINAL SIGNED BY:"

Sydney Miner, Project Manager
 Operating Reactors Branch #4
 Division of Licensing

8212010329 821123
 PDR ADOCK 05000302
 P PDR

- Enclosures:**
1. Amendment No. 60
 2. Safety Evaluation
 3. Notice

cc w/enclosures: See next page

**F.R. NOTICE
 +
 (AMENDMENT)**

OFFICE	ORB#4:DL	ORB#4:DL	C-ORB#4:DL	AD-OR/DL	OELD		
SURNAME	RIngram	SMiner/ead	JS	Alvares	H. KARNAN		
DATE	11/15/82	11/16/82	11/10/82	11/10/82	11/17/82		



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

November 23, 1982

Docket No. 50-302

Mr. John A. Hancock
Vice President, Nuclear Operations
Florida Power Corporation
ATTN: Manager, Nuclear Operations
P. O. Box 14042, M.A.C. H-2
St. Petersburg, Florida 33733

Dear Mr. Hancock:

The Commission has issued the enclosed Amendment No. 60 to Facility Operating License No. DPR-72 for the Crystal River Unit No. 3 Nuclear Generating Plant (CR-3). This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated October 28, 1981.

This amendment (1) clarifies which post-accident monitoring instrument readouts are supplemented with recorders and (2) changes the measurement units for the specification of the Reactor Coolant Total Flow measurement range from percent to lb/hr.

In your letter of October 28, 1981, you proposed a TS change which would decrease from two to one the "minimum channels operable" requirement for the Power Range Nuclear Flux instrumentation in TS Table 3.3-10. We did not approve this TS change for the reasons outlined in the enclosed Safety Evaluation. Should you desire to proceed with this proposed change, you should resubmit TS 3.3.3.6 and Table 3.3-10 in the format and conditions of Babcock and Wilcox Standard Technical Specifications, Rev. 4.

A copy of the Notice of Issuance is also enclosed.

Sincerely,

M. B. Fairtilepar
Sydney Miner, Project Manager
Operating Reactors Branch #4
Division of Licensing

Enclosures:

1. Amendment No. 60
2. Safety Evaluation
3. Notice

cc w/enclosures: See next page

Crystal River Unit No.
Florida Power Corporation

50-302

cc w/enclosure(s):
Mr. S. A. Brandimore
Florida Power Corporation
Vice President and General Counsel
P. O. Box 14042
St. Petersburg, Florida 33733

Mr. Wilbur Langely, Chairman
Board of County Commissioners
Citrus County
Iverness, Florida 36250

Regional Radiation Representative
EPA Region IV
345 Courtland Street, N.E.
Atlanta, Georgia 30308

Mr. Robert B. Borsum
Babcock & Wilcox
Nuclear Power Generation Division
Suite 220, 7910 Woodmont Avenue
Bethesda, Maryland 20814

Mr. Tom Stetka, Resident Inspector
U.S. Nuclear Regulatory Commission
Route #3, Box 717
Crystal River, Florida 32629

Mr. T. C. Lutkehaus
Nuclear Plant Manager
Florida Power Corporation
P. O. Box 219
Crystal River, Florida 32629

Bureau of Intergovernmental Relations
660 Apalachee Parkway
Tallahassee, Florida 32304

Administrator
Department of Environmental Regulation
Power Plant Siting Section
State of Florida
2600 Blair Stone Road
Tallahassee, Florida 32301

Attorney General
Department of Legal Affairs
The Capitol
Tallahassee, Florida 32304

Mr. James P. O'Reilly, Regional Administrator
U. S. Nuclear Regulatory Commission, Region II
101 Marietta Street, Suite 3100
Atlanta, Georgia 30303



