

December 31, 1998

Mr. James Scarola, V sident
Shearon Harris Nuclear Power Plant
Carolina Power & Light Company
Post Office Box 165, Mail Code: Zone 1
New Hill, North Carolina 27562-0165

SUBJECT: ISSUANCE OF AMENDMENT NO. 86 TO FACILITY OPERATING LICENSE
NO. NPF-63 REGARDING SURVEILLANCE REQUIREMENTS FOR THE
EMERGENCY CORE COOLING ACCUMULATORS - SHEARON HARRIS
NUCLEAR POWER PLANT, UNIT 1 (TAC NO.M98189)

Dear Mr. Scarola:

The Nuclear Regulatory Commission has issued Amendment No. 86 to Facility Operating License No. NPF-63 for the Shearon Harris Nuclear Power Plant, Unit 1. This amendment changes the Technical Specifications (TS) in response to your request dated March 10, 1997, as supplemented by letters dated May 23, 1997, and October 15, 1998. This amendment revises TS 3.5.1, "Emergency Core Cooling System (ECCS) Accumulators," by (1) increasing the allowed outage time (from 1 hour to 72 hours) that one ECCS accumulator can be inoperable as a result of the boron concentration being outside of TS limits and (2) modifying surveillance requirement 3/4.5.1 consistent with the guidance provided in NUREG-1366, "Improvements to Technical Specifications Surveillance Requirements," December 1992, and the Standard Technical Specifications (STS) for Westinghouse Plants, NUREG-1431, Revision 1.

A copy of the related Safety Evaluation is enclosed. Notice of Issuance will be included in the Commission's regular bi-weekly Federal Register notice.

Sincerely,

Original signed by:

Scott C. Flanders, Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

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Docket No. 50-400

Enclosures:

1. Amendment No. 86 to NPF-63
2. Safety Evaluation

cc w/enclosures: See next page

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NAME	SFlanders	EDunnington	TCollins	11/7/98	FHebdon
DATE	11/24/98	11/24/98	12/21/98	12/30/98	12/31/98
COPY	Yes/No	Yes/No	Yes/No	Yes/No	Yes/No

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December 31, 1998

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Original signed by:

Scott C. Flanders, Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

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NAME	SFlanders	EDunnington ETD	TCollins [Signature]	[Signature]	FHebdon [Signature]
DATE	11/24/98	11/24/98	11/21/98	12/30/98	12/31/98
COPY	Yes/No	Yes/No	Yes/No	Yes/No	Yes/No

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#134



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

December 31, 1998

Mr. James Scarola, Vice President
Shearon Harris Nuclear Power Plant
Carolina Power & Light Company
Post Office Box 165, Mail Code: Zone 1
New Hill, North Carolina 27562-0165

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Sincerely,

A handwritten signature in black ink, appearing to read "Scott C. Flanders".

Scott C. Flanders, Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-400

Enclosures:

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2. Safety Evaluation

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AMENDMENT NO. 86 TO FACILITY OPERATING LICENSE NO. NPF-63 - HARRIS, UNIT 1

Docket File
PUBLIC
PDII-1 Reading
J. Zwolinski
OGC
G. Hill (2)
W. Beckner
T. Collins
ACRS
OPA
OC/LFMB
L. Plisco, RII

cc: Harris Service List

AMENDMENT NO. 86 TO FACILITY OPERATING LICENSE NO. NPF-63 - HARRIS, UNIT 1

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cc: Harris Service List

Mr. James Scarola
Carolina Power & Light Company

Shearon Harris Nuclear Power Plant
Unit 1

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

CAROLINA POWER & LIGHT COMPANY, et al.

DOCKET NO. 50-400

SHEARON HARRIS NUCLEAR POWER PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 86
License No. NPF-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Carolina Power & Light Company, (the licensee), dated March 10, 1997, as supplemented by letters dated May 23, 1997, and October 15, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. NPF-63 is hereby amended to read as follows:

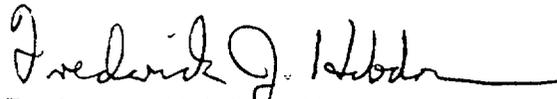
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(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, as revised through Amendment No. 86, are hereby incorporated into this license. Carolina Power & Light Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Frederick J. Hebdon, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 31, 1998

ATTACHMENT TO LICENSE AMENDMENT NO. 86

FACILITY OPERATING LICENSE NO. NPF-63

DOCKET NO. 50-400

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Remove Pages

3/4 5-1
3/4 5-2
B3/4 5-1

Insert Pages

3/4 5-1
3/4 5-2
B3/4 5-1
B3/4 5-1a

3/4.5 EMERGENCY CORE COOLING SYSTEMS

3/4.5.1 ACCUMULATORS

COLD LEG INJECTION

LIMITING CONDITION FOR OPERATION

3.5.1 Each Reactor Coolant System (RCS) accumulator shall be OPERABLE with:

- a. The isolation valve open with power supply circuit breaker open,
- b. A contained borated water volume of between 66 and 96% indicated level,
- c. A boron concentration of between 2400 and 2600 ppm, and
- d. A nitrogen cover-pressure of between 585 and 665 psig.

