

Docket No. 50-261

December 7, 1992

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See next page

Mr. R. A. Watson
Senior Vice President
Nuclear Generation
Carolina Power & Light Company
Post Office Box 1551
Raleigh, North Carolina 27602

Dear Mr. Watson:

SUBJECT: ISSUANCE OF AMENDMENT NO. 142 TO FACILITY OPERATING LICENSE NO. DPR-23 REGARDING FIRE PROTECTION - H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2 (TAC NO. M82863)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 142 to Facility Operating License No. DPR-23 for the H. B. Robinson Steam Electric Plant, Unit No. 2 (HBR2). This amendment consists of changes to the Operating License and the Technical Specifications (TS) in response to your request dated February 21, 1992, as supplemented August 7, 1992.

The Amendment revises the fire protection license condition of the Operating License and relocates the fire protection TS to plant procedures and to the Final Safety Analysis Report (FSAR), in accordance with the guidance provided in Generic Letters 86-10 and 88-12.

A copy of our related Safety Evaluation is enclosed. Notice of Issuance will be included in the Commission's bi-weekly Federal Register notice.

Sincerely,

Original signed by:

Brenda L. Mozafari, Project Manager
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 142 to DPR-23
2. Safety Evaluation

cc w/enclosures:
See next page

OFC	LA:PD21:DRPE	PM:PD21:DRPE	SPLB*	OGC
NAME	PAnderson	BMozafari:dt	CMcCracken	S. Hony
DATE	11/19/92	11/19/92	11/13/92	11/23/92
OFC	D:PD21:DRPE			
NAME	EAAdensam			
DATE	12/7/92			

Document Name: ROB82863.AMD

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P PDR

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Mr. R. A. Watson
Carolina Power & Light Company

H. B. Robinson Steam Electric
Plant, Unit No. 2

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AMENDMENT NO. 142 TO FACILITY OPERATING LICENSE NO. DPR-23 - H. B. ROBINSON
STEAM ELECTRIC PLANT, UNIT NO. 2

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

CAROLINA POWER & LIGHT COMPANY

DOCKET NO. 50-261

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 142
License No. DPR-23

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Carolina Power & Light Company (the licensee), dated February 21, 1992, as supplemented August 7, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 3.B. of Facility Operating License No. DPR-23 is hereby amended to read as follows:

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B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 142, are hereby incorporated in the license. Carolina Power & Light Company shall operate the facility in accordance with the Technical Specifications.


3. License Condition 3.E. of Facility Operating License No. DPR-23 is hereby amended to read as follows:

E. Fire Protection

Carolina Power & Light Company shall implement and maintain in effect all provisions of the approved Fire Protection Program as described in the Final Safety Analysis Report for the facility and as approved in the Fire Protection Safety Evaluation Report, dated February 28, 1978, and supplements thereto. Carolina Power & Light Company may make changes to the approved Fire Protection Program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

4. This license amendment is effective as of the date of its issuance and shall be implemented within 90 days.

FOR THE NUCLEAR REGULATORY COMMISSION


Elinor G. Adensam, Director
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 7, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 142

FACILITY OPERATING LICENSE NO. DPR-23

DOCKET NO. 50-261

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Remove Pages

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3.14-4
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Insert Pages

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3.9.6	Compliance with 40 CFR Part 190 - Radioactive Effluents from Uranium Fuel Cycle Sources	3.9-6
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3.10.3	Quadrant Power Tilt Limits	3.10-7a
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3.12	Seismic Shutdown	3.12-1
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- a. All non-automatic containment isolation valves not required for normal operation are closed and blind flanges are properly installed where required.
- b. The equipment door is properly closed and sealed.
- c. At least one door in the personnel air lock is properly closed and sealed.
- d. All automatic containment isolation trip valves required to be closed during accident conditions are operable or are secured closed except as stated in Specification 3.6.3. Manual valves qualifying as automatic containment isolation valves are secured closed.
- e. The uncontrolled containment leakage satisfies Specification 4.4.

1.8 QUADRANT POWER TILT

The quadrant power tilt is defined as the ratio of maximum to average of the upper excore detector currents or the lower excore detector currents, whichever is greater. If one excore is out of service, the three in-service units are used in computing the average.

1.9 DELETED

1.10 STAGGERED TEST BASIS

A Staggered Test Basis shall consist of:

- a. A test schedule for n systems, subsystems, trains or designated components obtained by dividing the specified test interval into n equal subintervals.

PAGES 3.14-1 THROUGH 3.14-8 HAVE BEEN DELETED.

