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Docket No. 50-302

Mr. J. A. Hancock
Vice President, Nuclear Operations
Florida Power Corporation
ATTN: Manager, Nuclear Licensing
& Fuel Management
P. O. Box 14042, M.A.C. H-2
St. Petersburg, Florida 33733

Dear Mr. Hancock:

The Commission has issued the enclosed Amendment No. 63 to Facility Operating License No. DPR-72 for the Crystal River Unit No. 3 Nuclear Generating Plant (CR-3). The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated January 11, 1980, as revised November 2, 1981. By telephone, our staffs mutually agreed to modify the requested changes.

This amendment exempts certain containment isolation valves, after the valves have been placed in their containment isolation position, from the provisions of TS 3.0.4.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,
ORIGINAL SIGNED BY
JOHN F. STOLZ

John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Enclosures:
1. Amendment No. 63
2. Safety Evaluation
3. Notice

cc w/enclosures:
See next page

F.R. NOTICE
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DATE	2/24/83	2/24/83	2/25/83	3/2/83	3/7/83	2/25/83	

NRC FORM 318 (10-80) NRCM 0240 OFFICIAL RECORD COPY USGPO: 1981—335-960

cc w/enclosure(s):
Mr. S. A. Brandimore
Florida Power Corporation
Vice President and General Counsel
P. O. Box 14042
St. Petersburg, Florida 33733

Mr. Wilbur Langely, Chairman
Board of County Commissioners
Citrus County
Iverness, Florida 36250

Regional Radiation Representative
EPA Region IV
345 Courtland Street, N.E.
Atlanta, Georgia 30308

Mr. Robert B. Borsum
Babcock & Wilcox
Nuclear Power Generation Division
Suite 220, 7910 Woodmont Avenue
Bethesda, Maryland 20814

Mr. Tom Stetka. Resident Inspector
U.S. Nuclear Regulatory Commission
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Crystal River, Florida 32629

Nuclear Plant Manager
Florida Power Corporation
P. O. Box 219
Crystal River, Florida 32629

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Tallahassee, Florida 32304

Administrator
Department of Environmental Regulation
Power Plant Siting Section
State of Florida
2600 Blair Stone Road
Tallahassee, Florida 32301

Attorney General
Department of Legal Affairs
The Capitol
Tallahassee, Florida 32304

Mr. James P. O'Reilly, Regional Administrator
U. S. Nuclear Regulatory Commission, Region II
101 Marietta Street, Suite 3100
Atlanta, Georgia 30303

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

March 10, 1983

Docket No. 50-302

Mr. J. A. Hancock
Vice President, Nuclear Operations
Florida Power Corporation
ATTN: Manager, Nuclear Licensing
& Fuel Management
P. O. Box 14042, M.A.C. H-2
St. Petersburg, Florida 33733

Dear Mr. Hancock:

The Commission has issued the enclosed Amendment No. 63 to Facility Operating License No. DPR-72 for the Crystal River Unit No. 3 Nuclear Generating Plant (CR-3). The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated January 11, 1980, as revised November 2, 1981. By telephone, our staffs mutually agreed to modify the requested changes.

This amendment exempts certain containment isolation valves, after the valves have been placed in their containment isolation position, from the provisions of TS 3.0.4.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

John F. Stolz

John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

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1. Amendment No. 63
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3. Notice

cc w/enclosures:
See next page

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER CORPORATION
CITY OF ALACHUA
CITY OF BUSHNELL
CITY OF GAINESVILLE
CITY OF KISSIMMEE
CITY OF LEESBURG
CITY OF NEW SMYRNA BEACH AND UTILITIES COMMISSION, CITY OF NEW SMYRNA BEACH
CITY OF OCALA
ORLANDO UTILITIES COMMISSION AND CITY OF ORLANDO
SEBRING UTILITIES COMMISSION
SEMINOLE ELECTRIC COOPERATIVE, INC.
CITY OF TALLAHASSEE

DOCKET NO. 50-302

CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 63
License No. DPR-72

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Florida Power Corporation, et al (the licensees) dated January 11, 1980, as revised November 2, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-72 is hereby amended to read as follows:

   Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 63, are hereby incorporated in the license. Florida Power Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: MAR 10 1983