APPLICABILITY: MODES 1, 2, and 3*.

ACTION:

- a. With one accumulator inoperable, except as a result of a closed isolation valve or boron concentration not within limits, restore the inoperable accumulator to OPERABLE status within 1 hour or be in at least HOT STANDBY within the next 6 hours and reduce pressurizer pressure to less than 1000 psig within the following 6 hours.
- b. With one accumulator inoperable due to the isolation valve being closed, either immediately open the isolation valve or be in at least HOT STANDBY within 6 hours and reduce pressurizer pressure to less than 1000 psig within the following 6 hours.
- c. With one accumulator inoperable due to boron concentration not within limits, restore the boron concentration within limits within 72 hours or be in at least HOT STANDBY within the next 6 hours and reduce pressurizer pressure to less than 1000 psig within the following 6 hours.

SURVEILLANCE REQUIREMENTS

4.5.1.1 Each accumulator shall be demonstrated OPERABLE:

- a. At least once per 12 hours by:
 1. Verifying that the contained borated water volume and nitrogen cover-pressure in the tanks are within their limits, and
 2. Verifying that each accumulator isolation valve is open.

*RCS pressure above 1000 psig.

EMERGENCY CORE COOLING SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- b. At least once per 31 days and within 6 hours after each solution volume increase of greater than or equal to 76 gallons, which is equivalent to an indicated level change of 9%, by verifying the boron concentration of the accumulator solution#; and
- c. At least once per 31 days when the RCS pressure is above 1000 psig by verifying that the circuit breaker supplying power to the respective isolation valve operator is open.

This surveillance is not required when the volume increase makeup source is the Refueling Water Storage Tank (RWST) and the RWST has not been diluted since verifying that the RWST boron concentration is equal to or greater than the accumulator boron concentration limit.

3/4.5 EMERGENCY CORE COOLING SYSTEMS

BASES

3/4.5.1 ACCUMULATORS

The OPERABILITY of each Reactor Coolant System (RCS) accumulator ensures that a sufficient volume of borated water will be immediately forced into the reactor core through each of the cold legs in the event the RCS pressure falls below the pressure of the accumulators. This initial surge of water into the core provides the initial cooling mechanism during large RCS pipe ruptures.

The limits on accumulator volume, boron concentration and pressure ensure that the assumptions used for accumulator injection in the safety analysis are met. The value of 66% indicated level ensures that a minimum of 7440 gallons is maintained in the accumulators. The maximum indicated level of 96% ensures that an adequate volume exists for nitrogen pressurization.

The accumulator power operated isolation valves are considered to be "operating bypasses" in the context of IEEE Std. 279-1971, which requires that bypasses of a protective function be removed automatically whenever permissive conditions are not met. In addition, as these accumulator isolation valves fail to meet single failure criteria, removal of power to the valves is required.

The limits for operation with an accumulator inoperable for any reason except an isolation valve closed or boron concentration not within limits minimizes the time exposure of the plant to a LOCA event occurring concurrent with failure of an additional accumulator which may result in unacceptable peak cladding temperatures. The boron in the accumulators contributes to the assumption that the combined ECCS water in the partially recovered core during the early reflooding phase of a large break LOCA is sufficient to keep that portion of the core subcritical. One accumulator below the minimum boron concentration limit, however, will have no effect on the available ECCS water and an insignificant effect on core subcriticality during reflood. Boiling of ECCS water in the core during reflood concentrates boron in the saturated liquid that remains in the core. In addition, current analysis demonstrates that the accumulators do not discharge following a large steam line break for HNP. Therefore, 72 hours is permitted to return the boron concentration to within limits. If a closed isolation valve cannot be immediately opened, the full capability of one accumulator is not available and prompt action is required to place the reactor in a mode where this capability is not required.

