

April 21, 1995

Mr. W. R. Robinson, Vice President
Shearon Harris Nuclear Power Plant
Carolina Power & Light Company
Post Office Box 165, Mail Code: Zone 1
New Hill, North Carolina 27562-0165

SUBJECT: ISSUANCE OF AMENDMENT NO. 57 TO FACILITY OPERATING LICENSE
NO. NPF-63 REGARDING NUCLEAR ASSESSMENT SECTION - SHEARON HARRIS
NUCLEAR POWER PLANT, UNIT 1 (TAC NO. M90003)

Dear Mr. Robinson:

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 57 to Facility Operating License No. NPF-63 for the Shearon Harris Nuclear Power Plant, Unit 1. This amendment changes the Technical Specifications in response to your request dated July 22, 1994, as supplemented March 6, 1995.

The amendment allows implementation of a performance-based assessment program and the corresponding functional and organizational changes in the Nuclear Assessment Department. The changes affect the independent review function, the independent assessment of plant activity, and the independent safety engineering group. These functions will now be performed by the Nuclear Assessment Section.

A copy of the related Safety Evaluation is enclosed. Notice of Issuance will be included in the Commission's regular bi-weekly Federal Register notice.

Sincerely,

(Original Signed By)

Ngoc B. Le, Project Manager
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-400

Enclosures:

1. Amendment No. 57 to NPF-63
2. Safety Evaluation

cc w/enclosures:

See next page

DOCUMENT NAME: G:\HARRIS\HAR90003.AMD - *See previous concurrence

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Mr. W. R. Robinson
Carolina Power & Light Company

Shearon Harris Nuclear Power Plant
Unit 1

cc:

Mr. R. E. Jones
General Counsel - Legal Department
Carolina Power & Light Company
Post Office Box 1551
Raleigh, North Carolina 27602

Admiral Kinnaird R. McKee
214 South Morris Street
Oxford, Maryland 21654

Resident Inspector/Harris NPS
c/o U.S. Nuclear Regulatory Commission
5421 Shearon Harris Road
New Hill, North Carolina 27562-9998

Mr. Robert D. Martin
3775 Millers Pond Way
Snellville, Georgia 30278

Karen E. Long
Assistant Attorney General
State of North Carolina
Post Office Box 629
Raleigh, North Carolina 27602

Mr. J. W. Donahue
Plant Manager - Harris Plant
Carolina Power & Light Company
Shearon Harris Nuclear Power Plant
Post Office Box 165, MC: Zone 1
New Hill, North Carolina 27562-0165

Public Service Commission
State of South Carolina
Post Office Drawer 11649
Columbia, South Carolina 29211

Mr. Robert P. Gruber
Executive Director
Public Staff NCUC
Post Office Box 29520
Raleigh, North Carolina 27626

Regional Administrator, Region II
U.S. Nuclear Regulatory Commission
101 Marietta St., N.W. Suite 2900
Atlanta, Georgia 30323

Chairman of the North Carolina
Utilities Commission
Post Office Box 29510
Raleigh, North Carolina 27626-0510

Mr. Dayne H. Brown, Director
Division of Radiation Protection
N.C. Department of Environmental
Commerce & Natural Resources
Post Office Box 27687
Raleigh, North Carolina 27611-7687

Mr. H. W. Habermeyer, Jr.
Vice President
Nuclear Services Department
Carolina Power & Light Company
Post Office Box 1551
Raleigh, North Carolina 27602

AMENDMENT NO. 57 TO FACILITY OPERATING LICENSE NO. NPF-63 - HARRIS, UNIT 1

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S. Varga

J. Zwolinski

OGC

G. Hill (2)

C. Grimes (11E22)

S. Black

F. Allenspach

ACRS (4)

OPA

OC/LFMB

E. Mershoff RII

cc: Harris Service List

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

CAROLINA POWER & LIGHT COMPANY, et al.