ATTACHMENT TO LICENSE AMENDMENT NO. 63

FACILITY OPERATING LICENSE NO. DPR-72

DOCKET NO. 50-302

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages
3/4 6-15
3/4 6-17
3/4 6-18
3/4 6-19
3/4 6-20
3/4 6-21a

3/4.6.3 CONTAINMENT ISOLATION VALVES

LIMITING CONDITION FOR OPERATION

3.6.3.1 The containment isolation valves specified in Table 3.6-1 shall be OPERABLE with isolation times as shown in Table 3.6-1.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With one or more of the isolation valve(s) specified in Table 3.6-1 inoperable, either:

a. Restore the inoperable valve(s) to OPERABLE status within 4 hours, or

b. Isolate each affected penetration within 4 hours by use of at least one deactivated automatic valve secured in the isolation position, or

c. Isolate each affected penetration within 4 hours by use of at least one closed manual valve or blind flange; or

d. Be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.3.1.1 The isolation valves specified in Table 3.6-1 shall be demonstrated OPERABLE prior to returning the valve to service after maintenance, repair or replacement work is performed on the valve or its associated actuator, control or power circuit by performance of a cycling test and verification of isolation time.

SURVEILLANCE REQUIREMENTS (Continued)

4.6.3.1.2 Each isolation valve specified in Table 3.6-1 shall be demonstrated OPERABLE during the COLD SHUTDOWN or REFUELING MODE at least once per 18 months by:

a. Verifying that on a containment isolation test signal, each automatic isolation valve actuates to its isolation position.

b. Verifying that on a containment radiation-high test signal, each purge and exhaust automatic valve actuates to its isolation position.

TABLE 3.6-1

CONTAINMENT ISOLATION VALVES

VALVE NUMBER	FUNCTION	ISOLATION TIME (seconds)
A. CONTAINMENT ISOLATION		
1. BSV-27 check #	closed dur. nor. operation and open dur. RB spray	NA
BSV-3 #	"	60
BSV-26 check #	"	NA
BSV-4 #	"	60
2. CAV-126(A)*	iso. CA sys. fr. RC letdn.	60
CAV-1 (A)*	iso. CA sys. fr. pzr.	60
CAV-3 (A)*	"	60
CAV-2 (B)*	iso. CA sys. fr. RB	60
CAV-4 # (A)*	isolate liquid sampling system	60
CAV-6 # (B)*	"	60
CAV-5 # (A)*	"	60
CAV-7 # (B)*	"	60
3. CFV-20 check	iso. N_2 supply fr. CFT-1A	NA
CFV-28 (A/B)*	"	60
CFV-18 check	iso. MU system fr. CFT-1B	NA
CFV-26 (A/B)*	"	60
CFV-19 check	iso. MU system fr. CFT-1A	NA
CFV-25 (A/B)*	"	60
CFV-42 (B)*	iso. liquid sampling fr. CF system	60
CFV-15 (A)*	iso. WD sys. fr. CF tanks	60
CFV-16 (A)*	"	60
CFV-29 (B)*	"	60
CFV-11 (A)*	iso. CF tanks fr. liquid sampling system	60
CFV-12 (A)*	"	60
CFV-17 check	iso. N_2 supply fr. CFT-1B	NA
CFV-27 (A/B)*	"	60

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

	VALVE NUMBER	FUNCTION	ISOLATION TIME (seconds)
4.	CIV-41*	iso. CI sys. fr. RB	60
	CIV-40*	"	60
	CIV-34*	"	60
	CIV-35*	"	60
5.	DHV-93 check	iso. DH system fr. pzr.	NA
	DHV-91*	"	60
	DHV-43 #	iso. DH sys. fr. RB sump	120
	DHV-42 #	"	120
	DHV-4# & 41#	iso. DH sys. fr. RC	120
	DHV-6 #	iso. DH system from Reactor Vessel	60
	DHV-5 #	"	60
6.	DWV-162 check	iso. DW system fr. RB	NA
	DWV-160 (A/B)*	"	60
7.	FWV-44 check #	iso. feedwater from RCSG-1A	NA
	FWV-45 check #	"	NA
	FWV-43 check #	iso. feedwater from RCSG-1B	NA
	FWV-46 check #	"	NA
8.	MSV-130 #(A/B)*	iso. MDT-1 from RCSG-1A	60
	MSV-148 #(A/B)*	iso. MDT-1 from RCSG-1B	60
	MSV-411 # *	iso. main steam lines from RCSG-1A	60
	MSV-412 # *	iso. main steam lines from RCSG-1A	60
	MSV-413 # *	iso. main steam lines from RCSG-1B	60
	MSV-414 # *	iso. main steam lines from RCSG-1B	60