3/4.5.2 AND 3/4.5.3 ECCS SUBSYSTEMS

The OPERABILITY of two independent ECCS subsystems ensures that sufficient emergency core cooling capability will be available in the event of a LOCA assuming the loss of one subsystem through any single failure consideration. Either subsystem operating in conjunction with the accumulators is capable of supplying sufficient core cooling to limit the peak cladding temperatures within acceptable limits for all postulated break sizes ranging from the double ended break of the largest RCS cold leg pipe downward. In addition, each ECCS subsystem provides long-term core cooling capability in the recirculation mode during the accident recovery period.

3/4.5 EMERGENCY CORE COOLING SYSTEMS

BASES

ECCS SUBSYSTEMS (Continued)

With the RCS temperature below 350°F, one OPERABLE ECCS subsystem is acceptable without single failure consideration on the basis of the stable reactivity condition of the reactor and the limited core cooling requirements.

The limitation for a maximum of one charging/safety injection pump to be OPERABLE and the Surveillance Requirement to verify one charging/safety injection pump OPERABLE below 325°F provides assurance that a mass addition pressure transient can be relieved by the operation of a single PORV.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 86 TO FACILITY OPERATING LICENSE NO. NPF-63

CAROLINA POWER & LIGHT COMPANY

SHEARON HARRIS NUCLEAR POWER PLANT, UNIT 1

DOCKET NO. 50-400

1.0 INTRODUCTION

By letter dated March 10, 1997, as supplemented by letters dated May 23, 1997, and October 15, 1998, the Carolina Power & Light Company (CP&L or the licensee) submitted a request for changes to the Harris Nuclear Plant (HNP), Unit 1, Technical Specifications (TS). Specifically, CP&L proposed to modify TS 3/4.5.1 by:

- Extending the allowed outage time (AOT) when one emergency core cooling system (ECCS) accumulator is inoperable due to the boron concentration not being within the TS limits. The licensee proposes to extend the time required to restore the boron concentration of the inoperable ECCS accumulator from 1 hour to 72 hours.
- Changing surveillance requirement 3/4.5.1 consistent with the recommendations in NUREG-1366, "Improvements to Technical Specifications Surveillance Requirements," December 1992, and the Westinghouse Improved Standard Technical Specifications.

The May 23, 1997, letter provided modified TS pages which incorporate the changes from Amendment No. 71. Amendment No. 71 independently revised a TS page that was included in the March 10, 1997, submittal. The supplemental information provided on May 23, 1997, and October 15, 1998, did not change the initial proposed no significant hazards consideration determination.

2.0 EVALUATION

The ECCS accumulators are pressure vessels partially filled with borated water and pressurized with nitrogen gas. The function of the ECCS accumulators is to supply borated water to the reactor coolant system (RCS) when the RCS pressure drops below the accumulator nitrogen pressure. The ECCS accumulators are isolated from the RCS by two swing disc check valves. The limiting conditions for operation of the ECCS accumulators are described in TS 3.5.1. TS 3.5.1 includes the accumulator operating limits for the borated water volume, boron

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concentration, and nitrogen cover pressure. Surveillance Requirement 4.5.1 describes how operability of the ECCS accumulators is to be demonstrated. The licensee proposes to modify TS 3.5.1 and Surveillance Requirement 4.5.1.

2.1 Technical Specification 3.5.1

The licensee is requesting to extend the period that the boron concentration of one accumulator can be outside of the TS limits. The TS Bases state that the purpose for the boron concentration limits is to ensure that the assumptions used for the accumulator injection in the safety analysis are met. For HNP, the analysis in which the accumulators are primarily required is the large break loss-of-coolant accident (LBLOCA). When the RCS pressure falls below the pressure of the accumulators and they begin to inject, most of the water provided does not enter the core, but instead goes out the break. The HNP analyses assume that water from the accumulators does not begin to reach the core until the "beginning-of-core-recovery" (BOCREC).

During this phase (BOCREC) of a LBLOCA system voids provide the primary reactivity control; however, the total ECCS flow of borated water supplements the reactivity control. The HNP analyses show that the accumulators inject for 20 seconds, but only 2 to 2.5 seconds is after BOCREC. As a result, only about 1000 gallons of accumulator volume actually reaches the core. The majority of the reflood volume is provided by the borated water injected from the high- and low-head ECCS pumps, which inject at a rate of approximately 2500 gallons per minute (gpm). The supply of borated water for the high- and low-head ECCS pumps is the refueling water storage tank (RWST). The RWST volume, temperature, and boron concentration is controlled by TS 3.5.4. If the concentration is outside of the TS band, then the concentration must be restored within 1 hour or the plant must be in Hot Standby within 6 hours and Cold Shutdown within the following 30 hours. Therefore, if one ECCS accumulator has no boron, it would not affect the volume of water available for reflood, and the impact on core subcriticality would be insignificant.