DOCKET NO. 50-400

SHEARON HARRIS NUCLEAR POWER PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 57
License No. NPF-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Carolina Power & Light Company, (the licensee), dated July 22, 1994, as supplemented March 6, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. NPF-63 is hereby amended to read as follows:

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(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, as revised through Amendment No. 57, are hereby incorporated into this license. Carolina Power & Light Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



David B. Matthews, Director
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 21, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 57

FACILITY OPERATING LICENSE NO. NPF-63

DOCKET NO. 50-400

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

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6.2.4 SHIFT TECHNICAL ADVISOR

6.2.4.1 The Shift Technical Advisor shall provide advisory technical support to the Shift Foreman in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operation of the unit. The Shift Technical Advisor shall have a baccalaureate degree or equivalent in a scientific or engineering discipline and shall have received specific training in the response and analysis of the unit for transients and accidents, and in unit design and layout, including the capabilities of instrumentation and controls in the control room.

6.3 Deleted

Qualified Safety Reviewers (Continued)

These individuals shall have a baccalaureate degree in an engineering or related field or equivalent, and 2 years of related experience. Such designation shall include the disciplines or procedure categories for which each individual is qualified. Qualified individuals or groups not on the plant staff may be relied upon to perform safety reviews if so designated by the Plant General Manager.

6.5.1.4 Safety Evaluations and Approvals

6.5.1.4.1 The safety evaluation prepared in accordance with Specification 6.5.1.1.1 shall include a written determination, with basis, of whether or not the procedures or changes thereto, proposed tests and experiments and changes thereto, and modifications constitute an unreviewed safety question as defined in Paragraph 50.59 of 10 CFR Part 50, or whether they involve a change to the Final Safety Analysis Report, the Technical Specifications, or the Operating License.

6.5.1.4.2 The safety evaluation shall be prepared by a qualified individual. The safety evaluation shall be reviewed by a second qualified individual.

6.5.1.4.3 A safety evaluation and subsequent review that conclude that the subject action may involve an unreviewed safety question, a change to the Technical Specifications, or a change to the Operating License, will be referred to the Plant Nuclear Safety Committee (PNSC) for their review in accordance with Specification 6.5.2.6. If the PNSC recommendation is that an item is an unreviewed safety question, a change to the Technical Specifications, or a change to the Operating License, the action will be referred to the Commission for approval prior to implementation. Implementation may not proceed until after review by the Nuclear Assessment Section (NAS) in accordance with Specification 6.5.3.9.

6.5.1.4.4 If a safety evaluation and subsequent review conclude that the subject action does not involve an unreviewed safety question, a change to the Technical Specification, or a change to the Operating License, the action may be approved by the Plant General Manager or his designee or, as applicable, by the Manager of the primary functional area affected by the action. The individual approving the action shall assure that the reviewers collectively possess the background and qualification in all of the disciplines necessary and important to the specific review for both safety and technical aspects.

6.5.1.4.5 A safety evaluation and subsequent review that conclude that the modifications, procedures, tests or experiments which constitute a change to the facility as described in the Final Safety Analysis Report shall be referred to the Nuclear Assessment Section for review in accordance with Specification 6.5.3.9, but implementation may proceed prior to the completion of that review.

6.5.1.4.6 The individual approving the procedure, tests, or experiment or change thereto shall be other than those who prepared the safety evaluation or performed the safety review.

ADMINISTRATIVE CONTROLS

RESPONSIBILITIES (Continued)

- b. Review of all proposed tests and experiments that affect nuclear safety and that have been initially determined to appear to constitute an unreviewed safety question or involve an unreviewed change to the Technical Specifications;
- c. Review of all proposed changes to Appendix "A" Technical Specifications;
- d. Review of all proposed changes or modifications to unit systems or equipment that affect nuclear safety and that have been initially determined to appear to constitute an unreviewed safety question as defined in 10 CFR 50.59 or involve a change to the Technical Specifications;
- e. Investigation of all violations of the Technical Specifications, including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence, to the Vice President - Harris Nuclear Plant;
- f. Review of all REPORTABLE EVENTS;
- g. Review of unit operations to detect potential hazards to nuclear safety;
- h. Performance of special reviews, investigations, or analyses and reports thereon as requested by the Plant General Manager or the Manager - Nuclear Assessment Section;
- i. Review of the Security Plan;
- j. Review of the Emergency Plan;
- k. Review of any accidental, unplanned, or uncontrolled radioactive release including the preparation of reports covering evaluation, recommendations, and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Vice President - Harris Nuclear Plant;
- l. Review, prior to implementation, of changes to the PROCESS CONTROL PROGRAM, the OFFSITE DOSE CALCULATION MANUAL, the Radwaste Treatment Systems, and the Technical Specification Equipment List Program.