(NEXT PAGE IS 3.15-1)

TABLE 4.1-3

FREQUENCIES FOR EQUIPMENT TESTS

		<u>Check</u>	<u>Frequency</u>	<u>Maximum Time Between Tests</u>
1.	Control Rods	Rod drop times of all full length rods	Each refueling shutdown	NA*
2.	Control Rod	Partial movement of all full length rods	Every 2 weeks during reactor critical operations	20 days
3.	Pressurizer Safety Valves	Set point	Each refueling shutdown	NA
4.	Main Steam Safety Valves	Set point	Each refueling shutdown	NA
5.	Containment Isolation Trip	Functioning	Each refueling shutdown	NA
6.	Refueling System Interlocks	Functioning	Prior to each refueling shutdown	NA
7.	Service Water System	Functioning	Each refueling shutdown	NA
8.	DELETED			
9.	Primary System Leakage	Evaluate	Daily when reactor coolant system is above cold shutdown condition	NA
10.	Diesel Fuel Supply	Fuel Inventory	Weekly	10 days
11.	Critical Headers of Auxiliary Coolant System	100 Psig Hydrostatic Test	Every five years	6 years
12.	Turbine Steam Stop, Control, Reheat Stop, and Interceptor Valves	Closure	Monthly during power operation and prior to startup	45 days

PAGES 4.14-1 THROUGH 4.14-4 HAVE BEEN DELETED.

(NEXT PAGE IS 4.15-1)

- f) ALL CORE ALTERATIONS after the initial fuel loading shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator limited to fuel handling who has no other concurrent responsibilities during this operation.
- g) DELETED
- h) The shift complement may be one less than the minimum requirement of Section 6.2.3.a and 6.2.3.b for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift members provided immediate action is taken to restore the shift complement to within the minimum requirements of Section 6.2.3.a and 6.2.3.b. This provision does not permit any shift member position to be unmanned upon shift change due to an oncoming shift member being late or absent.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the plant staff shall be maintained under the direction of the Manager - Training and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

6.4.2 DELETED

6.5.1.6.5 A quorum of the PNSC shall consist of the Chairman, and four members, of which two may be alternates.

6.5.1.6.6 The PNSC activities shall include the following:

- a) Perform an overview of Specifications 6.5.1.1 and 6.5.1.2 to assure that processes are effectively maintained.
- b) Performance of special reviews, investigations, and reports thereon requested by the Manager - Nuclear Assessment Department.
- c) Annual review of the Security Plan and Emergency Plan.
- d) Perform reviews of Specifications 6.5.1.1.6, 6.5.1.2.4, 6.5.1.3.1, and 6.5.1.4.1.
- e) Perform review of all reportable events.
- f) Review of facility operations to detect potential nuclear safety hazards.
- g) Review of every unplanned on site release of radioactive material to the environs including the preparation and forwarding of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrences to the Vice President - Robinson Nuclear Project, Manager - Nuclear Assessment Department.
- h) Review of changes to the Process Control Program and the Offsite Dose Calculation Manual.
- i) Review of major changes to radioactive liquid, gaseous, and solid waste treatment systems.
- j) Review of changes to the CORE OPERATING LIMITS REPORT.
- k) Annual review of the Fire Protection Program, including Program changes.

6.9.2 Deleted

6.9.3 Special Reports

6.9.3.1 Special reports shall be submitted to the NRC within the time period specified for each report. These reports shall be submitted covering the activities identified below pursuant to the requirements of the applicable reference specification:

	<u>Area</u>	<u>Reference</u>	<u>Submittal Date</u>
a)	Containment Leak Rate Testing	4.4	Upon completion of each test.
b)	Containment Sample Tendon Surveillance	4.4	Upon completion of the inspection at 25 years of operation.
c)	Post-Operational Containment Structural Test	4.4	Upon completion of the test at 20 years of operation.
d)	DELETED		
e)	Overpressure Protection System Operation	3.1.2.1.e	Within 30 days of operation.
f)	Auxiliary Feedwater Pump	3.4	Within 30 days after becoming inoperable.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 142 TO FACILITY OPERATING LICENSE NO. DPR-23

CAROLINA POWER & LIGHT COMPANY

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2

DOCKET NO. 50-261

1.0 INTRODUCTION

By letter dated February 21, 1992, as supplemented August 7, 1992, the Carolina Power & Light Company (the licensee) submitted a request for changes to the H. B. Robinson Steam Electric Plant, Unit No. 2, Technical Specifications (TS). The requested changes would remove requirements for the fire detection systems, fire suppression systems, fire barriers, and fire brigade staffing requirements recommended by Generic Letter 86-10. The proposed changes would also modify the administrative control requirements of the TS to add requirements for the Fire Protection Program that are similar to requirements for other programs implemented by license condition. Guidance on these proposed TS changes was provided to all power reactor licensees and applicants by Generic Letter 88-12, dated August 2, 1988. The August 7, 1992, letter provided an updated TS page that did not change the initial proposed no significant hazards consideration determination.

