

March 2, 1993

Docket No. 50-400

Mr. R. A. Watson
Senior Vice President
Nuclear Generation
Carolina Power & Light Company
Post Office Box 1551
Raleigh, North Carolina 27602

Dear Mr. Watson:

SUBJECT: ISSUANCE OF AMENDMENT NO.35 TO FACILITY OPERATING LICENSE NO.
NPF-63 REGARDING INSTRUMENTATION - SHEARON HARRIS NUCLEAR POWER
PLANT, UNIT 1 (TAC NO. M83151)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 35 to Facility Operating License No. NPF-63 for the Shearon Harris Nuclear Power Plant, Unit 1. This amendment consists of changes to the Technical Specifications in response to your request dated April 2, 1992, as supplemented October 14, 1992, and December 21, 1992.

The amendment revises Instrumentation Technical Specifications (TS) 3.3.2, 3.3.3.1, and 3.3.3.6 by making editorial corrections, clarifying rewording and/or simplifying the presentation of the requirements. The Action Requirements of TS 3.3.3.1 are also being revised for consistency.

A copy of the related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's regular bi-weekly Federal Register notice.

Sincerely,

ORIGINAL SIGNED BY:

Ngoc B. Le, Project Manager
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 35 to NPF-63
2. Safety Evaluation

cc w/enclosures:
See next page

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DATE	02/25/93	03/1/93	02/10/93	02/02/93	11/13/92*

Document Name: HAR83151.AMD

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AMENDMENT NO. 35 TO FACILITY OPERATING LICENSE NO. NPF-63 - HARRIS, UNIT 1

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CAROLINA POWER & LIGHT COMPANY, et al.

DOCKET NO. 50-400

SHEARON HARRIS NUCLEAR POWER PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 35
License No. NPF-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Carolina Power & Light Company, (the licensee), dated April 2, 1992, as supplemented October 14, and December 21, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. NPF-63 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, as revised through Amendment No. are hereby incorporated into this license. Carolina Power & Light Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



for Jocelyn A. Mitchell, Acting Director
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 2, 1993

ATTACHMENT TO LICENSE AMENDMENT NO. 35

FACILITY OPERATING LICENSE NO. NPF-63

DOCKET NO. 50-400

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Remove Pages

3/4 3-21

3/4 3-51

3/4 3-53

3/4 3-66

3/4 3-68

3/4 3-69

3/4 3-70

3/4 3-71

Insert Pages

3/4 3-21

3/4 3-51

3/4 3-53

3/4 3-66

3/4 3-68

3/4 3-69

3/4 3-70

3/4 3-71

TABLE 3.3-3 (Continued)ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
3. Containment Isolation (Continued)					
(1) RCS Leak Detection (normal purge)	1	See Table 3.3-6, Item 1b1, for initiating functions and requirements.			
(2) Preentry Purge Detector	1	See Table 3.3-6, Item 1b2, for initiating functions and requirements.			
c. Airborne Particulate Radioactivity					
(1) RCS Leak Detection (normal purge)	1	See Table 3.3-6, Item 1C1, for initiating functions and requirements.			
(2) Preentry Purge Detector	1	See Table 3.3-6, Item 1C2, for initiating functions and requirements.			
5) Manual Phase "A" Isolation	See Item 3.a.1) above for Manual Phase "A" Isolation initiating functions and requirements.				
4. Main Steam Line Isolation					
a. Manual Initiation					
1) Individual MSIV Closure	1/steam line	1/steam line	1/operating steam line	1, 2, 3, 4	23
2) System	2	1	2	1, 2, 3	22

TABLE 3.3-6

RADIATION MONITORING INSTRUMENTATION FOR PLANT OPERATIONS

<u>INSTRUMENT</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ALARM/TRIP SETPOINT</u>	<u>ACTION</u>
1. Containment Radioactivity--					
a. Containment Ventilation Isolation Signal Area Monitors	2	3	1, 2, 3, 4, 6	#	27
b. Airborne Gaseous Radioactivity					
1) RCS Leakage Detection	1	1	1, 2, 3, 4	$\leq 1.0 \times 10^{-3}$ $\mu\text{Ci/ml}$	26, 27
2) Pre-entry Purge	1	1	##	$\leq 2.0 \times 10^{-3}$ $\mu\text{Ci/ml}$	30
c. Airborne Particulate Radioactivity					
1) RCS Leakage Detection	1	1	1, 2, 3, 4	$\leq 4.0 \times 10^{-8}$ $\mu\text{Ci/ml}$	26, 27
2) Pre-entry Purge	1	1	##	$\leq 1.5 \times 10^{-8}$ $\mu\text{Ci/ml}$	30
2. Spent Fuel Pool Area-- Fuel Handling Building Emergency Exhaust Actuation					
a. Fuel Handling Building Operating Floor--South Network	1/train***	1/train 2 trains	**	≤ 100 mR/hr	28
b. Fuel Handling Building Operating Floor--North Network	1/train***	1/train 2 trains	*	≤ 100 mR/hr	28
3. Control Room Outside Air Intakes--					
a. Normal Outside Air Intake Isolation	1	2	All	$\leq 4.9 \times 10^{-6}$ $\mu\text{Ci/ml}$	29

TABLE 3.3-6 (Continued)

TABLE NOTATIONS

- * With irradiated fuel in the Northend Spent Fuel Pool or transfer of irradiated fuel from or to a spent fuel shipping cask.
- ** With irradiated fuel in the Southend Spent Fuel Pool or New Fuel Pool.
- *** Each channel consists of 3 detectors with 1 of 3 logic. A channel is OPERABLE when 1 or more of the detectors are OPERABLE.
- # For MODES 1, 2, 3 and 4 the setpoint shall be less than or equal to three times detector background at RATED THERMAL POWER. During fuel movement the setpoint shall be less than or equal to 150 mR/hr.
- ## Required OPERABLE whenever pre-entry purge system is to be used.

ACTION STATEMENTS

- ACTION 26 - Must satisfy the ACTION requirement for Specification 3.4.6.1.
- ACTION 27 - With less than the Minimum Channels OPERABLE requirement, operation may continue provided the containment purge makeup and exhaust isolation valves are maintained closed.
- ACTION 28 - With less than the Minimum Channels OPERABLE requirement, declare the associated train of Fuel Handling Building Emergency Exhaust inoperable and perform the requirements of Specification 3.9.12.
- ACTION 29 - With the number of OPERABLE channels one less than the Minimum Channels OPERABLE requirement, within 1 hour initiate isolation of the respective air intake. With no outside air intakes available, maintain operation of the Control Room Emergency Filtration System in the Recirculation Mode of Operation.
- ACTION 30 - With less than the Minimum Channels OPERABLE requirement, pre-entry purge operations shall be suspended and the containment pre-entry purge makeup and exhaust valves shall be maintained closed.

INSTRUMENTATION

ACCIDENT MONITORING INSTRUMENTATION

LIMITING CONDITIONS FOR OPERATION

3.3.3.6 The accident monitoring instrumentation channels shown in Table 3.3-10 shall be OPERABLE.

APPLICABILITY: MODES 1, 2, and 3.

ACTION:

- a. With the number of OPERABLE accident monitoring instrumentation channels less than the Total Required Number of Channels requirements shown in Table 3.3-10 restore the inoperable channel(s) to OPERABLE status within 7 days, or be in at least HOT STANDBY within the next 6 hours and in at least HOT SHUTDOWN within the following 6 hours.
- b. With the number of OPERABLE accident monitoring instrumentation channels, except the radiation monitors, the Pressurizer Safety Valve Position Indicator, or the Reactor Coolant System Subcooling Margin Monitor, less than the Minimum Channels OPERABLE requirements of Table 3.3-10, restore the inoperable channel(s) to OPERABLE status within 48 hours or be in at least HOT STANDBY within the next 6 hours and in at least HOT SHUTDOWN within the following 6 hours.
- c. With the number of OPERABLE accident monitoring instrumentation channels for the radiation monitor(s), the Pressurizer Safety Valve Position Indicator*, or the Reactor Coolant System Subcooling Margin Monitor#, less than the Minimum Channels OPERABLE requirements of Table 3.3-10, initiate the preplanned alternate method of monitoring the appropriate parameter(s) within 72 hours, and either restore the inoperable channel(s) to OPERABLE status within 7 days or prepare and submit a Special Report to the Commission, pursuant to Specification 6.9.2, within the next 14 days, that provides actions taken, cause of the inoperability, and the plans and schedule for restoring the channel(s) to OPERABLE status.
- d. The provisions of Specification 3.0.4 are not applicable.

* The alternate method shall be a check of safety valve piping temperatures and evaluation to determine position.

The alternate method shall be the initiation of the backup method as required by Specification 6.8.4.d.

TABLE 3.3-10

ACCIDENT MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>TOTAL REQUIRED NO. OF CHANNELS</u>	<u>MINIMUM CHANNELS OPERABLE</u>
1. Containment Pressure		
a. Narrow Range	2	1
b. Wide Range	2	1
2. Reactor Coolant Hot-Leg Temperature--Wide Range	2	1
3. Reactor Coolant Cold-Leg Temperature--Wide Range	2	1
4. Reactor Coolant Pressure--Wide Range	2	1
5. Pressurizer Water Level	2	1
6. Steam Line Pressure	2/steam generator	1/steam generator
7. Steam Generator Water Level--Narrow Range	N.A.	1/steam generator
8. Steam Generator Water Level--Wide Range	N.A.	1/steam generator
9. Refueling Water Storage Tank Water Level	2	1
10. Auxiliary Feedwater Flow Rate	N.A.	1/steam generator
11. Reactor Coolant System Subcooling Margin Monitor	N.A.	1
12. PORV Position Indicator*	N.A.	1/valve
13. PORV Block Valve Position Indicator**	N.A.	1/valve
14. Pressurizer Safety Valve Position Indicator	N.A.	1/valve
15. Containment Water Level (ECCS Sump)--Narrow Range	2	1
16. Containment Water Level--Wide Range	2	1

TABLE 3.3-10 (Continued)

ACCIDENT MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>TOTAL REQUIRED NO. OF CHANNELS</u>	<u>MINIMUM CHANNELS OPERABLE</u>
17. In Core Thermocouples	4/core quadrant	2/core quadrant
18. Plant Vent Stack--High Range Noble Gas Radiation Monitor	N.A.	1
19. Main Steam Line Radiation Monitors	N.A.	1/steam line
20. Containment--High Range Radiation Monitor	N.A.	1
21. Reactor Vessel Level	2	1
22. Containment Spray NaOH Tank Level	2	1
23. Turbine Building Vent Stack High Range Noble Gas Radiation Monitor	N.A.	1
24. Waste Processing Building Vent Stack High Range Noble Gas Radiation Monitors		
a. Vent Stack 5	N.A.	1
b. Vent Stack 5A	N.A.	1
25. Condensate Storage Tank Level	2	1

*Not applicable if the associated block valve is in the closed position.

**Not applicable if the block valve is verified in the closed position and power is removed.

TABLE 4.3-7

ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

INSTRUMENT	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
1. Containment Pressure		
a. Narrow Range	M	R
b. Wide Range	M	R
2. Reactor Coolant Hot-Leg Temperature--Wide Range	M	R
3. Reactor Coolant Cold-Leg Temperature--Wide Range	M	R
4. Reactor Coolant Pressure--Wide Range	M	R
5. Pressurizer Water Level	M	R
6. Steam Line Pressure	M	R
7. Steam Generator Water Level--Narrow Range	M	R
8. Steam Generator Water Level--Wide Range	M	R
9. Refueling Water Storage Tank Water Level	M	R
10. Auxiliary Feedwater Flow Rate	M	R
11. Reactor Coolant System Subcooling Margin Monitor	M	R
12. PORV Position Indicator	M	R
13. PORV Block Valve Position Indicator	M	R
14. Pressurizer Safety Valve Position Indicator	M	R
15. Containment Water Level (ECCS Sump)--Narrow Range	M	R
16. Containment Water Level--Wide Range	M	R

TABLE 4.3-7 (Continued)

ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
17. In Core Thermocouples	M	R
18. Plant Vent Stack--High Range Noble Gas Monitor	M	R
19. Main Steam Line Radiation Monitors	M	R
20. Containment--High Range Radiation Monitor	M	R*
21. Reactor Vessel Level	M	R
22. Containment Spray NaOH Tank Level	M	R
23. Turbine Building Vent Stack High Range Noble Gas Monitor	M	R
24. Waste Processing Building Vent Stack High Range Noble Gas Monitors		
a. Vent Stack 5	M	R
b. Vent Stack 5A	M	R
25. Condensate Storage Tank Level	M	R

* CHANNEL CALIBRATION may consist of an electronic calibration of the channel, not including the detector, for range decades above 10 R/h and a one point calibration check of the detector below 10 R/h with an installed or portable gamma source.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO.35 TO FACILITY OPERATING LICENSE NO. NPF-63
CAROLINA POWER & LIGHT COMPANY
SHEARON HARRIS NUCLEAR POWER PLANT, UNIT 1
DOCKET NO. 50-400

1.0 INTRODUCTION

By letters dated April 2, 1992, October 14, 1992, and December 21, 1992, Carolina Power & Light (CP&L, the licensee) submitted NLS-92-090, NLS-92-269, and NLS 92-332 requesting amendment to the Technical Specifications for the Shearon Harris Nuclear Power Plant (SHNPP). The proposed amendment revises Instrumentation Technical Specifications (TS) 3.3.2, 3.3.3.1, and 3.3.3.6 by making editorial corrections, clarifying rewording and/or simplifying presentation of the requirements.

2.0 DISCUSSION AND EVALUATION

The changes requested in this amendment are mostly editorial corrections and rewording to the following TS to clarify and simplify their presentation and, in some cases, to provide consistency with other TS requirements.

1. TS 3.3.2: Engineering Safety Features Actuation System Instrumentation.

The outline notation for Table 3.3-3, Item 3.c.4.c. was revised from "c)" to "c.".

Evaluation:

The staff verified that, Table 3.3-3 is arranged in an outline fashion with each line item representing a set or subset of instruments, and that the revision to the outline notion for the above Item in Table 3.3-3, on TS page 3/4 3-21, from 3.c.4.c) to Item 3.c.4.c. (i.e., ending notion "c)" to "c."), is consistent with the remainder of the notation convention in the Table. This is an editorial correction, and is appropriate and acceptable.

2. TS 3.3.3.1: Radiation Monitoring for Plant Operations.

The staff reviewed the following three proposed changes related to Table 3.3-6:

a. Table 3.3-6, Items 2a. and 2b.

Proposed change: The proposed change revises column entries for both Items 2a. and 2b. in Table 3.3-6, TS page 3/4 3-51, in the following manner:

Entry "1***" is revised to read "1/train***" for column "CHANNEL TO TRIP", and Entry "2" is revised to read "1/train 2 trains" for column "MINIMUM CHANNELS OPERABLE."

Evaluation:

The staff has verified: (1) from the plant Final Safety Analysis Report (FSAR), that Items 2a. and 2b. are the North and South area radiation monitor networks of the Fuel Handling Building (FHB) - Spent Fuel Pool Area, and (2) from the plant TS, that the operability of the FHB-Emergency Exhaust System (EES) is governed by TS 3.9.12 and that its Action Statements require that two channels, one from each train, be operable in the North monitoring network as well as in the South network. The staff also reviewed the Bases Section for TS 3.3.3.1, and verified that the above revised entries are consistent with the required operability of the radiation monitoring instrumentation for ensuring associated action will be initiated when the radiation level monitored by each channel or combination thereof in each train reaches its setpoint during plant operations.

From the above review, the staff has determined that the above revised entries for items 2a. and 2b. in Table 3.3-6 on TS page 3/4 3-51 reflect the number of operable channels required per safety train and are consistent with the plant FSAR, the requirements of TS 3.9.12, and the Bases in TS B 3/4. 3.3.1. Therefore, the changes are appropriate and acceptable.

b. Action Statement 28 in Table 3.3-6

The Action Statement 28 in Table 3.3-6, on page 3/4 3-53, for the Fuel Pool operating floor area radiation monitors is changed to make it consistent with, but not more restrictive than, the Fuel Handling Building Emergency Exhaust System Action Statements for TS 3.9.12. Specifically, if a required train of radiation monitors is inoperable then the associated train of the FHB-EES is declared inoperable and the Action Statements of TS 3.9.12 for the FHB-EES apply.

Evaluation:

The staff reviewed the licensee's submittal and verified from the plant FSAR that the monitor network's high radiation alarm for Items 2a. and 2b. (i.e., the North and South area radiation monitor networks of the Fuel Handling Building (FHB) - Spent Fuel Pool Area) is designed to automatically activate the FHB-Emergency Exhaust System (EES) when the monitor's setpoint is reached. This is consistent with the Bases Section for TS 3.3.3.1, which stated that the operability requirements for the radiation monitoring instrumentation are to ensure that, during plant operation, the associated action will be initiated when the radiation level setpoint for the radiation monitor network instrumentation's setpoint is reached. The associated action, in this case, is the automatic initiation of the two FHB-EES safety trains stated in TS 3.9.12.

The staff verified operability requirements for the FHB-EES contained in TS 3.9.12, in that it requires two EES trains to be operable in the FHB. Action Statement "a" for TS 3.9.12 states that with one of the two required EES trains inoperable, fuel movement within the storage pool or crane operation with loads over the storage pool may proceed, provided the operable FHB-EES train is capable of being powered from an operable emergency power source and is in operation and discharging through at least one train of HEPA filters and charcoal adsorber. With both trains inoperable, the Action Statement requires that all operations be suspended until one train is restored to operable status.

