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February 4, 2002
L-02-008

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555-0001

Subject: Beaver Valley Power Station, Unit No. 1 and No. 2
BV-1 Docket No. 50-334, License No. DPR-66
BV-2 Docket No. 50-412, License No. NPF-73
Supplement to License Amendment Request Nos. 295 and 167

Pursuant to 10 CFR 50.90, FirstEnergy Nuclear Operating Company (FENOC) requested an amendment to the above licenses in the form of changes to the technical specifications. These changes were submitted by FENOC letter L-01-135, dated October 31, 2001. The License Amendment Requests propose the creation of a Pressure and Temperature Limits Report (PTLR) for each unit based on the guidance provided by Generic Letter 96-03 "Relocation of the Pressure Temperature Limit Curves and Low Temperature Overpressure Protection System Limits." This letter transmits additional technical specification pages to be included with License Amendment Request Nos. 295 and 167.

Attachment A contains three pages that should be included with those proposed by License Amendment Request Nos. 295 and 167. License Amendment Request Nos. 295 and 167 proposed revisions to the technical specifications to reference the PTLR instead of specific pressure and temperature limits within the technical specifications. The attached technical specification pages contain additional references that must be changed to properly implement the PTLR. These changes are editorial in that they replace a reference to Technical Specification 3.4.9.3 with a reference to the PTLR. Two of the pages, i.e., page 3/4 4-6 (Unit 1) and 3/4 4-9 (Unit 2), are the result of Amendments 246 (Unit 1) and 124 (Unit 2) issued on January 24, 2002. The changes incorporated into these amendments were proposed by License Amendment Request Nos. 287 and 159 submitted by FENOC letter L-01-047 dated March 28, 2001. These amendments relocated a number of technical specifications and created these pages (Technical Specification 3/4.4.3, "Safety Valves") for each unit. These technical specifications pages did not exist when License Amendment Request Nos. 295 and 167 were submitted. The third page, i.e., 3/4 4-2 (Unit 2), was inadvertently omitted from the original submittal of License Amendment Request Nos. 295 and 167.

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Since the attached changes are editorial, the safety analysis and no significant hazard evaluation contained in L-01-135 are not changed.

An implementation period of up to 60 days is requested following the effective date of this amendment.

If there are any questions concerning this matter, please contact Mr. Larry R. Freeland, Manager, Regulatory Affairs/Corrective Action at 724-682-5284.

I declare under penalty of perjury that the foregoing is true and correct. Executed on February 4, 2002.

Sincerely,



Lew W. Myers

c: Mr. L. J. Burkhart, Project Manager
Mr. D. M. Kern, Sr. Resident Inspector
Mr. H. J. Miller, NRC Region I Administrator
Mr. D. A. Allard, Director BRP/DEP
Mr. L. E. Ryan (BRP/DEP)

REACTOR COOLANT SYSTEM

3/4.4.3 SAFETY VALVES

LIMITING CONDITION FOR OPERATION

3.4.3 All pressurizer code safety valves shall be OPERABLE with a lift setting* of 2485 PSIG +1% -3%.**

APPLICABILITY: MODES 1, 2 and 3,
MODE 4 with all RCS cold leg temperatures > the enable temperature specified in ~~3.4.9.3~~the PTLR.

ACTION:

- a. With one pressurizer code safety valve inoperable, either restore the inoperable valve to OPERABLE status within 15 minutes or be in HOT SHUTDOWN with any RCS cold leg temperature \leq the enable temperature specified in ~~3.4.9.3~~the PTLR and apply RCS overpressure protection requirements in accordance with Specification 3.4.9.3 within 12 hours.
- b. With a pressurizer code safety valve having discharged liquid water from a water solid pressurizer to mitigate an overpressure event, be in at least HOT STANDBY within the next 6 hours, and in HOT SHUTDOWN with any RCS cold leg temperature \leq the enable temperature specified in ~~3.4.9.3~~the PTLR and apply RCS overpressure protection requirements in accordance with Specification 3.4.9.3 within the following 6 hours.

SURVEILLANCE REQUIREMENTS

4.4.3 No additional requirements other than those required by Specification 4.0.5.

* The Lift Setting pressure shall correspond to ambient conditions of the valve at nominal operating temperature and pressure.

** Within \pm 1% following pressurizer code safety valve testing.

REACTOR COOLANT SYSTEM

3/4.4.3 SAFETY VALVES

LIMITING CONDITION FOR OPERATION

3.4.3 All pressurizer code safety valves shall be OPERABLE with a lift setting* of 2485 psig + 1% - 3%.**

APPLICABILITY: MODES 1, 2, and 3,
With all RCS cold leg temperatures > the enable temperature specified in ~~3.4.9.3~~the PTLR.

ACTION:

- a. With one pressurizer code safety valve inoperable, either restore the inoperable valve to OPERABLE status within 15 minutes or be in HOT SHUTDOWN with any RCS cold leg temperature \leq the enable temperature specified in ~~3.4.9.3~~the PTLR and apply RCS overpressure protection requirements in accordance with Specification 3.4.9.3 within 12 hours.
- b. After any pressurizer code safety valve lift, as indicated by the safety valve position indicator, involving loop seal or water discharge; be in at least HOT STANDBY within the next 6 hours, and in HOT SHUTDOWN with any RCS cold leg temperature \leq the enable temperature specified in ~~3.4.9.3~~the PTLR and apply RCS overpressure protection requirements in accordance with Specification 3.4.9.3 within the following 6 hours.

SURVEILLANCE REQUIREMENTS

4.4.3 No additional requirements other than those required by Specification 4.0.5.

* The lift setting shall correspond to ambient conditions of the valve at nominal operating temperature and pressure.

** Within $\pm 1\%$ following pressurizer code safety valve testing.

REACTOR COOLANT SYSTEM

HOT STANDBY

This page contains
changes proposed by
LAR 157.

LIMITING CONDITION FOR OPERATION

- 3.4.1.2 a. At least two reactor coolant loops and associated steam generators and reactor coolant pumps# shall be in operation* when the rod control system is capable of control bank rod withdrawal.
- b. At least two reactor coolant loops and associated steam generators and reactor coolant pumps# shall be OPERABLE and one reactor coolant loop shall be in operation* when the rod control system is incapable of control bank rod withdrawal.

APPLICABILITY: MODE 3**

ACTION:

- a. With less than the above required reactor coolant loops OPERABLE, restore the required loops to OPERABLE status within 72 hours or be in HOT SHUTDOWN within the next 12 hours.
- b. With less than two reactor coolant loops in operation, immediately deenergize all control rod drive mechanisms, or align the rod control system so that it is incapable of control bank rod withdrawal.
- c. With no reactor coolant loop in operation, suspend all operations involving a reduction in boron concentration of the Reactor Coolant System and immediately initiate corrective action to return the required reactor coolant loop to operation.

* All reactor coolant pumps may be deenergized for up to 1 hour provided (1) no operations are permitted that would cause dilution of the Reactor Coolant System boron concentration and (2) core outlet temperature is maintained at least 10°F below saturation temperature. This does not preclude natural circulation cooldown under abnormal cooldown conditions.

** See Special Test Exception 3.10.4.

No reactor coolant pump in a non-isolated loop shall be started with one or more non-isolated RCS cold leg temperatures less than or equal to the enable temperature specified in the PTLRset forth in Specification 3.4.9.3, unless the secondary side water temperature of each steam generator in a non-isolated loop is less than 50°F above each of the non-isolated RCS cold leg temperatures.