

October 19, 1999

Mr. James Scarola, Vice President
Shearon Harris Nuclear Power Plant
Carolina Power & Light Company
Post Office Box 165, Mail Code: Zone 1
New Hill, North Carolina 27562-0165

SUBJECT: SHEARON HARRIS NUCLEAR POWER PLANT, UNIT 1 - ISSUANCE OF
AMENDMENT RE: RELOCATION OF TECHNICAL SPECIFICATION
REQUIREMENTS TO QUALITY ASSURANCE PROGRAM DESCRIPTION (TAC
NO. MA5618)

Dear Mr. Scarola:

The Nuclear Regulatory Commission has issued Amendment No. 92 to Facility Operating License No. NPF-63 for the Shearon Harris Nuclear Power Plant (HNP), Unit No. 1, in response to your request dated June 2, 1999, as supplemented on September 1, 1999. This amendment relocates HNP Technical Specification (TS) Section 6.5, "REVIEW AND AUDIT," TS 6.8.2, TS 6.8.3, and TS Section 6.10, "RECORD RETENTION," intact from the HNP TS to the Quality Assurance Program Description currently located in HNP Final Safety Analysis Report Section 17.3. Future changes to the associated relocated TS will be processed in accordance with 10 CFR 50.54(a).

A copy of the related Safety Evaluation is enclosed. Notice of Issuance will be included in the Commission's regular bi-weekly Federal Register notice.

Sincerely,

Original signed by:

Richard J. Laufer, Project Manager, Section 2
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-400

Enclosures:

1. Amendment No. 92 to NPF-63
2. Safety Evaluation

cc w/enclosures:
See next page

Distribution:
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

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Shearon Harris Nuclear Power Plant
Carolina Power & Light Company
Post Office Box 165, Mail Code: Zone 1
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A copy of the related Safety Evaluation is enclosed. Notice of Issuance will be included in the Commission's regular bi-weekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script, reading "Richard J. Laufer".

Richard J. Laufer, Project Manager, Section 2
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-400

Enclosures:

1. Amendment No. 92 to NPF-63
2. Safety Evaluation

cc w/enclosures:
See next page

AMENDMENT NO. 92 TO FACILITY OPERATING LICENSE NO. NPF-63 - HARRIS, UNIT 1

Docket File

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cc: Harris Service List



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

CAROLINA POWER & LIGHT COMPANY, et al.

DOCKET NO. 50-400

SHEARON HARRIS NUCLEAR POWER PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 92
License No. NPF-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Carolina Power & Light Company, (the licensee), dated June 2, 1999, as supplemented on September 1, 1999, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. NPF-63 is hereby amended to read as follows:

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(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, as revised through Amendment No. 92, are hereby incorporated into this license. Carolina Power & Light Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Sheri R. Peterson, Chief, Section 2
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 19, 1999

ATTACHMENT TO LICENSE AMENDMENT NO. 92

FACILITY OPERATING LICENSE NO. NPF-63

DOCKET NO. 50-400

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

<u>Remove Pages</u>	<u>Insert Pages</u>
xviii	xviii
xix	xix
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6-8	-----
6-9	-----
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6.4 TRAINING

6.4.1 A retraining and replacement training program for the unit staff shall be maintained and shall meet or exceed the requirements and recommendations of the September 1979 draft of ANS 3.1, with the exceptions and alternatives noted in Section 1.8 (Personnel Selection and Training) of the FSAR. The initial and requalification training for licensed personnel is through an accredited program based on the systematic approach to training, as allowed by 10 CFR 55.31, 10 CFR 55.59, and Generic Letter 87-07.

6.5 DELETED

(PAGES 6-8 THROUGH 6-15 DELETED)
(NEXT PAGE IS 6-16)

PROCEDURES AND PROGRAMS (Continued)

- g. Quality Assurance Program for effluent and environmental monitoring; and
- h. Fire protection program implementation.
- i. Technical Specification Equipment List Program.