Based on the above review, the staff finds that the above revision to the Action Statement 28 in Table 3.3-6 is consistent with the required operability of the radiation monitoring instrumentation during plant operations and fuel movement and, therefore, is acceptable.

c. Table 3.3-6, Action Statement 30

Change is made to Action 30, on page 3/4 3-53, to capitalize "Minimum Channel". This change is editorial and consistent with the existing format in the other Action Statements for Table 3.3-6 and, therefore, is acceptable.

3. TS 3.3.3.6: Accident Monitoring Instrumentation

Five changes are made to this TS which affect Action Statements a. through c., and Tables 3.3-10 and 4.3-7. The staff evaluation of these changes are as followed:

- a. Remove the ambiguity between the defined scope of Action Statement a. (with its stated exceptions) and entries under column "Total Required Number of Channels" in Table 3.3-10 for Items 11 and 14.

The staff reviewed the correlation between (1) TS 3.3.3.6, Action Statement a. on page 3/4 3-66 which states, "With the number of OPERABLE accident monitoring instrumentation channels less than the Total Required Number of Channels shown in Table 3.3-10, except for the pressurizer safety valve position indicator or the sub-cooling margin monitor, restore the inoperable channel(s) to Operable status within 7 days, ...", and (2) Table 3.3-10 for entries under column "Total Required Number of Channels" on page 3/4 3-68. The staff also reviewed the applicable FSAR section for Reactor Vessel Level Instrumentation System and found no correlation between entries under the above mentioned column and the design requirements for Table 3.3-10, Items 11 and 14. The staff agreed that the current entries under column "Total Required Number of Channels" as "2" for both Items 11 (sub-cooling margin monitor) and 14 (Pressurizer safety valve position indicator) has no meaning and is ambiguous to the actual requirements of the TS. Based on this review, the staff finds the change to be appropriate and acceptable.

b. Editorial Changes

Changes to remove the ";or" at the end of both Action Statements a. and b. on page 3/4 3-66 to make their format consistent with the rest of other Action Statements in the TS were proposed. These are editorial changes and are acceptable.

c. Editorial Changes

Changes were proposed to Action Statement c. on page 3/4 3-66 which include referencing Table 3.3-10, deleting an unnecessary comma, and placing parentheses around the "s" of channels so the format of this Action Statement is consistent with Action Statements a. and b. on the same page. These are editorial changes and are acceptable.

d. Changes to Table 3.3-10

These changes would replace each entry under the column headed "Total Required Number of Channels" with an "N.A." It also adds the word "Pressurizer" to Item 14.

The staff reviewed the old entry under column "Total Required Number of Channels" for each of these Items and found each entry listed the same requirement as under column "Minimum Channels Operable." The staff reviewed action required for both Action Statements a. and b., and found (1) the Limiting Condition for Operation (LCO) for Action Statement a., with a seven day time limit, would apply if the number of operable instruments falls below the entry under column "Total Required Number of Channels"; and (2) the LCO for Action Statement b., with a 48 hour time limit, would also apply, if the number of operable instruments falls below the "Minimum Channels Operable." The staff found that the above changes are appropriate since 1) the entry number for the required operable instruments under column "Total..." is the same number as that under column "Minimum...", for each of Items 7, 8, 10, 12 and 13 in Table 3.3-10, 2) they delete inference to the required actions under Action Statement a., a less restrictive of the two action requirements, and 3) they retain the required actions under Action Statement b., which is more restrictive. For Item 14, the staff finds the addition of the term "Pressurizer" to the Item will provide better clarification and consistency to the listed instrument. Based on the above review, the staff found the changes will prevent the possibility of a misreading of the TS requirements in a non-conservative way and, therefore, are acceptable.

e. Editorial Changes

Changes are made to revise (1) the instrument description for Table 3.3-10, Items 18, 23 and 24 on page 3/4 3-69, and (2) the surveillance requirements for Table 4.3-7, Items 14, 18, 23 and 24 on pages 3/4 3-70 and 3/4 3-71. This revision is needed to make the instrument description for the above Items consistent with other references to these instruments elsewhere in these TS as well as standard plant nomenclature as described in FSAR Section 11.5.2, Plant Radiation Monitoring System. These changes are appropriate and are acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the State of North Carolina official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment involves changes with respect to the location or use of facility components located within the restricted area as defined in 10 CFR Part 20, or changes a surveillance requirement. The staff has determined that the amendment involve no significant increase in the amounts, and no significant change in types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (57 FR 28197). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: N. B. Le

Date: March 2, 1993