

March 22, 1995

Mr. W. R. Robinson, Vice President
Shearon Harris Nuclear Power Plant
Carolina Power & Light Company
Post Office Box 165, Mail Code: Zone 1
New Hill, North Carolina 27562-0165

SUBJECT: ISSUANCE OF AMENDMENT NO. 55 TO FACILITY OPERATING LICENSE
NO. NPF-63 REGARDING RELOCATION OF TECHNICAL SPECIFICATIONS IN
ACCORDANCE WITH NUREG-1431 - SHEARON HARRIS NUCLEAR POWER PLANT,
UNIT 1 (TAC NO. M90767)

Dear Mr. Robinson:

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 55 to Facility Operating License No. NPF-63 for the Shearon Harris Nuclear Power Plant, Unit 1. This amendment changes the Technical Specifications in response to your request dated October 24, 1994, as supplemented December 6, 1994.

The amendment revises TS 3/4.3.3.4, Turbine Overspeed Protection, and the associated bases to be consistent with the new Standard Technical Specifications for Westinghouse plants.

All other changes proposed in the October 24, and December 6, 1994, letters are under review.

A copy of the related Safety Evaluation is enclosed. Notice of Issuance will be included in the Commission's regular bi-weekly Federal Register notice.

Sincerely,

(Original Signed By)

Ngoc B. Le, Project Manager
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-400

Enclosures:

1. Amendment No. 55 to NPF-63
2. Safety Evaluation

cc w/enclosures:
See next page

DOCUMENT NAME: G:\HARRIS\HAR90767.AMD

OFFICE	LA:PD II-1	PM:PDII-1	PD:PDII-1	OGC	SPLB/BC
NAME	PAnderson	NLe	WBateman	JH	EMcCracken
DATE	03/9/95	03/9/95	03/22/95	03/15/95	03/10/95 #40 SEA
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Shearon Harris Nuclear Power Plant
Unit 1

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AMENDMENT NO. 55 TO FACILITY OPERATING LICENSE NO. NPF-63 - HARRIS, UNIT 1

Docket File
PUBLIC
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C. Grimes (11E22)
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

CAROLINA POWER & LIGHT COMPANY, et al.

DOCKET NO. 50-400

SHEARON HARRIS NUCLEAR POWER PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 55
License No. NPF-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Carolina Power & Light Company, (the licensee), dated October 24, 1994, as supplemented December 6, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. NPF-63 is hereby amended to read as follows:

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(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, as revised through Amendment No. 55, are hereby incorporated into this license. Carolina Power & Light Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 90 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



FOR William H. Bateman, Director
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance:

ATTACHMENT TO LICENSE AMENDMENT NO. 55

FACILITY OPERATING LICENSE NO. NPF-63

DOCKET NO. 50-400

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

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INSTRUMENTATION

3/4.3.4 TURBINE OVERSPEED PROTECTION - DELETED

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INSTRUMENTATION

BASES

3/4.3.3.11 RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION

The radioactive gaseous effluent instrumentation is provided to monitor and control, as applicable, the releases of radioactive materials in gaseous effluents during actual or potential releases of gaseous effluents. The Alarm/Trip Setpoints for these instruments shall be calculated and adjusted in accordance with the methodology and parameters in the ODCM to ensure that the alarm/trip will occur prior to exceeding the limits of 10 CFR Part 20. This instrumentation also includes provisions for monitoring (and controlling) the concentrations of potentially explosive gas mixtures in the GASEOUS RADWASTE TREATMENT SYSTEM. The OPERABILITY and use of this instrumentation is consistent with the requirements of General Design Criteria 60, 63, and 64 of Appendix A to 10 CFR Part 50. The sensitivity of any noble gas activity monitors used to show compliance with the gaseous effluent release requirements of Specification 3.11.2.2 shall be such that concentrations as low as $1 \times 10^{-6} \mu\text{Ci/ml}$ are measurable.

3/4.3.4 DELETED



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 55 TO FACILITY OPERATING LICENSE NO. NPF-63
CAROLINA POWER & LIGHT COMPANY
SHEARON HARRIS NUCLEAR POWER PLANT, UNIT 1
DOCKET NO. 50-400