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

	VALVE NUMBER	FUNCTION	ISOLATION TIME (seconds)
9.	MUV-40 (A)*	iso. MU system from RC	60
	MUV-41 (A)*	"	60
	MUV-49 (B)	"	60
	MUV-261	iso. MU system from control bleed-off	60
	MUV-260	"	60
	MUV-259	"	60
	MUV-258	"	60
	MUV-253	"	60
	MUV-163 check #	open during HPI and closed dur. nor. operation	NA
	MUV-25 #	"	60
	MUV-164 check #	"	NA
	MUV-26 #	"	60
	MUV-160 check #	"	NA
	MUV-23 #	"	60
	MUV-161 check #	"	NA
	MUV-24 #	"	60
	MUV-27 #	open dur. nor. operation and closed during RB Isolation	60
10.	SWV-39 #	iso. NSCCC from AHF-1C	60
	SWV-45 #	"	60
	SWV-35 #	iso. NSCCC from AHF-1A	60
	SWV-41 #	"	60
	SWV-37 #	iso. NSCCC from AHF-1B	60
	SWV-43 #	"	60
	SWV-48 #*	isolate NSCCC from MUHE-1A & 1B and WDT-5	60
	SWV-47 #*	"	60
	SWV-49 #*	"	60
	SWV-50 #*	"	60
	SWV-80 #	iso. NSCCC from RCP-1A	60
	SWV-84 #	"	60
	SWV-82 #	iso. NSCCC from RCP-1C	60
	SWV-86 #	"	60

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

VALVE NUMBER		FUNCTION	ISOLATION TIME (seconds)
	SWV-81 #	iso. NSCCC from RCP-1D	60
	SWV-85 #	"	60
	SWV-79 #	iso. NSCCC from RCP-1B	60
	SWV-83 #	"	60
	SWV-109 #	iso. NSCCC from DRRD-1	60
	SWV-110 #	"	60
11.	WDV-4 (B)	iso. WDT-4 from RB sump	60
	WDV-3 (A)	"	60
	WDV-60 (A)*	iso. WDT-4 from WDT-5	60
	WDV-61 (B)*	"	60
	WDV-94 (A)	iso. WDT-4 from WDP-8	60
	WDV-62 (B)	"	60
	WDV-406 (A)*	iso. waste gas disposal from vents in RC system	60
	WDV-405 (B)*	"	60
12.	WSV-3	iso. containment monitoring system from RB	60
	WSV-4	"	60
	WSV-5	"	60
	WSV-6	"	60
B.	CONTAINMENT PURGE AND EXHAUST		
1.	AHV-1C (A)	iso. pur. sup. system fr. RB	60
	AHV-1D (B)	"	60
	AHV-1B (A)	iso. pur. exhaust system fr. RB	60
	AHV-1A (B)	"	60
C.	MANUAL		
1.	IAV-28	iso. IA from RB	NA
	IAV-29	"	NA
2.	LRV-50	iso. leak rate test system from RB	NA
	LRV-36	"	NA

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

	VALVE NUMBER	FUNCTION	ISOLATION TIME (seconds)
	LRV-51	iso. atmos. vent and RB	NA
	LRV-35 & 47	purge exhaust system from RB	NA
	LRV-49	iso. atmos. vent from RB	NA
	LRV-38 & 52	"	NA
	LRV-45	iso. LR test panel from RB	NA
	LRV-44	"	NA
	LRV-46	"	NA
3.	MSV-146#	iso. misc. waste storage tank from RCSG-1B	NA
4.	NGV-62	iso. NG system from steam generators	NA
	NGV-81 #	"	NA
	NGV-82	iso. NG system from pzr.	NA
5.	SAV-24	iso. SA from RB	NA
	SAV-23 & 122	"	NA
6.	SFV-18	iso. SF system	NA
	SFV-19	"	NA
	SFV-119 #	iso. Fuel Transfer tubes from F.T. Canal	NA
	SFV-120 #	"	NA
7.	WSV-1	iso. containment monitoring system from RB	NA
	WSV-2	"	NA

D. PENETRATIONS REQUIRING TYPE B TESTS

Blind Flange 119	iso. RB	NA
Blind Flange 120	"	NA
Blind Flange 202	"	NA

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

VALVE NUMBER	FUNCTION	ISOLATION TIME (seconds)
Blind Flange 348	iso. fuel transfer tube from Transfer Canal	NA
Blind Flange 436	"	NA
Equipment Hatch	iso. RB	NA
Personnel Hatch	iso. RB	NA

Not subject to Type C Leakage Test

* The provisions of Specification 3.0.4 are not applicable.