6.5.2.7 The PNSC shall:

- a. Render determinations in writing with regard to whether or not each item considered under Specification 6.5.2.6a. through e. constitutes an unreviewed safety question; and

ADMINISTRATIVE CONTROLS

RESPONSIBILITIES (Continued)

- b. Provide written notification within 24 hours to the Vice President - Harris Nuclear Plant of disagreement between the PNSC and the Plant General Manager. However, the Plant General Manager shall have responsibility for resolution of such disagreements pursuant to Specification 6.1.1.

RECORDS

6.5.2.8 The PNSC shall maintain written minutes of each PNSC meeting that, at a minimum, document the results of all PNSC activities performed under the responsibility provisions of these Technical Specifications. Copies shall be provided to the Vice President - Harris Nuclear Plant and the Manager - Nuclear Assessment Section.

6.5.3 NUCLEAR ASSESSMENT SECTION INDEPENDENT REVIEW PROGRAM

FUNCTION

6.5.3.1 The Nuclear Assessment Section shall function to provide independent review of plant changes, tests, and procedures; verify that REPORTABLE EVENTS are investigated in a timely manner and corrected in a manner that reduces the probability of recurrence of such events; and detect trends that may not be apparent to a day-to-day observer.

ORGANIZATION

6.5.3.2 The individuals assigned responsibility for independent reviews shall be technically qualified in a specified technical discipline or disciplines. These individuals shall collectively have the experience and competence required to review activities in the following areas:

- a. Nuclear power plant operations,
- b. Nuclear engineering,
- c. Chemistry and radiochemistry,
- d. Metallurgy,
- e. Nondestructive testing,
- f. Instrumentation and control,
- g. Radiological safety,
- h. Mechanical and electrical engineering,
- i. Administrative controls,
- j. Seismic and environmental,
- k. Quality assurance practices, and
- l. Other appropriate fields.

ADMINISTRATIVE CONTROLS

ORGANIZATION (Continued)

6.5.3.3 The Manager - Nuclear Assessment Section shall have a bachelor degree in an engineering or related field and, in addition, shall have a minimum of 10 years' related experience, of which a minimum of 5 years shall be in the operation and/or design of nuclear power plants.

6.5.3.4 The individuals performing independent safety reviews shall have a bachelor degree in an engineering or related field or equivalent and, in addition, shall have a minimum of 5 years' related experience.

6.5.3.5 An individual may possess competence in more than one specialty area. If sufficient expertise is not available within the Nuclear Assessment Section, competent individuals from other Carolina Power & Light Company organizations or outside consultants shall be utilized in performing independent reviews and investigations.

6.5.3.6 The documents submitted under 6.5.3.9 shall be reviewed by individuals meeting the requirements of 6.5.3.2 and 6.5.3.4 to ensure disciplines are encompassed. Multiple reviews will be conducted on documents where required to meet applicable disciplines of 6.5.3.2.

6.5.3.7 Independent safety reviews shall be performed by individuals not directly involved with the activity under review or responsible for the activity under review.

6.5.3.8 The Nuclear Assessment Section independent safety review program shall be conducted in accordance with written, approved procedures.

REVIEW

6.5.3.9 The Nuclear Assessment Section shall perform reviews of the following:

- a. Written safety evaluations of changes in the facility as described in the Safety Analysis Report, changes in procedures as described in the Safety Analysis Report, and tests or experiments not described in the Safety Analysis Report which are completed without prior NRC approval under the provisions of 10CFR50.59(a)(1). These reviews are to verify that such changes, tests, or experiments do not involve a change in the Technical Specifications or an unreviewed safety question as defined in 10CFR50.59(a)(2). These reviews may be conducted after appropriate management approval, and implementation may proceed prior to completion of the review.
- b. Proposed changes in procedures required by these Technical Specifications, proposed changes in the facility, or proposed tests or experiments, any of which involve a change in the Technical Specifications or an unreviewed safety question pursuant to 10CFR50.59(a)(2) prior to implementation.
- c. Proposed changes to the Technical Specifications or this Operating License prior to implementation.
- d. Violations, deviations and REPORTABLE EVENTS, which require reporting to the NRC in writing, such as:

ADMINISTRATIVE CONTROLS

REVIEW (Continued)

- 1) Violations of applicable codes, regulations, orders, Technical Specifications, license requirements or internal procedures or instructions having safety significance.
 - 2) Significant operating abnormalities or deviations from normal or expected performance of plant safety-related structures, systems, or components, and
 - 3) REPORTABLE EVENTS, as specified in 10CFR50.73.
- e. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety.
- f. Any other matter involving safe operation of the nuclear power plant that the Manager - Nuclear Assessment Section, deems appropriate for consideration or which is referred to the Manager - Nuclear Assessment Section, by the on-site operating organization, Plant Nuclear Safety Committee (PNSC) or by other functional organizational units within Carolina Power & Light Company.

RECORDS

- 6.5.3.10 Results of Nuclear Assessment Section independent safety reviews shall be documented and retained.

6.5.4 NUCLEAR ASSESSMENT SECTION ASSESSMENT PROGRAM

- 6.5.4.1 Assessments of facility activities shall be performed by the Nuclear Assessment Section. Assessments will be performance based and will be scheduled based on plant performance and importance to safety but at a frequency not to exceed 24 months. These assessments shall encompass:
- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions.
 - b. The performance, training and qualifications of the entire facility staff.
 - c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety.
 - d. The performance of activities required by the QA Program to meet the criteria of Appendix B, 10 CFR Part 50.
 - e. Any other area of facility operation considered appropriate by the Vice President - Harris Nuclear Plant.
 - f. The Radiological Environmental Monitoring Program and the results thereof.
 - g. The OFF-SITE DOSE CALCULATION MANUAL and implementing procedures.

ADMINISTRATIVE CONTROLS

6.5.4 NUCLEAR ASSESSMENT SECTION ASSESSMENT PROGRAM (Continued)

- h. The PROCESS CONTROL PROGRAM and implementing procedures for processing and packaging of radioactive wastes.
- 6.5.4.2 Assessments of activities prescribed by Title 10 of the Code of Federal Regulations will be performed at the frequencies prescribed by the applicable regulation. These assessments shall encompass:
- a. Emergency Preparedness (per 10 CFR 50.54(t))
 - b. Security (per 10 CFR 50.54(p))
 - c. Radiation Protection (per 10 CFR 20.1101(c))
- 6.5.4.3 The NAS shall include at least three individuals to perform the Independent Safety Engineering Group (ISEG) function. Each reviewer shall have a bachelor's degree in engineering or related science or equivalent and at least two years professional level experience in his field, at least one year of which shall be in the nuclear field.
- 6.5.4.3.1 The NAS shall function to examine unit operating characteristics, NRC issuances, industry advisories, Licensee Event Reports, and other sources of unit design information which may indicate areas of improving unit safety. Review responsibilities shall also include plant operations, modifications, maintenance, and surveillances to verify independently that these activities are performed safely and correctly and that human errors are reduced as much as practical.

6.5.5 OUTSIDE AGENCY INSPECTION AND AUDIT PROGRAM

- 6.5.5.1 An independent fire protection and loss prevention inspection and audit shall be performed at least once per 12 months using either qualified offsite licensee personnel or an outside fire protection firm.
- 6.5.5.2 An inspection and audit of the fire protection and loss prevention program shall be performed by an outside qualified fire consultant at intervals no greater than 36 months.
- 6.5.5.3 Copies of the audit reports and responses to them shall be forwarded to the Vice President - Harris Nuclear Plant and the Manager - Nuclear Assessment Section.

(PAGE 6-15 DELETED)

(NEXT PAGE IS 6-16)

ADMINISTRATIVE CONTROLS

6.6 REPORTABLE EVENT ACTION

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and a report submitted pursuant to the requirements of Section 50.73 to 10 CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the PNSC, and the results of this review shall be submitted to the Manager - Nuclear Assessment Section and the Vice President - Harris Nuclear Plant.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within 1 hour. The Vice President - Harris Nuclear Plant shall be notified within 24 hours;
- b. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PNSC. This report shall describe:
(1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems, or structures, and (3) corrective action taken to prevent recurrence;
- c. The Safety Limit Violation Report shall be submitted, within 14 days of the violation, to the Commission, the Manager - Nuclear Assessment Section, and the Vice President - Harris Nuclear Plant; and
- d. Operation of the unit shall not be resumed until authorized by the Commission.

6.8 PROCEDURES AND PROGRAMS

6.8.1 Written procedures shall be established, implemented, and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix A of Regulatory Guide 1.33, Revision 2, February 1978;
- b. The emergency operating procedures required to implement the requirements of NUREG-0737 and Supplement 1 to NUREG-0737 as stated in Generic Letter No. 82-33;
- c. Security Plan implementation;
- d. Emergency Plan implementation;
- e. PROCESS CONTROL PROGRAM implementation;
- f. OFFSITE DOSE CALCULATION MANUAL implementation;

ADMINISTRATIVE CONTROLS

RECORD RETENTION (Continued)

- h. Records of annual physical inventory of all sealed source material of record.

6.10.3 The following records shall be retained for the duration of the unit Operating License:

- a. Records and drawing changes reflecting unit design modifications made to systems and equipment described in the Final Safety Analysis Report;
- b. Records of new and irradiated fuel inventory, fuel transfers, and assembly burnup histories;
- c. Records of radiation exposure for all individuals entering radiation control areas;
- d. Records of gaseous and liquid radioactive material released to the environs;
- e. Records of transient or operational cycles for those unit components identified in Table 5.7-1;
- f. Records of reactor tests and experiments;
- g. Records of training and qualification for current members of the unit staff;
- h. Records of inservice inspections performed pursuant to these Technical Specifications;
- i. Records of quality assurance activities required by the Corporate Quality Assurance Program;
- j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59;
- k. Records of meetings of the PNSC;
- l. Records of the service lives of all hydraulic and mechanical snubbers required by Specification 3.7.8 including the date at which the service life commences and associated installation and maintenance records;
- m. Records of secondary water sampling and water quality;
- n. Records of analyses required by the Radiological Environmental Monitoring Program that would permit evaluation of the accuracy of the analysis at a later date. This should include procedures

ADMINISTRATIVE CONTROLS

RECORD RETENTION (Continued)

- effective at specified times and QA records showing that these procedures were followed; and
- o. Records of facility radiation and contamination surveys.
 - p. Records of independent reviews.

6.11 RADIATION PROTECTION PROGRAM

6.11.1 Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained, and adhered to for all operations involving personnel radiation exposure.

6.12 HIGH RADIATION AREA

6.12.1 Pursuant to Paragraph 20.203(c)(5) of 10 CFR Part 20, in lieu of the "control device" or "alarm signal" required by Paragraph 20.203(c), each high radiation area, as defined in 10 CFR Part 20, in which the intensity of radiation is equal to or less than 1000 mR/h at 45 cm (18 in.) from the radiation source or from any surface which the radiation penetrates shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit (RWP). Individuals qualified in radiation protection procedures (e.g., Health Physics Technicians or personnel continuously escorted by such individuals may be exempt from the RWP issuance requirement during the performance of their assigned duties in high radiation areas with exposure rates equal to or less than 1000 mR/h, provided they are otherwise following plant radiation protection procedures for entry into such high radiation areas. Any individual or group of individuals permitted to enter such areas shall be provided with or accompanied by one or more of the following:

- a. A radiation monitoring device that continuously indicates the radiation dose rate in the area; or
- b. A radiation monitoring device that continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate levels in the area have been established and personnel have been made knowledgeable of them; or
- c. An individual qualified in radiation protection procedures, with a radiation dose rate monitoring device, who is responsible for providing positive control over the activities within the area and who shall perform periodic radiation surveillance at the frequency specified by the Radiation Control Supervisor in the RWP.

