

August 7, 1996

Mr. W. R. Robinson, Vice President
Shearon Harris Nuclear Power Plant
Carolina Power & Light Company
Post Office Box 165, Mail Code: Zone 1
New Hill, North Carolina 27562-0165

SUBJECT: ISSUANCE OF AMENDMENT NO. 66 TO FACILITY OPERATING LICENSE
NO. NPF-63 REGARDING THE LOCATION OF A TRIAXIAL PEAK ACCELEROGRAPH
RECORDER - SHEARON HARRIS NUCLEAR POWER PLANT, UNIT 1 (TAC M95667)

Dear Mr. Robinson:

The Nuclear Regulatory Commission has issued Amendment No. 66 to Facility Operating License No. NPF-63 for the Shearon Harris Nuclear Power Plant, Unit 1. This amendment changes the Technical Specifications in response to your request dated May 31, 1996.

The amendment changes Technical Specifications (TS) Table 3.3-7, Seismic Monitoring Instrumentation, and TS Table 4.3-4, Seismic Monitoring Instrumentation Surveillance Requirements, to correct the location described for one of the three Triaxial Peak Accelerograph recorders.

A copy of the related Safety Evaluation is enclosed. Notice of Issuance will be included in the Commission's regular bi-weekly Federal Register notice.

Sincerely,
Original signed by:
Ngoc B. Le, Project Manager
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-400

Enclosures:

- 1. Amendment No. 66 to NPF-63
- 2. Safety Evaluation

cc w/enclosures:
See next page

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OFFICE	LA:PDII-1	PM:PDII-1	ECGE	OGC	D:PDII-1
NAME	Dunnington	NLe	GBagchi	R. Boehmann	EImbro
DATE	7/10/96	7/10/96	7/11/96	7/15/96	8/17/96
COPY	Yes/No	Yes/No	Yes/No	Yes/No	Yes/No

*Rec'd PDII-1
8/1/96
from OGC*

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AMENDMENT NO. 66 TO FACILITY OPERATING LICENSE NO. NPF-63 - HARRIS, UNIT 1

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Shearon Harris Nuclear Power Plant
Unit 1

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

CAROLINA POWER & LIGHT COMPANY, et al.

DOCKET NO. 50-400

SHEARON HARRIS NUCLEAR POWER PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 66
License No. NPF-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Carolina Power & Light Company, (the licensee), dated May 31, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. NPF-63 is hereby amended to read as follows:

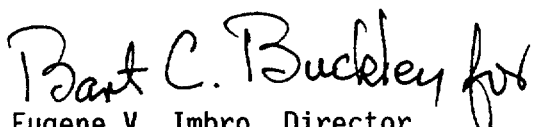
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(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, as revised through Amendment No. 66, are hereby incorporated into this license. Carolina Power & Light Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Eugene V. Imbro, Director
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 7, 1996

ATTACHMENT TO LICENSE AMENDMENT NO. 66

FACILITY OPERATING LICENSE NO. NPF-63

DOCKET NO. 50-400

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Remove Pages

3/4 3-58

3/4 3-59

Insert Pages

3/4 3-58

3/4 3-59

TABLE 3.3-7

SEISMIC MONITORING INSTRUMENTATION

<u>INSTRUMENTS AND SENSOR LOCATIONS</u>	<u>MEASUREMENT RANGE</u>	<u>MINIMUM INSTRUMENTS OPERABLE</u>
1. Triaxial Time-History Accelerographs		
a. Containment Mat (E1 221 ft)	0.01-1.0 g	1**
b. Containment (E1 286 ft)	0.01-1.0 g	1**
c. Diesel Fuel Oil Storage Tank Building (E1 242 ft)	0.01-1.0 g	1**
2. Triaxial Peak Accelerograph Recorders		
a. Reactor Coolant Pipe (Loop B)	± 10 g	1
b. Steam Generator 1A Pedestal (E1 238 ft)	± 2 g	1
c. Reactor Auxiliary Building (E1 236 ft)	± 10 g	1
3. Triaxial Seismic Switches		
a. Starter Unit for Time History Accelerograph System--Containment Mat (E1 221 ft)	0.005 - 0.05	1*
b. Triaxial Seismic Switch--Containment Mat (E1 221 ft)	0.025 - 0.25	1*
4. Triaxial Response-Spectrum Recorders		
a. Steam Generator 1B Pedestal (E1 238 ft)	± 2 g	1
b. Reactor Auxiliary Building (E1 216 ft)	± 2 g	1
c. Diesel Fuel Oil Storage Tank Building (E1 242 ft)	± 2 g	1
d. Containment Building (E1 221 ft)	± 2 g	1*

*With main control room indication.