CP&L also stated that, because the accumulator is a static system which has very slow changes in conditions and there are no routine makeups or discharges from the system, with the exception of the monthly samples to verify boron concentration, it is unlikely that the accumulator boron concentration will be far outside of the TS limit. In addition, the proposed change is consistent with the Standard Technical Specifications for Westinghouse Plants, NUREG-1431, Rev 1. Therefore, based on the above discussion, the staff concludes that the proposed change to extend the AOT from 1 hour to 72 hours for one ECCS accumulator boron concentration being outside TS limits is acceptable.

2.2 Surveillance Requirement 4.5.1

The NRC staff completed a comprehensive examination of surveillance requirements in TS that require testing during power operations. The effort was part of the NRC TS Improvement Program. The results of this effort was documented in NUREG-1366. In NUREG-1366, the staff concluded that, while the majority of testing at power is important, safety can be improved, equipment degradation decreased, and unnecessary burden on personnel resources can be eliminated by reducing the amount of testing that the TS require during power operation. To this end, the staff identified changes that could be made in the ECCS accumulator surveillance requirements. The NRC staff issued Generic Letter (GL) 93-05, "Line-Item Technical

Specifications Improvements To Reduce Surveillance Requirements For Testing During Power Operation,” which provided guidance to licensees on how to prepare license amendments for the recommendations included in NUREG-1366. GL 93-05 included the NUREG-1366 recommendation for each item and for some recommendations, the actual TS wording. The GL also stated that licensees should determine and include a statement that all proposed TS changes are compatible with plant operating experience and consistent with the guidance.

2.2.1 Accumulator Water Level and Pressure Channel Surveillance (TS 4.5.1.1a)

GL 93-05 recommended that TS 4.5.1.1a.1 be modified by (1) removing the reference that accumulator operability is verified by “the absence of alarms,” and (2) adding clarification that verification is by noting that the parameters are within limit. This configuration provides adequate assurance that the accumulator can perform its intended function. CP&L modified the HNP TS 4.5.1.1a.1 consistent with the recommendations in GL 93-05. Therefore, the staff finds the proposed change acceptable.

2.2.2 Accumulator Boron Concentration Surveillance (TS 4.5.1.1b)

GL 93-05 recommended to clarify TS 4.5.1.1b by adding a note that specified when this surveillance was not required. The proposed note stated:

“This surveillance is not required when the volume increase makeup source is the RWST and the RWST has not been diluted since verifying that the RWST boron concentration is equal to or greater than the accumulator boron concentration”

The purpose of this surveillance is to identify whether inleakage has caused a reduction in the boron concentration to below the required TS limits. When the increase makeup source is from the RWST and the RWST boron concentration is known to be greater than or equal to the ECCS accumulator boron concentration, it is clear that the ECCS accumulator boron concentration has not been diluted. Therefore, it is not necessary to perform the surveillance. CP&L modified the HNP TS 4.5.1.1b consistent with the recommendations provided in GL 93-05. Therefore, the staff finds the proposed TS change acceptable.

2.2.3 Accumulator Water Level and Pressure Instrumentation Surveillance (TS 4.5.1.2)

GL 93-05 states that the water level and pressure channel surveillance may be removed from the TS but should be retained as an existing plant procedure requirement that may be subsequently modified under plant change control procedures and that related requirements of the Administrative Controls Section of the TS.

CP&L proposed to remove this surveillance from the HNP TS. CP&L stated that the calibration requirements for accumulator instrumentation are contained in existing plant maintenance procedures. Changes to the maintenance procedures are adequately controlled in accordance with plant change control procedures, the requirements of the Administrative Controls section of the TS, and 10 CFR 50.59. This surveillance is not required to be in TS under 10 CFR 50.36(c)(3), in that it is not needed to (1) assure that the necessary quality of the ECCS accumulators or any other systems and components, (2) assure that the facility operation will be within specified limits, or (3) assure that the limiting conditions for operation will be met. In

addition, the proposed change is consistent with the guidance provided in GL 93-05. Therefore, the staff finds the proposed change acceptable.

Also, as required by GL 93-05, CP&L reviewed the proposed changes and concluded that the changes are compatible with plant operating experience. CP&L revised TS Bases 3/4.5.1 to reflect the proposed changes described above.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the State of North Carolina official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes the Surveillance Requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (62 FR 17226). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: S. Flanders

Date: December 31, 1998