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER CORPORATION
CITY OF ALACHUA
CITY OF BUSHNELL
CITY OF GAINESVILLE
CITY OF KISSIMMEE
CITY OF LEESBURG
CITY OF NEW SMYRNA BEACH AND UTILITIES COMMISSION, CITY OF NEW SMYRNA BEACH
CITY OF OCALA
ORLANDO UTILITIES COMMISSION AND CITY OF ORLANDO
SEBRING UTILITIES COMMISSION
SEMINOLE ELECTRIC COOPERATIVE, INC.
CITY OF TALLAHASSEE

DOCKET NO. 50-302

CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 60
License No. DPR-72

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power Corporation, et al (the licensees) dated October 28, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

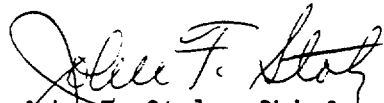
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-72 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 60, are hereby incorporated in the license. Florida Power Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 23, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 60

FACILITY OPERATING LICENSE NO. DPR-72

DOCKET NO. 50-302

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

3/4 3-37

3/4 3-38

INSTRUMENTATION

POST-ACCIDENT INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.3.6 The post-accident monitoring instrumentation channels shown in Table 3.3-10 shall be OPERABLE with readouts on all channels in the control room. Recorders on instruments 1 through 10 shall be OPERABLE.

APPLICABILITY: MODES 1, 2, and 3.

ACTION:

- a. With the number of OPERABLE post-accident monitoring channels less than required by Table 3.3-10, either restore the inoperable channel to OPERABLE status within 30 days, or be in HOT SHUTDOWN within the next 12 hours.
- b. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.3.3.6 Each post-accident monitoring instrumentation channel shall be demonstrated OPERABLE by performance of the CHANNEL CHECK and CHANNEL CALIBRATION operations at the frequencies shown in Table 4.3-7.

TABLE 3.3-10

POST-ACCIDENT MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>MEASUREMENT RANGE</u>	<u>MINIMUM CHANNELS OPERABLE</u>
1. Power Range Nuclear Flux	0 - 125%	2
2. Reactor Building Pressure	0 - 70 psia	2
3. Source Range Nuclear Flux	10^{-1} to 10^6 cps	2
4. Reactor Coolant Outlet Temperature	520°F - 620°F	2 per loop
5. Reactor Coolant Total Flow	0 - 160×10^6 lb./hr.	1
6. RC Loop Pressure	0 - 2500 psig	2
	0 - 600 psig	1
	1700 - 2500 psig	2
7. Pressurizer Level	0 - 320 inches	2
8. Steam Generator Outlet Pressure	0 - 1200 psig	2/steam generator
9. Steam Generator Operating Range Level	0 - 100%	2/steam generator
10. Borated Water Storage Tank Level	0 - 50 feet	2
11. Startup Feedwater Flow	0 - 1.5×10^6 lb/hr.	2
12. Reactor Coolant System Subcooling Margin Monitor	-658°F to +668°F	1
13. PORV Position Indicator (Primary Detector)	N/A	1
14. PORV Position Indicator (Backup Detector)	N/A	0
15. PORV Block Valve Position Indicator	N/A	0
16. Safety Valve Position Indicator (Primary Detector)	N/A	1/Valve
17. Safety Valve Position Indicator (Backup Detector)	N/A	0

CRYSTAL RIVER-UNIT 3

3/4 3-38

Amendment No. ~~38~~ 50



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 60 TO FACILITY OPERATING LICENSE NO. DPR-72

FLORIDA POWER CORPORATION, ET AL

CRYSTAL RIVER UNIT NO. 3 NUCLEAR GENERATING PLANT

DOCKET NO. 50-302

INTRODUCTION

By letter dated October 28, 1981, Florida Power Corporation (FPC) submitted Technical Specification (TS) Change Request No. 71 for Crystal River Unit No. 3 (CR-3) to (1) clarify TS 3.3.3.6 with regard to specifying which post-accident monitoring instrumentation readouts are supplemented with recorders, (2) decrease from two to one the "minimum channels operable" requirement for the Power Range Nuclear Flux instrumentation in TS Table 3.3-10, and (3) specify the measurement range of Reactor Coolant Total Flow from percent to lb/hr to reflect actual indication.