**With main control room recording.

TABLE 4.3-4

SEISMIC MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENTS AND SENSOR LOCATIONS</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>ANALOG CHANNEL OPERATIONAL TEST</u>
1. Triaxial Time-History Accelerographs			
a. Containment Mat (E1 221 ft)	M*	R	SA***
b. Containment (E1 286 ft)	M*	R	SA***
c. Diesel Fuel Oil Storage Tank Building (E1 242 ft)	M*	R	SA***
2. Triaxial Peak Accelerograph Recorders			
a. Reactor Coolant Pipe (Loop B)	N.A.	R	N.A.
b. Steam Generator 1A Pedestal (E1 238 ft)	N.A.	R	N.A.
c. Reactor Auxiliary Building (E1 236 ft)	N.A.	R	N.A.
3. Triaxial Seismic Switches			
a. Starter Unit for Time History Accelerograph System--Containment Mat (E1 221 ft)**	M	R	SA***
b. Triaxial Seismic Switch--Containment Mat (E1 221 ft)**	M	R	SA***
4. Triaxial Response-Spectrum Recorders			
a. Containment Building (Active) (E1 221 ft)**	M	R	SA***
b. Steam Generator (Passive) 1B Pedestal	N.A.	R	N.A.
c. Reactor Auxiliary Building (Passive) (E1 216 ft)	N.A.	R	N.A.
d. Diesel Fuel Oil Storage Tank Building (Passive) (E1 242 ft)	N.A.	R	N.A.

*Except seismic starter unit.

**With main control room alarms.

***The bistable trip setpoint need not be determined during the performance of a channel operational test.



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 66 TO FACILITY OPERATING LICENSE NO. NPF-63
CAROLINA POWER & LIGHT COMPANY
SHEARON HARRIS NUCLEAR POWER PLANT, UNIT 1
DOCKET NO. 50-400

1.0 INTRODUCTION

By letter dated May 31, 1996, the Carolina Power & Light Company (the licensee) submitted a request for changes to the Shearon Harris Nuclear Power Plant, Unit 1, (SHNPP) Technical Specifications (TS). The requested change revises TS Table 3.3-7, Seismic Monitoring Instrumentation, and TS Table 4.3-4, Seismic Monitoring Instrumentation Surveillance Requirements, to correct the location described for one of the triaxial peak accelerograph (TPA) recorders.

2.0 EVALUATION

Section 3.7.4 of the SHNPP Final Safety Analysis Report describes the seismic monitoring instrumentation system installed to meet the requirements of Title 10 Code of Federal Regulations, Part 100, Appendix A, "Seismic and Geologic Siting Criteria for Nuclear Power Plants." The instrumentation system follows the guidance contained in Regulatory Guide (RG) 1.12 and includes a TPA mounted on the piping of a reactor coolant system. This conforms with the RG position that a plant have a TPA on the reactor piping. SHNPP TS Table 3.3-7 and Table 4.3-4 describe the locations of the TPA. The location of one of the TPAs is described in the TS as "Reactor Coolant Pipe (Loop 1);" however, the TPA is actually installed on reactor coolant piping connected to Loop 2 (Loop B). An early plant design placed the TPA on Loop 1. However, this was later revised and the TPA was installed on Loop 2. The current plant drawings show the correct location, but the TS was not changed.

The staff finds the proposed TS change acceptable based on the following reasons:

There is no physical alteration to the plant; hence, the change is only administrative.

The change does not involve an increase in the probability or consequences of an accident previously evaluated; hence, the TPA cannot initiate an accident or mitigate accident consequences.

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The possibility of a new accident or different kind of accident from those previously evaluated is not created by the change; hence, no physical alterations to plant equipment and no changes that alter how any safety related system performs its function are being made.

The change will not alter the quantity or quality of the data available following an earthquake; hence, the change is administrative and does not affect the number of instruments or where they are actually located.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the State of North Carolina official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (61 FR 34888). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: R. L. Rothman

Date: August 7, 1996