6.12.2 In addition to the requirements of Specification 6.12.1, accessible areas with radiation levels greater than 1000 mR/h at 45 cm (18 in.) from the radiation source or from any surface which the radiation penetrates, shall be provided with locked doors to prevent unauthorized entry, and the keys shall be maintained under the administrative control of the shift Foreman on duty



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 57 TO FACILITY OPERATING LICENSE NO. NPF-63

CAROLINA POWER & LIGHT COMPANY

SHEARON HARRIS NUCLEAR POWER PLANT, UNIT 1

DOCKET NO. 50-400

1.0 INTRODUCTION

By letter dated July 22, 1994, as supplemented March 6, 1995, Carolina Power & Light Company (CP&L) requested changes to the Administrative Controls Section of the Technical Specifications (TS) for the Shearon Harris Nuclear Power Plant, Unit 1. These changes supersede the TS changes requested by letter dated September 15, 1993. The March 6, 1995 letter did not change the initial no significant hazards finding and did not expand the scope of the original Federal Register notice. The following is a brief description and an evaluation of the requested changes.

2.0 EVALUATION

a. Section 6.2.3 - Project Assessment Section

CP&L proposes to delete this section and integrate the functions into revised Section 6.5.4 (Nuclear Assessment Section [NAS] Assessment Program). The current requirement for reviewers of this function to have a bachelor degree has been changed to a bachelor degree or equivalent, and the number of personnel has been reduced to three. Specific comments on the revised NAS Program are provided in item e. below.

The NRC staff finds this change acceptable as the Project Assessment Section function has been retained, the review personnel retain acceptable qualification requirements, and the number of individuals performing this function is consistent with the number previously approved, which includes additional management personnel to direct and oversee this function.

b. Section 6.5.1.4 - Safety Evaluations and Approvals

CP&L proposes to change the phrase in subsection 6.5.1.4.5 from "conclude that the subsection involves a change in the Final Safety Analysis Report" to "conclude that the modifications, procedures, tests or experiments which constitute a change in the Final Safety Analysis", and to change the reference to Project Assessment Section from Project Assessment Department.

The NRC staff finds this change acceptable as it is consistent with Section 4.3.4 of ANSI N18.7 - 1976, "Administrative Controls and Quality Assurance for the Operational Phase of Nuclear Power Plants," and the title change is consistent with the proposed revised organization.

c. Section 6.5.2 - Plant Nuclear Safety Committee (PNSC)

CP&L proposes to delete reference to the position of Manager, Nuclear Assessment Department with respect to the distribution of reports and PNSC minutes and from receiving notification of disagreements between the Plant General Manager and the PNSC as the position no longer exists.

The NRC staff finds these proposed changes acceptable as they reflect the revised organization and the reports, PNSC minutes and notifications of disagreements will be forwarded to the Vice President, Harris Nuclear Plant.

d. Section 6.5.3 - Nuclear Assessment Section (NAS) Independent Review Program

CP&L proposes to make numerous changes to this section. These changes include:

1. Modifying the list of discipline areas by adding seismic and environmental areas and deleting from item k the words "associated with the unique characteristics."
2. Changing the term baccalaureate degree to bachelor's degree.
3. Replacing the requirement that at least three individuals shall review each item with the requirement that each item will be reviewed by individuals meeting the appropriate technical requirements of subsections 6.5.3.2 and 6.5.3.4 to ensure disciplines are encompassed and that multiple reviews will be conducted where required to meet applicable disciplines identified in subsection 6.5.3.2.
4. Combining the requirement of subsections 6.5.3.9.a, 6.5.3.9.b, 6.5.3.9.c & 6.5.3.9.g into subsections 6.5.3.9.a & 6.5.3.9.b and replacing the phrase "all procedures and programs required by Specification 6.8 and other procedures that affect nuclear safety and changes thereto" with "changes in procedures required by these Technical Specifications," and the phrase "proposed modifications" is replaced with "proposed changes in the facility."
5. The existing requirement of subsection 6.5.3.9.d is contained in the proposed subsection 6.5.3.9.d and the requirement to conduct an Independent Review "prior to implementation" has been added.
6. The existing requirement of subsection 6.5.3.9.e is now incorporated in the proposed subsections 6.5.3.9.1 & 6.5.3.9.2.
7. The existing requirement of subsection 6.5.3.9.f is now incorporated in the proposed subsection 6.5.3.9.d.3.

8. The existing requirements of subsections 6.5.3.9.h & 6.5.3.9.i are moved to subsections 6.5.3.9.f & 6.5.3.9.e respectively.
9. The existing requirement of subsection 6.5.3.9.j with respect to the Nuclear Safety Section review of PNSC reports and minutes is being deleted.
10. The existing requirements of subsections 6.5.3.10 & 6.5.3.11.a are combined into proposed subsection 6.5.3.10.
11. The existing requirements of subsections 6.5.3.11.b & c are deleted because recommendations and identified adverse condition are addressed as part of the Corrective Action Program (CAP) and subsection 6.5.3.11.c requirements are included in the QA Program.

It should be noted that the new NAS function will report to the Site Vice President, rather than to the offsite position of Executive Vice President, Nuclear Generation.

The staff finds these changes acceptable as they reflect the revised reorganization, are due to reformatting, provide consistency with ANSI N18.7 as appropriate, and continue to provide for independent review of items. In addition, with respect to the new NAS reporting relationship, CP&L has provided a detailed description of the oversight function of the Nuclear Services Department Performance Evaluation Section (PES), which states that "the PES shall provide oversight of each plant's NAS by reviewing NAS assessment reports and by performing an NAS effectiveness assessment at least once every 24 months." The PES is briefly described in Section 17.3.3.2 of the proposed revised QA program and the staff considers that this description for the specific oversight function of assessing the NAS performance is acceptable.

e. Section 6.5.4 - NAS Assessment Program

CP&L proposes to establish a Nuclear Safety Assessment Section that incorporates the current function of the Corporate Quality Assurance Audit Program (Section 6.5.4) except for audit frequency requirements. The assessments will encompass those areas currently required, except that the assessments will be scheduled based on plant performance with a maximum period of 24 months between assessments. In addition, the NAS functions will include the current requirements of Section 6.2.3 Project Assessment Section (see item a. above). A key change is that these functions will report to the Site Vice President rather than to an offsite corporate officer.

The staff finds these changes acceptable as they conform to the Improved Standard Technical Specifications (ISTS) with respect to the integration of assessment activities, the assessment (audit) of important activities are retained, the audit frequency is consistent with the ISTS and ANSI N18.7, and the Project Assessment Section function is maintained.

f. Section 6.5.5 - Outside Agency Inspection and Audit Program

The licensee proposes to change the words "Project" to "Plant" and "Department" to "Section" in subsection 6.5.5.3.

The staff finds this change acceptable as it is consistent with the proposed revised organization.

g. Section 6.6 - Reportable Events and Section 6.7 - Safety Limit Violations

In subsections 6.6.1.b and 6.7.1.a & c, the title of the Manager, Nuclear Assessment Department, will be replaced with the new title Manager, Nuclear Assessment Section, and the requirement that the Manager, Nuclear Assessment Department receive 24 hour notification of Safety Limit Violations will be deleted.

The staff finds these changes acceptable as they reflect the revised organization, and because the Manager, NAS, will receive copies of reports.

h. Section 6.10.3 - Records Retention

CP&L proposes to split the list of records described in subsection k into two separate listings in subsections k and p.

The staff finds this change acceptable as records of independent reviews are retained.

3.0 SUMMARY

The staff has found the changes requested by CP&L letter dated July 22, 1994, as supplemented March 6, 1995, to the Administrative Controls Section of the TS for Shearon Harris acceptable as they meet the appropriate acceptance criteria of Sections 13.4 and 17.3 of NUREG-0800, the Standard Review Plan, and conform to the ISTS.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the State of North Carolina official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

This amendment changes recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: F. Allenspach

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