EVALUATION

TS 3.3.3.6 presently requires post-accident monitoring instrumentation channels to be operable with readouts and recorders in the control room. Pursuant to the requirements of NUREG-0578, additional post-accident monitoring channels were installed at CR-3 with only readouts in the control room. TS Amendment No. 38 to TS Table 3.3-10 added this additional instrumentation but did not identify the fact that this new instrumentation contained only readouts and no recorders.

This proposed change to TS 3.3.3.6 will clarify which post-accident monitoring instruments require recorders in the control room.

CR-3 TS Table 3.3-10 presently requires a minimum of two Power Range Nuclear Flux channels to be operable. Babcock and Wilcox (B&W) Standard Technical Specifications (STS), Rev. 4, requires a minimum of one Power Range Nuclear Flux channel to be operable. The B&W STS, Rev. 4, also contain a "Required No. of Channels" column with an associated action statement in STS 3.3.3.6 if the number of channels operable decreases below the number specified. For Power Range Nuclear Flux channels, this "Required No. of Channels" number is two. The present CR-3 TS was formatted on B&W STS, Rev. 3, which does not contain a "Required No. of Channels" column nor an action statement in TS 3.3.3.6 if only one Power Range Nuclear Flux channel was operable.

This proposed change would have made the "Minimum Channels Operable" column of TS Table 3.3-10 inconsistent with the B&W STS, Rev. 4, due to the absence of a "Required No. of Channels" column and an associated action required for this condition. The intent of B&W STS to have two Power Range Nuclear Flux channels operable or be in an action statement is not met. Therefore, this portion of the proposed TS changes is considered unacceptable.

TS Table 3.3-10 indicates the measurement range of the post-accident monitoring instrumentation. The measurement range for Total Reactor Coolant Flow is listed as 0-110% full flow. The actual installed measurement range is 0-160 x 10⁶ lb/hr.

The proposed change to TS Table 3.3-10 would replace the measurement range for Total Reactor Coolant Flow to 0-160 x 10⁶ lb/hr which is consistent with the installed instruments range.

ENVIRONMENTAL CONSIDERATION

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §1.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

CONCLUSION

Based on the considerations discussed above, we have concluded, except for the item decreasing the minimum channels operable for the power range nuclear flux instrument which is not authorized, that: (1) because the amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant reduction in a margin of safety, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: November 23, 1982

The following NRC personnel have contributed to this Safety Evaluation:
Barry Smith, Tom Stetka, John Rogge.

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-302FLORIDA POWER CORPORATION, ET ALNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 60 to Facility Operating License No. DPR-72, issued to the Florida Power Corporation, City of Alachua, City of Bushnell, City of Gainesville, City of Kissimmee, City of Leesburg, City of New Smyrna Beach and Utilities Commission, City of New Smyrna Beach, City of Ocala, Orlando Utilities Commission and City of Orlando, Sebring Utilities Commission, Seminole Electric Cooperative, Inc., and the City of Tallahassee (the licensees) which revised the Technical Specifications (TSs) for operation of the Crystal River Unit No. 3 Nuclear Generating Plant (the facility) located in Citrus County, Florida. The amendment is effective as of the date of issuance.

This amendment (1) clarifies which post-accident monitoring instrument readouts are supplemented with recorders and (2) changes the measurement units for the specification of the Reactor Coolant Total Flow measurement range from percent to lb/hr.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior

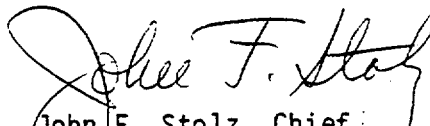
public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated October 28, 1981, (2) Amendment No. 60 to License No. DPR-72, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW, Washington, D.C., and at the Crystal River Public Library, 668 N.W. First Avenue, Crystal River, Florida. A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 23rd day of November 1982.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing