

February 28, 2002

Mr. Dale E. Young, Vice President
Crystal River Nuclear Plant (NA1B)
ATTN: Supervisor, Licensing & Regulatory Programs
15760 W. Power Line Street
Crystal River, Florida 34428-6708

SUBJECT: BULLETIN 2001-01, "CIRCUMFERENTIAL CRACKING OF REACTOR
PRESSURE VESSEL HEAD PENETRATION NOZZLES," RESPONSES FOR
CRYSTAL RIVER UNIT 3 (TAC NO. MB2623)

Dear Mr. Young:

On August 3, 2001, the U.S. Nuclear Regulatory Commission (NRC) staff issued Bulletin 2001-01, "Circumferential Cracking of Reactor Pressure Vessel Head Penetration Nozzles," to the industry requesting that addressees provide information related to the structural integrity of the reactor pressure vessel head penetration (VHP) nozzles for their respective facilities, including the extent of VHP nozzle leakage and cracking that has been found to date, the inspections and repairs that have been undertaken to satisfy applicable regulatory requirements, and the basis for concluding that their plans for future inspections will ensure compliance with applicable regulatory requirements at their respective pressurized water reactor plants. You were requested to respond to Items 1 and 3 of the Bulletin within 30 days of its issuance.

You provided your Bulletin response by letter dated August 30, 2001, indicating that your plant is classified as having moderate susceptibility to primary water stress corrosion cracking in the VHP nozzles, based on a relative susceptibility ranking of 5 to 30 effective full-power years from the Oconee Nuclear Station, Unit 3, condition. The staff finds that you have provided the requested information. In your response, you committed to perform an effective visual examination of 100 percent of the VHP nozzles during the September 2001 refueling outage. This is consistent with the description for such an examination provided in Bulletin 2001-01. The NRC staff has concluded that the inspection schedule and scope described in your Bulletin response provide reasonable assurance that the public health and safety will be maintained at your plant.

You also provided a response to Item 5 of the Bulletin on November 19, 2001, within 30 days after plant restart as requested in Bulletin 2001-01. This response described the extent of VHP nozzle leakage and cracking detected at your plant, including the number, location, size, and nature of each crack detected, and a description of the inspections (type, scope, qualification requirements, and acceptance criteria), repairs, and other corrective actions you have taken to satisfy applicable regulatory requirements. This response noted that an engineering evaluation

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subsequent to your Bulletin 2001-01 response determined that the visual examination completed during the September 29, 2001, refueling outage meets the intent of the qualified visual examination described in Bulletin 2001-01.

If you have any questions, please feel free to give me a call at (301) 415-1437.

Sincerely,

/RA/

John M. Goshen, Project Manager, Section 2
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-302

cc: See next page

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Sincerely,

/RA/

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Florida Power Corporation

**CRYSTAL RIVER UNIT NO. 3
GENERATING PLANT**

cc:

Mr. R. Alexander Glenn
Associate General Counsel (MAC-BT15A)
Florida Power Corporation
P.O. Box 14042
St. Petersburg, Florida 33733-4042

Chairman
Board of County Commissioners
Citrus County
110 North Apopka Avenue
Inverness, Florida 34450-4245

Mr. Daniel L. Roderick
Plant General Manager
Crystal River Nuclear Plant (NA2C)
15760 W. Power Line Street
Crystal River, Florida 34428-6708

Ms. Sherry L. Bernhoft
Manager Regulatory Affairs
Crystal River Nuclear Plant (NA2H)
15760 W. Power Line Street
Crystal River, Florida 34428-6708

Mr. Michael A. Schoppman
Framatome ANP
1911 North Ft. Myer Drive, Suite 705
Rosslyn, Virginia 22209

Senior Resident Inspector
Crystal River Unit 3
U.S. Nuclear Regulatory Commission
6745 N. Tallahassee Road
Crystal River, Florida 34428

Mr. William A. Passetti, Chief
Department of Health
Bureau of Radiation Control
2020 Capital Circle, SE, Bin #C21
Tallahassee, Florida 32399-1741

Mr. Richard L. Warden
Manager Nuclear Assessment
Crystal River Nuclear Plant (NA2C)
15760 W. Power Line Street
Crystal River, Florida 34428-6708

Attorney General
Department of Legal Affairs
The Capitol
Tallahassee, Florida 32304

Mr. Joe Myers, Director
Division of Emergency Preparedness
Department of Community Affairs
2740 Centerview Drive
Tallahassee, Florida 32399-2100