

November 17, 1992

Docket No. 50-400

DISTRIBUTION

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Mr. R. A. Watson
Senior Vice President
Nuclear Generation
Carolina Power & Light Company
Post Office Box 1551
Raleigh, North Carolina 27602

Dear Mr. Watson:

SUBJECT: ISSUANCE OF AMENDMENT NO. 34 TO FACILITY OPERATING LICENSE NO.
NPF-63 REGARDING CONTAINMENT ISOLATION VALVES - SHEARON HARRIS
NUCLEAR POWER PLANT, UNIT 1 (TAC NO. 81210)

The Nuclear Regulatory Commission (NRC) has issued the enclosed Amendment No. 34 to Facility Operating License NO. NPF-63 for the Shearon Harris Nuclear Power Plant, Unit 1. This amendment consists of changes to the Technical Specifications in response to your request dated July 26, 1991.

The amendment revises Technical Specification (TS) 3/4.6.3, Containment Isolation Valves, by adding a statement to the limiting condition for operation under the action requirements to the effect that the provisions of TS 3.0.4 do not apply. This change is in accordance with the recommendations and guidance of NRC Generic Letter 91-08, "Removal of Component Lists from Technical Specifications," dated May 6, 1991.

A copy of the related Safety Evaluation is enclosed. Notice of Issuance will be included in the Commission's regular bi-weekly Federal Register notice.

Sincerely,

Original signed by:

Ngoc B. Le, Project Manager
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 34 to NPF-63
2. Safety Evaluation

cc w/enclosures:
See next page

* See Previous Concurrence

Office	LA:PD21	PM:PD21	*SPLB:NRR	OGC	D:PD21 WSE
Name	PAnderson	NLe	CMcCracken	Utd	EAdams
Date	9/16/92	9/16/92	9/09/92	9/16/92	10/17/92

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Mr. R. A. Watson
Carolina Power & Light Company

Shearon Harris Nuclear Power Plant,
Unit 1

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

CAROLINA POWER & LIGHT COMPANY, et al.

DOCKET NO. 50-400

SHEARON HARRIS NUCLEAR POWER PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 34
License No. NPF-63

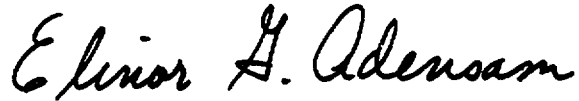
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Carolina Power & Light Company, (the licensee), dated July 26, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. NPF-63 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, as revised through Amendment No. 34, are hereby incorporated into this license. Carolina Power & Light Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Elinor G. Adensam, Director
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 17, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 34

FACILITY OPERATING LICENSE NO. NPF-63

DOCKET NO. 50-400

Replace the following page of the Appendix A Technical Specifications with the enclosed page. The revised areas are indicated by marginal lines.

Remove Pages

3/4 6-14

Insert Pages

3/4 6-14

CONTAINMENT SYSTEMS

3/4.6.3 CONTAINMENT ISOLATION VALVES

LIMITING CONDITION FOR OPERATION

3.6.3 Each containment isolation valve specified in the Technical Specification Equipment List Program, plant procedure PLP-106, shall be OPERABLE with isolation times less than or equal to required isolation times.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTION:

With one or more of the containment isolation valve(s) inoperable, maintain at least one isolation valve OPERABLE in each affected penetration that is open and:

- a. Restore the inoperable valve(s) to OPERABLE status within 4 hours, or
- b. Isolate each affected penetration within 4 hours by use of at least one deactivated automatic valve secured in the isolation position, or
- c. Isolate each affected penetration within 4 hours by use of at least one closed manual valve or blind flange, or
- d. Be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- e. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.6.3.1 Each isolation valve shall be demonstrated OPERABLE prior to returning the valve to service after maintenance, repair or replacement work is performed on the valve or its associated actuator, control or power circuit by performance of a cycling test, and verification of isolation time.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 34 TO FACILITY OPERATING LICENSE NO. NPF-63
CAROLINA POWER & LIGHT COMPANY
SHEARON HARRIS NUCLEAR POWER PLANT, UNIT 1
DOCKET NO. 50-400

1.0 INTRODUCTION

By letter dated July 26, 1991, Carolina Power & Light Company (CP&L or the licensee) submitted a request for a change to the Shearon Harris Nuclear Power Plant, Unit 1 (Harris), Technical Specifications (TS). The requested change would restore the nonapplicability of TS 3.0.4 to TS 3/4.6.3 in accordance with the guidance provided in Generic Letter (GL) 91-08, "Removal of Component Lists from the Technical Specifications," by adding a statement to the Limiting Condition for Operation (LCO) 3.6.3 to state "The provisions of Specification 3.0.4 are not applicable." This exception would apply to all containment isolation valves.

Enclosure 1 to Generic Letter 87-09, "Alternatives to the STS Requirements to Resolve Three Specific Problems with Limiting Conditions for Operation and Surveillance Requirements," states that although TS 3.0.4 may unduly restrict facility operation, nothing in the staff position should be interpreted as endorsing or encouraging a plant startup with inoperable equipment. However, even with the staff guidance and recommendations in GL 87-09, the staff believes there is still room for misinterpretation of the intent of an exception to Specification 3.0.4. Therefore, included in this Safety Evaluation is a discussion of the staff's intent and expectations in granting a TS 3.0.4 exception.

2.0 EVALUATION

Prior to the issuance of the full power operating license for Harris, TS Table 3.6-1, Containment Isolation Valves, and a footnote were removed from the TS and relocated to the Technical Specification Equipment List Program, plant procedures PLP-106. The list of containment isolation valves typically includes footnotes that modify the TS requirements for these valves. As outlined in GL 91-08, these footnotes must be incorporated into the LCO so that they will remain in effect when the table containing the footnotes is removed from the TS. One of these footnotes involves valves that are exempt from the requirements of TS 3.0.4. In the case of the Harris TS, TS 3.0.4 precludes entry into an operational mode or condition when an LCO would not be met without reliance on the provisions of the action requirements. The action requirements for containment isolation valves permit continued operation with

an inoperable valve when the associated penetration is isolated. Therefore, as long as the staff intent and expectations regarding the exceptions from TS 3.0.4. (discussed below) are met, an exception to the limitation of TS 3.0.4 on changes in operational modes or conditions is acceptable for this specification.

In 1987, to implement the guidance provided in GL 87-09, the licensee applied a note to the valves listed in Plant Procedure (PLP)-106, Table 3.6-1, stating that the provisions of TS 3.0.4 were not applicable. The NRC staff subsequently performed a review and determined that a Plant Procedure alone can not state the applicability of the TS requirements. The applicability for these requirements must be contained within the TS. Based on this staff determination, the note was deleted from PLP-106.

To implement the staff guidance provided in GL 91-08, the licensee proposed a statement be added into the LCO for TS 3.6.3 as follows:

The provisions of Specification 3.0.4 are not applicable.

The staff has reviewed the licensee's submittal and has found the proposed change to be acceptable. The exception from requirements of TS 3.0.4 is applicable to all containment isolation valves. This change is acceptable because it is consistent with the guidance provided in GL 91-08 as long as the intent and expectations regarding the exception from TS 3.0.4 (discussed below) are met.

The guidance provided in GL 91-08 and GL 87-09 should not be interpreted to allow containment isolation valves (CIVs) to remain in a degraded condition indefinitely. CIVs serve a significant safety function in that they establish containment integrity. Therefore, there is an urgency in returning these valves to an operable status at the earliest possible opportunity.

In general, the staff believes that nuclear plants should start up from an outage, e.g., a refueling outage, with all CIVs in an OPERABLE condition. But because of extenuating circumstances, such as unavailability of equipment needed for repair or replacement, or the discovery of inoperable CIVs just prior to startup, there will be isolated instances where allowances will be made for restart with inoperable CIVs. However, these isolated instances should coexist with the understanding that the inoperable CIV(s) undergo repair or replacement at the earliest possible opportunity in accordance with 10 CFR Part 50, Appendix B. And the licensee is expected to keep the NRC staff abreast of such isolated instances.

Moreover, in approving the above licensee-proposed change to TS 3/4.6.3, the staff holds the following position which is contained in TS 4.0.4:

Entry into an operational mode or other specified condition shall not be made unless the Surveillance Requirement(s) associated with the Limiting Condition for Operation has been performed within the stated surveillance interval or as otherwise specified.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the State of North Carolina official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (56 FR 43804). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: T. Dunning
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Date: November 17, 1992