6.8.2 DELETED

6.8.3 DELETED

6.8.4 The following programs shall be established, implemented, and maintained:

a. Primary Coolant Sources Outside Containment

A program to reduce leakage, to as low as practical levels, from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident. The systems include:

- 1. Residual Heat Removal System and Containment Spray System, except spray additive subsystem and RWST,
- 2. Safety Injection System, except boron injection recirculation subsystem and accumulator,
- 3. Portions of the Chemical and Volume Control System:
 - a. Letdown subsystem, including demineralizers,
 - b. Boron re-cycle holdup tanks, and
 - c. Charging/safety injection pumps,
- 4. Post-Accident Sample System,

6.9.1.6 CORE OPERATING LIMITS REPORT (Continued)

6.9.1.6.3 The core operating limits shall be determined so that all applicable limits (e.g., fuel thermal-mechanical limits, core thermal-hydraulic limits, nuclear limits such as shutdown margin, and transient and accident analysis limits) of the safety analysis are met.

6.9.1.6.4 The CORE OPERATING LIMITS REPORT, including any mid-cycle revisions or supplements, shall be provided, upon issuance for each reload cycle, to the NRC Document Control Desk, with copies to the Regional Administrator and Resident Inspector.

SPECIAL REPORTS

6.9.2 Special reports shall be submitted to the NRC in accordance with 10CFR50.4 within the time period specified for each report.

6.10 DELETED

(PAGE 6-25 DELETED)

6.11 RADIATION PROTECTION PROGRAM

6.11.1 Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained, and adhered to for all operations involving personnel radiation exposure.

6.12 HIGH RADIATION AREA

6.12.1 Pursuant to Paragraph 20.203(c)(5) of 10 CFR Part 20, in lieu of the "control device" or "alarm signal" required by Paragraph 20.203(c), each high radiation area, as defined in 10 CFR Part 20, in which the intensity of radiation is equal to or less than 1000 mR/h at 45 cm (18 in.) from the radiation source or from any surface which the radiation penetrates shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit (RWP). Individuals qualified in radiation protection procedures (e.g., Health Physics Technicians) or personnel continuously escorted by such individuals may be exempt from the RWP issuance requirement during the performance of their assigned duties in high radiation areas with exposure rates equal to or less than 1000 mR/h, provided they are otherwise following plant radiation protection procedures for entry into such high radiation areas. Any individual or group of individuals permitted to enter such areas shall be provided with or accompanied by one or more of the following:

- a. A radiation monitoring device that continuously indicates the radiation dose rate in the area; or
- b. A radiation monitoring device that continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate levels in the area have been established and personnel have been made knowledgeable of them; or
- c. An individual qualified in radiation protection procedures, with a radiation dose rate monitoring device, who is responsible for providing positive control over the activities within the area and who shall perform periodic radiation surveillance at the frequency specified by the Radiation Control Supervisor in the RWP.

6.12.2 In addition to the requirements of Specification 6.12.1, accessible areas with radiation levels greater than 1000 mR/h at 45 cm (18 in.) from the radiation source or from any surface which the radiation penetrates, shall be provided with locked doors to prevent unauthorized entry, and the keys shall be maintained under the administrative control of the Superintendent-Shift Operations on duty and/or health physics supervision. Doors shall remain

HIGH RADIATION AREA (Continued)

locked except during periods of access by personnel under an approved RWP which shall specify the dose rate levels in the immediate work areas and the maximum allowable stay time for individuals in that area. In lieu of the stay time specification of the RWP, direct or remote (such as closed circuit TV cameras) continuous surveillance may be made by personnel qualified in radiation protection procedures to provide positive exposure control over the activities being performed within the area. During emergency situations that involve personal injury or actions taken to prevent major equipment damage, continuous surveillance and radiation monitoring of the work area by a qualified individual may be substituted for the routine RWP procedure.

For accessible individual high radiation areas, with radiation levels of greater than 1000 mR/h, that are located within large areas, such as PWR containment, where no enclosure exists for purposes of locking and where no enclosure can be reasonably constructed around the individual area, that individual area shall be barricaded and conspicuously posted, and a flashing light shall be activated as a warning device.

6.13 PROCESS CONTROL PROGRAM (PCP)

Changes to the PCP:

- a. Shall be documented and records of reviews performed shall be retained as required by FSAR Section 17.3. This documentation shall contain:
 - 1) Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s) and
 - 2) A determination that the change will maintain the overall conformance of the solidified waste product to existing requirements of Federal, State, or other applicable regulations.
- b. Shall become effective after review and acceptance by the PNSC and the approval of the Plant General Manager.

6.14 OFFSITE DOSE CALCULATION MANUAL (ODCM)

Changes to the ODCM:

- a. Shall be documented and records of reviews performed shall be retained as required by FSAR Section 17.3. This documentation shall contain:
 - 1) Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s) and



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

CAROLINA POWER & LIGHT COMPANY

SHEARON HARRIS NUCLEAR POWER PLANT, UNIT 1

DOCKET NO. 50-400

1.0 INTRODUCTION

By letter dated June 2, 1999, as supplemented on September 1, 1999, Carolina Power & Light Company (CP&L, the licensee) requested a revision to the Technical Specifications (TS) for the Shearon Harris Nuclear Power Plant (HNP). The proposed amendment would relocate HNP TS Section 6.5, "REVIEW AND AUDIT," TS 6.8.2, TS 6.8.3, and TS Section 6.10, "RECORD RETENTION," intact from the HNP TS to the Quality Assurance Program Description (QAPD) currently located in HNP Final Safety Analysis Report Section 17.3. Future changes to the associated relocated TS would be processed in accordance with 10 CFR 50.54(a). The proposed change is consistent with NUREG-1431, Revision 1, "Standard Technical Specifications, Westinghouse Plants," dated April 1995, and with the guidance provided in NRC Administrative Letter 95-06, "Relocation of Technical Specification Administrative Controls related To Quality Assurance," dated December 12, 1995.

The September 1, 1999, submittal provided a copy of the revisions made to the Final Safety Analysis Report (FSAR) Section 17.3, "HNP Quality Assurance Program Description," to incorporate the relocated TS requirements. This submittal contained clarifying information only and did not change the initial no significant hazards consideration determination.

2.0 BACKGROUND

Section 182a of the Atomic Energy Act (the "Act") requires applicants for nuclear power plant operating licenses to include TS as part of the license. The Commission's regulatory requirements related to the content of TS are set forth in 10 CFR 50.36. That regulation requires that the TS include items in five specific categories, including (1) safety limits, limiting safety system settings and limiting control settings; (2) limiting conditions for operation; (3) surveillance requirements; (4) design features; and (5) administrative controls. However, the regulation does not specify the particular requirements to be included in a plant's TS.

With respect to limiting conditions for operations (LCOs), 10 CFR 50.36 provides four criteria to be used in determining whether particular safety functions are required to be included in the TS. While the four criteria specifically apply to LCOs, in adopting the revision to the rule the Commission indicated that the intent of these criteria can be utilized to identify the optimum set of administrative controls in the TS (60 FR 36957). Addressing administrative controls, 10 CFR 50.36 states that they "are the provisions relating to organization and management, procedures, recordkeeping, review and audit, and reporting necessary to assure operation of

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the facility in a safe manner." The specific content of the administrative controls section of the TS is therefore that information that the Commission deems essential for the safe operation of the facility that is not already adequately covered by other regulations. Accordingly, the staff has determined that requirements that are not specifically required under §50.36(c)(6) and which are not otherwise necessary to obviate the possibility of an abnormal situation or event giving rise to an immediate threat to the public health and safety, can be removed from administrative controls. Existing TS requirements, therefore, may be relocated to more appropriate documents (e.g. Security Plan, Quality Assurance (QA) Plan, and Emergency Plan) and controlled by the applicable regulatory requirement. Similarly, while the required content of TS administrative controls is specified in 10 CFR 50.36(c)(6), particular details of administrative controls may be relocated to licensee-controlled documents where §50.54, §50.59, or other regulations provide adequate regulatory control.

3.0 EVALUATION

On December 12, 1995, the NRC issued Administrative Letter (AL) 95-06, "Relocation of Technical Specification Administrative Controls Related to Quality Assurance," which provided guidance for relocating TS administrative requirements. AL 95-06 specifically addressed the acceptability of relocating requirements related to review and audits, the procedure review process, and record retention, from the TS to the QAPD.

The licensee has proposed relocating the following TS requirements, intact, to the QAPD: (1) TS Section 6.5, "Review and Audit;" (2) TS 6.8.2 and TS 6.8.3 related to procedure reviews; and (3) TS Section 6.10, "Record Retention." The licensee's September 1, 1999, submittal, shows the revision that has been made to the QAPD, FSAR Section 17.3, to duplicate the current TS Section 6.5, TS 6.8.2, TS 6.8.3, and TS Section 6.10 requirements in the QAPD.

TS Section 6.5, "Review and Audit"

The provisions in the QAPD implement the Commission's regulations pertaining to the review and audit functions. Inclusion of these particular provisions in TS is not necessary to assure safe operation of the facility. The review and audit functions define an administrative framework to confirm that plant activities have been properly conducted in a safe manner. The reviews and audits serve to provide a cohesive program that provides senior level utility management with assessments of facility operation and recommended actions to improve safety and reliability. Based on the relocation of the review and audit provisions to the QAPD, it is not necessary to include redundant or additional requirements in the TS administrative controls.

TS 6.8.2 and TS 6.8.3 related to procedure reviews

The provisions in the QAPD implement the Commission's regulations pertaining to the control of procedures. The procedure review and approval functions currently in TS define an administrative framework to ensure that documents are reviewed for adequacy and approved for release by authorized personnel. The required control of these processes in the regulations and the revised QAPD is considered to be redundant and functionally equivalent to the provisions currently in TS. The staff has determined that the procedure review and approval functions are adequately addressed by existing regulations and the related QAPD

commitments. Based on the relocation of the procedure review provisions to the QAPD, it is not necessary to include redundant or additional requirements in the TS administrative controls.

TS Section 6.10, "Record Retention"

The provisions in the QAPD implement the Commission's regulations pertaining to the maintenance of records. The required controls related to record retention specified in various regulations and the provision incorporated into the QAPD are considered to be redundant to the requirements currently in TS. The staff has determined that record retention requirements are adequately addressed by existing regulations and the related QAPD commitments. Based upon the relocation of the record retention provisions to the QAPD, it is not necessary to include redundant or additional requirements in the TS administrative controls.

Summary

The staff finds that the proposed relocated TS requirements are adequately addressed by the QAPD. Changes to the QAPD are controlled in accordance with 10 CFR 50.54(a) and include requirements for prior NRC review and approval if a change constitutes a reduction in the QAPD commitment. The staff has determined that it is not necessary to include redundant or additional requirements in the TS Administrative Controls section. The licensee's proposed change is consistent with the guidance of AL 95-06, and with NUREG-1431, Revision 1, "Standard Technical Specifications, Westinghouse Plants," dated April 1995. Based on the above, the staff finds that the relocation of TS Section 6.5, TS 6.8.2, TS 6.8.3, and TS Section 6.10 from the TS to the QAPD is acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the State of North Carolina official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

This amendment relates to changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need to be prepared in connection with the issuance of this amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: R. Laufer

Date: October 19, 1999

Mr. James Scarola
Carolina Power & Light Company

Shearon Harris Nuclear Power Plant
Unit 1

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