(A) Isolates on Diverse Isolation Actuation Signal A
(B) Isolates on Diverse Isolation Actuation Signal B
(A/B) Isolates on Diverse Isolation Actuation Signal A or B

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 63 TO FACILITY OPERATING LICENSE NO. DPR-72

FLORIDA POWER CORPORATION, ET AL

CRYSTAL RIVER UNIT NO. 3 NUCLEAR GENERATING PLANT

DOCKET NO. 50-302

Introduction

By letter dated January 11, 1980, as revised November 2, 1981, Florida Power Corporation (FPC or the licensee) proposed a change to the Crystal River Unit 3 (CR-3) Technical Specifications (TSs). This TS change excepts certain containment isolation valves, after the valves have been placed in their containment isolation position, from the provisions of TS 3.0.4. The licensee requested changes to TS 3.6.3.1 and to Table 3.6-1. By telephone, our staffs mutually agreed to modify the request.

Background

At CR-3, certain containment isolation valves became inoperable, recently, thereby placing the plant in a TS Action Statement. This is significant in that if the plant were to be shutdown, subsequently, it would not be permitted to restart per the requirements of TS 3.0.4, until the valves were declared operable; however, TS 3.0.4 does provide for exceptions to this provision. The FPC November 2, 1981 letter was, as stated above, a request for such exceptions. The licensee has since placed these valves in their isolated configuration. Operation of these valves is not needed for an emergency core cooling function, normal plant operation, startup or shutdown. The FPC November 2, 1981, letter requested additional relief from TS 3.0.4 for other such valves in systems penetrating containment. The licensee has experienced difficulty obtaining replacement valves or parts; thus, the plant remains in the Action Statement with the possibility of an extended shutdown for a non-safety related cause.

Evaluation

Exception to TS 3.0.4 is provided for, as stated above, in the TSs. In recent editions of the Standard TSs, such exceptions are granted prior to issuance of the Operating License.

At CR-3, the licensee has placed the inoperable valves in their isolated configuration thus ensuring that the valves will perform their post-accident isolation function. In addition, the isolated valves do not interfere with normal plant operation. We further determined that alternate paths are available for TS sampling requirements should an isolated valve, in a sampling line, fail in the isolated position. We conclude, for the reasons stated above, that the requested changes to the CR-3 TSs in Table 3.6-1 are acceptable.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant reduction in a margin of safety, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: MAR 10 1983

The following NRC personnel have contributed to this Safety Evaluation: M. Fairtile, D. Brinkman.

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-302

FLORIDA POWER CORPORATION, ET AL

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 63 to Facility Operating License No. DPR-72, issued to the Florida Power Corporation, City of Alachua, City of Bushnell, City of Gainesville, City of Kissimmee, City of Leesburg, City of New Smyrna Beach and Utilities Commission, City of New Smyrna Beach, City of Ocala, Orlando Utilities Commission and City of Orlando, Sebring Utilities Commission, Seminole Electric Cooperative, Inc., and the City of Tallahassee (the licensees) which revised the Technical Specifications (TSs) for operation of the Crystal River Unit No. 3 Nuclear Generating Plant (the facility) located in Citrus County, Florida. The amendment is effective as of the date of issuance.

This amendment exempts certain containment isolation valves, after the valves have been placed in their containment isolation position, from the provisions of TS 3.0.4.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior

public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated January 11, 1980, as revised November 2, 1981, (2) Amendment No. 63 to License No. DPR-72, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C., and at the Crystal River Public Library, 668 N. W. First Avenue, Crystal River, Florida. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 10th day of March 1983.

FOR THE NUCLEAR REGULATORY COMMISSION

John F. Stolz

John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing