

April 14, 1998

Mr. W. R. Robinson, Vice President
Shearon Harris Nuclear Power Plant
Carolina Power & Light Company
Post Office Box 165, Mail Code: Zone 1
New Hill, North Carolina 27562-0165

SUBJECT: ISSUANCE OF AMENDMENT NO. 77 TO FACILITY OPERATING LICENSE
NO. NPF-63 REGARDING DELETION OF SHUTDOWN REQUIREMENT FROM
SELECTED SURVEILLANCES FOR SHEARON HARRIS NUCLEAR POWER
PLANT, UNIT 1 (TAC NO. M98271)

Dear Mr. Robinson:

The Nuclear Regulatory Commission has issued Amendment No. 77 to Facility Operating License No. NPF-63 for the Shearon Harris Nuclear Power Plant, Unit 1. This amendment changes the Technical Specifications in response to your request dated March 17, 1997, as supplemented by letter dated April 13, 1998.

The amendment revises Technical Specifications 4.1.2.2.c, 4.5.2.e, 4.6.2.1.c, 4.6.2.2.c, 4.6.3.2, 4.7.1.2.1.b, 4.7.3.b, and 4.7.4.b to delete specific restrictions in the text of the surveillances that the tests must be done while the unit is shut down.

A copy of the related Safety Evaluation is enclosed. Notice of Issuance will be included in the Commission's regular bi-weekly Federal Register notice.

Sincerely,

Original signed by:

Scott Flanders, Project Manager
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-400

Enclosures:

- 1. Amendment No. 77 to NPF-63
- 2. Safety Evaluation

cc w/enclosures:

See next page

cc: Harris Service List

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Shearon Harris Nuclear Power Plant
Unit 1

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AMENDMENT NO. 77 TO FACILITY OPERATING LICENSE NO. NPF-63 - HARRIS, UNIT 1

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

CAROLINA POWER & LIGHT COMPANY, et al.

DOCKET NO. 50-400

SHEARON HARRIS NUCLEAR POWER PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 77
License No. NPF-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Carolina Power & Light Company, (the licensee), dated March 17, 1997, as supplemented by letter dated April 13, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. NPF-63 is hereby amended to read as follows:

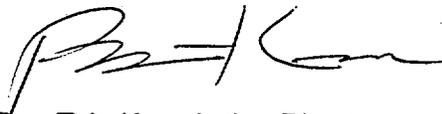
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(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, as revised through Amendment No. 77, are hereby incorporated into this license. Carolina Power & Light Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Pao-Tsin Kuo, Acting Director
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 14, 1998

ATTACHMENT TO LICENSE AMENDMENT NO. 77

FACILITY OPERATING LICENSE NO. NPF-63

DOCKET NO. 50-400

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Remove Pages

3/4 1-8
3/4 5-5
3/4 6-11
3/4 6-12
3/4 6-15
3/4 7-5
3/4 7-11
3/4 7-12

Insert Pages

3/4 1-8
3/4 5-5
3/4 6-11
3/4 6-12
3/4 6-15
3/4 7-5
3/4 7-11
3/4 7-12

REACTIVITY CONTROL SYSTEMS

FLOW PATHS - OPERATING

LIMITING CONDITION FOR OPERATION

3.1.2.2 At least two of the following three boron injection flow paths shall be OPERABLE:

- a. The flow path from the boric acid tank via a boric acid transfer pump and a charging/safety injection pump to the Reactor Coolant System (RCS), and
- b. Two flow paths from the refueling water storage tank via charging/safety injection pumps to the RCS.

APPLICABILITY: MODES 1, 2, and 3.

ACTION:

With only one of the above required boron injection flow paths to the RCS OPERABLE, restore at least two boron injection flow paths to the RCS to OPERABLE status within 72 hours or be in at least HOT STANDBY and borated to a SHUTDOWN MARGIN as specified in the CORE OPERATING LIMITS REPORT (COLR), plant procedure PLP-106 at 200°F within the next 6 hours; restore at least two flow paths to OPERABLE status within the next 7 days or be in HOT SHUTDOWN within the next 6 hours.

SURVEILLANCE REQUIREMENTS

4.1.2.2 At least two of the above required flow paths shall be demonstrated OPERABLE:

- a. At least once per 7 days by verifying that the temperature of the flow path between the boric acid tank and the charging/safety injection pump suction header tank is greater than or equal to 65°F when a flow path from the boric acid tank is used;
- b. At least once per 31 days by verifying that each valve (manual, power-operated, or automatic) in the flow path that is not locked, sealed, or otherwise secured in position, is in its correct position;
- c. At least once per 18 months by verifying that each automatic valve in the flow path actuates to its correct position on a safety injection test signal; and
- d. At least once per 18 months by verifying that the flow path required by Specification 3.1.2.2a. delivers at least 30 gpm to the RCS.

EMERGENCY CORE COOLING SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- d. At least once per 18 months by:
1. Verifying automatic interlock action of the RHR system from the Reactor Coolant System by ensuring that with a simulated or actual Reactor Coolant System pressure signal greater than or equal to 425 psig the interlocks prevent the valves from being opened.
 2. A visual inspection of the containment sump and verifying that the subsystem suction inlets are not restricted by debris and that the sump components (trash racks, screens, etc.) show no evidence of structural distress or abnormal corrosion.
- e. At least once per 18 months by:
1. Verifying that each automatic valve in the flow path actuates to its correct position on safety injection actuation test signal and on safety injection switchover to containment sump from an RWST Lo-Lo level test signal, and
 2. Verifying that each of the following pumps start automatically upon receipt of a safety injection actuation test signal:
 - a) Charging/safety injection pump.
 - b) RHR pump.
- f. By verifying that each of the following pumps develops the required differential pressure when tested pursuant to Specification 4.0.5:
1. Charging/safety injection pump (Refer to Specification 4.1.2.4)
 2. RHR pump \geq 100 psid at a flow rate of at least 3663 gpm.
- g. By verifying that the locking mechanism is in place and locked for the following High Head ECCS throttle valves:
1. Within 4 hours following completion of each valve stroking operation or maintenance on the valve when the ECCS subsystems are required to be OPERABLE, and
 2. At least once per 18 months.

CONTAINMENT SYSTEMS

3/4.6.2 DEPRESSURIZATION AND COOLING SYSTEMS

CONTAINMENT SPRAY SYSTEM

LIMITING CONDITION FOR OPERATION

3.6.2.1 Two independent Containment Spray Systems shall be OPERABLE with each Spray System capable of taking suction from the RWST and transferring suction to the containment sump.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTION:

With one Containment Spray System inoperable, restore the inoperable Spray System to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours; restore the inoperable Spray System to OPERABLE status within the next 48 hours or be in COLD SHUTDOWN within the following 30 hours. Refer also to Specification 3.6.2.3 Action.

SURVEILLANCE REQUIREMENTS

4.6.2.1 Each Containment Spray System shall be demonstrated OPERABLE:

- a. At least once per 31 days by verifying that each valve (manual, power-operated, or automatic) in the flow path that is not locked, sealed, or otherwise secured in position, is in its correct position;
- b. By verifying that, on an indicated recirculation flow of at least 1832 gpm, each pump develops a differential pressure of greater than or equal to 186 psi when tested pursuant to Specification 4.0.5;
- c. At least once per 18 months by:
 1. Verifying that each automatic valve in the flow path actuates to its correct position on a containment spray actuation test signal and
 2. Verifying that each spray pump starts automatically on a containment spray actuation test signal.
 3. Verifying that, coincident with an indication of containment spray pump running, each automatic valve from the sump and RWST actuates to its appropriate position following an RWST Lo-Lo test signal.
- d. At least once per 10 years by performing an air or smoke flow test through each spray header and verifying each spray nozzle is unobstructed.

CONTAINMENT SYSTEMS

SPRAY ADDITIVE SYSTEM

LIMITING CONDITION FOR OPERATION

3.6.2.2 The Spray Additive System shall be OPERABLE with:

- a. A Spray Additive Tank containing between 28 and 30 weight % NaOH and a contained volume of between 3268 and 3964 gallons which will be ensured by maintaining an indicated level between 92% and 96%, and
- b. Two spray additive eductors each capable of adding NaOH solution from the chemical additive tank to a Containment Spray System pump flow.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTION:

With the Spray Additive System inoperable, restore the system to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours; restore the Spray Additive System to OPERABLE status within the next 48 hours or be in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.2.2 The Spray Additive System shall be demonstrated OPERABLE:

- a. At least once per 31 days by verifying that each valve (manual, power-operated, or automatic) in the flow path that is not locked, sealed, or otherwise secured in position, is in its correct position;
- b. At least once per 6 months by:
 1. Verifying the contained solution volume in the tank, and
 2. Verifying the concentration of the NaOH solution by chemical analysis.
- c. At least once per 18 months by verifying that each automatic valve in the flow path actuates to its correct position on a containment spray or containment isolation phase A test signal as applicable; and
- d. At least once per 5 years by verifying each eductor flow rate is between 19.5 and 20.5 gpm, using the RWST as the test source containing at least 436,000 gallons of water.

CONTAINMENT SYSTEMS

CONTAINMENT ISOLATION VALVES

SURVEILLANCE REQUIREMENTS (Continued)

4.6.3.2 Each isolation valve shall be demonstrated OPERABLE at least once per 18 months by:

- a. Verifying that on a Phase "A" Isolation test signal, each Phase "A" isolation valve actuates to its isolation position;
- b. Verifying that on a Phase "B" Isolation test signal, each Phase "B" isolation valve actuates to its isolation position; and
- c. Verifying that on a Containment Ventilation Isolation test signal, each normal, preentry purge makeup and exhaust, and containment vacuum relief valve actuates to its isolation position, and
- d. Verifying that, on a Safety Injection "S" test signal, each containment isolation valve receiving an "S" signal actuates to its isolation position, and
- e. Verifying that, on a Main Steam Isolation test signal, each main steam isolation valve actuates to its isolation position, and
- f. Verifying that, on a Main Feedwater Isolation test signal, each feedwater isolation valve actuates to its isolation position.

4.6.3.3 The isolation time of each power-operated or automatic valve shall be determined to be within its limit specified in the Technical Specification Equipment List Program, plant procedure PLP-106, when tested pursuant to Specification 4.0.5.

PLANT SYSTEMS

AUXILIARY FEEDWATER SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

2. Demonstrating that the steam turbine - driven pump satisfies performance requirements by either:

NOTE: The provisions of Specification 4.0.4 are not applicable for entry into MODE 3.

- a) Verifying the pump develops a differential pressure that (when temperature - compensated to 70°F) is greater than or equal to 1433 psid at a recirculation flow of greater than or equal to 90 gpm (45 KPPH) when the secondary steam supply pressure is greater than 210 psig, or
 - b) Verifying the pump develops a differential pressure that (when temperature - compensated to 70°F) is greater than or equal to 1400 psid at a flow rate of greater than or equal to 430 gpm (215 KPPH) when the secondary steam supply pressure is greater than 280 psig.
3. Verifying by flow or position check that each valve (manual, power operated, or automatic) in the flow path that is not locked, sealed, or otherwise secured in position is in its correct position; and
 4. Verifying that the isolation valves in the suction line from the CST are locked open.
- b. At least once per 18 months by:
1. Verifying that each motor-driven auxiliary feedwater pump starts automatically, as designed, upon receipt of a test signal and that the respective pressure control valve for each motor-driven pump and each flow control valve with an auto-open feature respond as required;
 2. Verifying that the turbine-driven auxiliary feedwater pump starts automatically, as designed, upon receipt of a test signal. The provisions of Specification 4.0.4 are not applicable for entry into MODE 3; and
 3. Verifying that the motor-operated auxiliary feedwater isolation valves and flow control valves close as required upon receipt of an appropriate test signal for steamline differential pressure high coincident with main steam isolation.

PLANT SYSTEMS

3/4.7.3 COMPONENT COOLING WATER SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.3 At least two component cooling water (CCW) pumps*, heat exchangers and essential flow paths shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTION:

With only one component cooling water flow path OPERABLE, restore at least two flow paths to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.7.3 At least two component cooling water flow paths shall be demonstrated OPERABLE:

- a. At least once per 31 days by verifying that each valve (manual, power-operated, or automatic) servicing safety-related equipment that is not locked, sealed, or otherwise secured in position is in its correct position; and
- b. At least once per 18 months by verifying that:
 1. Each automatic valve servicing safety-related equipment or isolating non-safety-related components actuates to its correct position on a Safety Injection test signal, and
 2. Each Component Cooling Water System pump required to be OPERABLE starts automatically on a Safety Injection test signal.
 3. Each automatic valve serving the gross failed fuel detector actuates to its correct position on a Low Surge Tank Level test signal.

*The breaker for CCW pump 1C-SAB shall not be racked into either power source (SA or SB) unless the breaker from the applicable CCW pump (1A-SA or 1B-SB) is racked out.

PLANT SYSTEMS

3/4.7.4 EMERGENCY SERVICE WATER SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.4 At least two independent emergency service water loops shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTION:

With only one emergency service water loop OPERABLE, restore at least two loops to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.7.4 At least two emergency service water loops shall be demonstrated OPERABLE:

- a. At least once per 31 days by verifying that each valve (manual, power-operated, or automatic) servicing safety-related equipment that is not locked, sealed, or otherwise secured in position is in its correct position; and
- b. At least once per 18 months by verifying that:
 1. Each automatic valve servicing safety-related equipment or isolating non-safety portions of the system actuates to its correct position on a Safety Injection test signal, and
 2. Each emergency service water pump and each emergency service water booster pump starts automatically on a Safety Injection test signal.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 77 TO FACILITY OPERATING LICENSE NO. NPF-63

CAROLINA POWER & LIGHT COMPANY

SHEARON HARRIS NUCLEAR POWER PLANT, UNIT 1

DOCKET NO. 50-400

1.0 INTRODUCTION

By letter dated March 17, 1997, as supplemented by letter dated April 13, 1998, the Carolina Power & Light Company (CP&L or the licensee) submitted a request for changes to the Shearon Harris Nuclear Power Plant, Unit 1 (SHNPP), Technical Specifications (TS). The requested changes would revise TS surveillances 4.1.2.2.c, 4.5.2.e, 4.6.2.1.c, 4.6.2.2.c, 4.6.3.2, 4.7.1.2.1.b, 4.7.3.b, and 4.7.4.b to delete specific provisions in the text of the surveillances that the tests must be done while the unit is shut down. The current wording for these TS surveillances requires that all testing associated with the surveillance be performed during shutdown. Given such constraints, SHNPP should not take credit for equivalent tests on some components that are currently performed at power to comply with other surveillances. The proposed amendment would eliminate the shutdown requirement and allow SHNPP the flexibility to take credit for equivalent tests on some components that are currently being performed at power in accordance with the TS.

2.0 EVALUATION

The existing TS surveillances 4.1.2.2.c, 4.5.2.e, 4.6.2.1.c, 4.6.2.2.c, 4.6.3.2, 4.7.1.2.1.b, 4.7.3.b, and 4.7.4.b all require a specified testing frequency of 18 months and an additional modifier that the testing be done while the unit is shut down. These surveillances involve the Reactivity Control system, the Emergency Core Cooling system, Containment Spray, Containment Isolation valves, Auxiliary Feedwater system, Component Cooling Water system, and the Emergency Service Water system. The licensee states that complying with the requirement to perform these surveillances at shutdown often leads to unnecessary test repetition during refueling outages.

Deleting the requirement to perform these surveillances at shutdown would allow SHNPP to take credit for equivalent tests which are currently being performed at power to satisfy other TS surveillances. The proposed change does not specifically authorize the performance of any tests at power; however, it does allow SHNPP to evaluate if the plant conditions will allow safe testing of the components and perform the tests in accordance with those evaluations.

The Standard Technical Specifications (STS) for Westinghouse Plants, NUREG-1431, Revision 1, have surveillance requirements equivalent to seven of the eight TS surveillances subject to this amendment, and all of the STS surveillances have 18-month unrestricted frequencies. The STS Bases state that the 18-month frequency is based on the need for the surveillance to be performed under conditions that apply during a unit outage and the potential for unplanned plant transients if the surveillance were performed with the reactor at power. However, the Bases do not preclude taking credit for testing already performed for specific components, or from evaluating the specific plant conditions required for each test. The remaining TS surveillance, 4.1.2.2.c, is not included in the STS. The STS allows this surveillance to be relocated to a licensee controlled document. By relocating this surveillance to a controlled document, the licensee is permitted to change the surveillance pursuant to the requirements of 10 CFR 50.59. Therefore, removing the shutdown condition from the requested TS surveillances will not have a detrimental effect on the surveillance program.

In addition, the staff concluded in Generic Letter 91-04, "Changes in Technical Specification Surveillance Intervals To Accommodate A 24-Month Fuel Cycle," that the TS do not need to restrict surveillances to only being performed during shutdown. However, when refueling interval surveillances are performed during power operation, licensees should give proper regard for their effect on the safe operation of the plant. If the required surveillance tests during plant operation would adversely affect safety, the licensee should postpone the surveillance until the unit is shut down for refueling or is in a condition or mode that is consistent with the safe conduct of that surveillance. SHNPP will continue to determine the necessary plant conditions to safely perform the required surveillance tests on the affected components.

Based on the above evaluation, the staff finds that the proposed TS change for the requested TS surveillances is acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the State of North Carolina official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes the Surveillance Requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (62 FR 19826). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: T. Eaton

Date: April 14, 1998