1.0 INTRODUCTION

By letter dated October 24, 1994, as supplemented December 6, 1994, Carolina Power & Light Company (the licensee or CP&L) submitted a request for changes to the Shearon Harris Nuclear Power Plant, Unit 1 (SHNPP) Technical Specifications (TS). The proposed amendment would revise the TS to allow the relocation of TS 3/4.3.3.4, Turbine Over Speed Protection; 3/4.3.7.12, Area Temperature Monitoring; and 3/4.11.2.6, Gas Storage Tanks; and the associated Bases in the TS to licensee-controlled documents. Subsequent to its October 24, 1994, submittal, the licensee requested that the staff consider the request for the relocation of TS 3/4.3.3.4 first because this change would provide the licensee with the flexibility to implement the manufacturer's recommendations for turbine steam valve surveillance test requirements. Consequently, this Safety Evaluation covers the staff's review of the licensee's proposed changes to TS 3/4.3.3.4. All other changes proposed in the October 24, 1994, and December 6, 1994, submittals are still under staff review. The changes to TS 3/4.3.3.4 are consistent with NUREG-1431, the new Standard Technical Specifications for Westinghouse plants, and with the Commission's Final Policy Statement for relocation of current Technical Specifications. The December 6, 1994, letter revises Index page x in the TS to reflect the changes in the previously issued Amendment No. 51, as well as the changes proposed in the original CP&L letter dated October 24, 1994. The revised Index page x did not change either the scope of the October 24, 1994 letter, or the initial no significant hazards consideration determination.

Section 182a of the Atomic Energy Act (the "Act") requires that applicants for nuclear power plant operating licenses state TS and that these TS be included as a part of the license. The Commission's regulatory requirements related to the content of TS are set forth in 10 CFR 50.36. That regulation requires that the TS include items in five specific categories including: (1) safety limits, limiting safety system settings and limiting control settings; (2) limiting conditions for operation; (3) surveillance requirements; (4) design features; and (5) administrative controls, and it also states that the Commission may include such additional TS as it finds to be appropriate.

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However, the regulation does not specify the particular TS to be included in a plant's license.

The Commission has provided guidance for the contents of TS in its "Final Policy Statement on Technical Specifications Improvements for Nuclear Power Reactors" (Final Policy Statement), issued on July 22, 1993 (58 FR 39132), in which the Commission indicated that compliance with the Final Policy Statement satisfies Section 182a of the Act. In particular, the Commission indicated that certain items could be relocated from the TS to licensee-controlled documents, and consistent with this approach, the Final Policy Statement identified four criteria to be used in determining whether a particular matter is required to be included in the TS, as follows: (1) installed instrumentation that is used to detect and indicate in the control room a significant abnormal degradation of the reactor coolant pressure boundary; (2) a process variable, design feature, or operating restriction that is an initial condition of a design basis accident or transient analysis that either assumes the failure of, or presents a challenge to, the integrity of a fission product barrier; (3) a structure, system, or component that is part of the primary success path and which functions or actuates to mitigate a design basis accident or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier; (4) a structure, system, or component which operating experience or probabilistic safety assessment has shown to be significant to public health and safety.¹ As a result, the existing Limiting Condition for Operation (LCO) requirements that fall within or satisfy any of the criteria in the Final Policy Statement must be retained in the TS, while those LCO requirements which do not fall within or satisfy these criteria may be relocated to other appropriate licensee-controlled documents.

2.0 EVALUATION

2.1 TS 3/4.3.3.4. Turbine Overspeed Protection

The SHNPP turbine generator has several sets of steam valves to control turbine speed during normal operation and to protect it from overspeed during abnormal operations. These valves are the four high pressure turbine throttle valves, the four high pressure turbine governor valves, and the four low pressure reheat throttle (stop) and four low pressure reheat interceptor valves all of which are controlled during normal operation by the turbine digital-electric hydraulic control (DEH) system. The turbine throttle, governor, reheat, and reheat interceptor valves protect the turbine from exceeding set speeds and protect the reactor system from abnormal surge.

The turbine overspeed protection system consists of redundant mechanical and electrical trip mechanisms each capable of independently initiating fast closure of the turbine steam valves during abnormal conditions. The mechanical overspeed trip includes a spring-loaded weight mounted in a transverse hole of the turbine shaft and will actuate to trip the turbine and

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The Commission recently promulgated a proposed change to 10 CFR 50.36, pursuant to which the rule would be amended to codify and incorporate these criteria (59 FR 48180, September 20, 1994). The Commission's Final Policy Statement specified that the Reactor Core Isolation Cooling, Isolation Condenser, Residual Heat Removal, Standby Liquid Control, and Recirculation Pump Trip are included in the TS under Criterion 4 (58 FR 39132, July 22, 1993).

initiate fast closure of the turbine steam valves at 111 percent of rated speed. The DEH electrical overspeed trip device consists of magnetic pickups mounted at the turbine gear spacer and shaft driven oil pump provide an additional overspeed trip and additional overspeed protection because it will actuate when the turbine speed exceeds the setpoint of approximately 110 percent (1980 rpm) of the rated speed to shut all turbine inlet valves.

The TS (3/4.3.3.4) requires at least one turbine overspeed protection system to be operable in modes 1, 2, and 3, and provides surveillance requirements for periodic observation of turbine valve movement, system channel calibration, and inspection of internal parts of the turbine steam valves. The surveillance requirements include monthly cycling of each of the valves through at least one complete cycle. Cycling of the valves introduces the potential for causing plant transients which are detrimental to plant safety.

In its submittal of October 24, 1994, CP&L proposed that TS 3/4.3.3.4 and its associated Bases be deleted. The proposed change would also relocate the surveillance requirements to the appropriate plant administrative controls and would be referenced in the SHNPP Updated Safety Analysis Report. The relocated surveillance requirements would be based on the manufacturer's recommendations and operational experience. Surveillance testing in accordance with the manufacturer's recommendations will permit the licensee to optimize testing and inspection frequencies such that unnecessary testing and inspections will be reduced. Reduction of unnecessary testing and inspections will assist in reducing plant transients and may thereby enhance safety.

To date, the maintenance and test histories of the turbine steam valves have been satisfactory.

The potential effects of turbine missiles were evaluated in Section 3.5.1.3 of NUREG-1038, "Safety Evaluation Report Related to the Operation of the Shearon Harris Nuclear Power Plant, Unit 1," Supplement No. 4, dated October 1986. In that evaluation, the NRC staff determined that CP&L should develop and submit to the NRC a turbine system maintenance program based upon manufacturer's recommendations. By letter dated May 15, 1986, CP&L provided information regarding its turbine system maintenance program and provides a commitment to follow an inspection program that is based on the recommendations of Westinghouse (the manufacturer) for low pressure turbine rotor inspection intervals and procedural guidelines. The staff also had previously accepted the Westinghouse procedure for estimating turbine failure probabilities. This procedure is used to determine inspection intervals. The NUREG-1038 evaluation also concluded that with the implementation of this turbine system maintenance program, the overall probability for unacceptable damage by turbine missiles for SHNPP is considered acceptable.

The NRC staff also notes that the proposed deletion of TS 3/4.3.3.4 would make the SHNPP TS consistent with the guidance provided in the NRC's Standard Technical Specifications, Westinghouse Plants (NUREG-1431) in that the NRC's Standard Technical Specifications do not include TS requiring the operability of a turbine overspeed protection system.

The licensee has proposed changes to TS 3/4.3.3.4 to remove the requirements related to the operability of the turbine overspeed controls, and the related surveillance requirements and relocate them to the FSAR update or other

appropriate CP&L controlled documents. In the amendment application, the licensee committed to maintain a current surveillance program for the turbine valves. The staff also noted that the inspection requirement for the turbine valves is included in the Section 10.2.3.6, Inservice Inspection, of the SHNPP FSAR.

The turbine is equipped with control valves and stop valves which control turbine speed during normal plant operation and protect it from overspeed during abnormal conditions. The turbine overspeed protection system consists of separate mechanical and electrical sensing mechanisms that are capable of initiating fast closure of the steam valves. Currently, TS 3/4.3.3.4 requires particular operability and surveillance requirements for these steam control and stop valves to minimize the potential for fragment missiles that might be generated as the result of a turbine overspeed event. The licensee has proposed to relocate these provisions to the plant administrative controls and to reference them in the FSAR update so future changes to the operation and surveillance of the turbine overspeed features could be changed under 10 CFR 50.59.

Although the design basis accidents and transients include a variety of system failures and conditions which might result from turbine missiles striking various plant systems and equipment, system failures and plant conditions could be caused by other events as well as turbine failures. In view of the low likelihood of turbine missiles, this scenario does not constitute a part of the primary success path to prevent or mitigate such design basis accidents and transients. Similarly, the turbine overspeed control is not part of an initial condition of a design basis accident or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier.

Probabilistic safety assessments (PRA) and operating experience have demonstrated that proper maintenance of the turbine overspeed control valves is important to minimize the potential for overspeed events and turbine damage. However, that experience has also demonstrated that there is a low likelihood of significant risk to public health and safety because of turbine overspeed events. Further, the licensee has committed to maintain the same surveillance and inspection requirement for the turbine valves in their plant administrative procedures and referenced in their FSAR update. Furthermore, the licensee's inservice inspection program, which must comply with 10 CFR 50.55(a), and a surveillance program for the turbine control and stop valves derived from the manufacturer's recommendations remain the same.

Accordingly, the staff concluded that the requirements for turbine overspeed controls do not meet the TS criteria in the Final Policy Statement. The limiting conditions for operation and surveillance requirements for turbine overspeed controls were removed from the standard technical specifications.

On this basis, the staff concludes that these requirements are not required to be in the TS under 10 CFR 50.36 or Section 182a of the Atomic Energy Act, and are not required in order to provide adequate protection to the health and safety of the public. Further, they do not fall within any of the four criteria set forth in the Commission's Final Policy Statement, discussed above. In addition, the NRC staff finds that sufficient regulatory controls exist under 10 CFR 50.59 to ensure that future changes to these requirements

are acceptable. Accordingly, the staff has concluded that these requirements may be relocated from the TS to the licensee controlled document or reference in the SHNPP FSAR update.

The NRC staff has no objection to the deletion of the Bases associated with TS 3/4.3.3.4.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the appropriate North Carolina State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (59 FR 60379). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: N. B. Le

Date: March 22, 1995