2.0 BACKGROUND

Following the fire at the Browns Ferry Nuclear Power Plant on March 22, 1975, the Commission undertook a number of actions to ensure that improvements were implemented in the Fire Protection Programs for all power reactor facilities. Because of the extensive modification of Fire Protection Programs and the number of open issues resulting from staff evaluations, a number of revisions and alterations occurred in these programs over the years. Consequently, the licensees were requested by Generic Letter 86-10 to incorporate the final NRC-approved Fire Protection Program into their Final Safety Analysis Reports (FSARs). In this manner, the Fire Protection Program, including the systems, the administrative and technical controls, the organization, and other plant features associated with fire protection, would have a status consistent with that of other plant features described in the FSAR. In addition, the Commission concluded that a standard license condition, requiring compliance with the provisions of the Fire Protection Program as described in the FSAR, should be used to ensure uniform enforcement of fire protection requirements. Finally, the Commission stated that with the requested actions, licensees may request an amendment to delete the fire protection TS that would now be unnecessary.

The licensees for the Callaway and Wolf Creek plants submitted lead-plant proposals to remove fire protection requirements from their TS. This action was an industry effort to obtain NRC guidance on an acceptable format for license amendment requests to remove fire protection requirements from TS.

Additionally, in the licensing review of new plants, the staff has approved applicant requests to remove fire protection requirements from TS issued with the operating license. Thus, on the basis of the lead-plant proposals and the staff's experience with TS for new licenses, Generic Letter 88-12 was issued to provide guidance on removing fire protection requirements from TS.

3.0 EVALUATION

Generic Letter 86-10 recommended the removal of fire protection requirements from the TS. Although a comprehensive Fire Protection Program is essential to plant safety, the basis for this recommendation is that many details of this program that are currently addressed in the TS can be modified without affecting nuclear safety. Such modifications can be made provided that there are suitable administrative controls over these changes. These details, that are presently included in TS and which are removed by this amendment, do not constitute performance requirements necessary to ensure safe operation of the facility and, therefore, do not warrant being included in the TS. At the same time, suitable administrative controls ensure that there will be careful review and analysis by competent individuals of any changes in the Fire Protection Program, including those technical and administrative requirements removed from the TS, to ensure that nuclear safety is not adversely affected.

These controls include (1) the TS administrative controls that are applicable to the Fire Protection Program; (2) the license condition on implementation of, and subsequent changes to, the Fire Protection Program; and (3) the 10 CFR 50.59 criteria for evaluating changes to the Fire Protection Program as described in the FSAR.

The specific details relating to fire protection requirements removed from TS by this amendment include those specifications for fire detection systems, fire suppression systems, fire barriers, and fire brigade staffing requirements. The administrative control requirements have been modified to include Fire Protection Program implementation as an element for which written procedures must be established, implemented, and maintained. In addition, the responsibilities of the Operations Review Committee were expanded to include the review of the Fire Protection Program and its implementing procedures and submittal of recommended changes to the Plant Nuclear Safety Committee.

The TS changes proposed by the licensee are in accordance with the guidance provided by Generic Letter 88-12, as addressed in the items below.

1. The requirements of TS 3.14.1 and 4.14.1 (Fire Detection and Actuation Instrumentation), 3.14.2 and 4.14.2 (Fire Suppression Water System), 3.14.3 and 4.14.8 (Fire Water Pre-Action System), 3.14.4 and 4.14.4 (Fire Hose Stations), 3.14.5 and 4.14.3 (CO₂ Fire Protection System), 3.14.6 and 4.14.7 (Halon Fire Protection System), and 3.14.7 and 4.14.5 (Fire Barrier Penetration Seals) have been replaced with the requirements which appear in Section 9.5.1 of the FSAR. The operability and surveillance requirements for these systems will be maintained through plant procedures.
2. TS definition 1.9, FIRE PROTECTION WATER SYSTEM is deleted.

3. TS 6.5.1.6.6(j) (Operations Review Committee Responsibilities) will be added to require the Plant Nuclear Safety Committee (PNSC) to review the Fire Protection Program and any program changes.
4. TS 6.2.3(g) and (h) and the training requirements of TS 6.4.2 are deleted.
5. TS 6.9.3.1 will be revised to remove the special reporting requirements associated with fire protection systems.

The proposed license condition for HBR2 reads as follows:

Carolina Power & Light Company shall implement and maintain in effect all provisions of the approved Fire Protection Program as described in the Final Safety Analysis Report for the facility and as approved in the Fire Protection Safety Evaluation Report, dated February 28, 1978, and supplements thereto. Carolina Power & Light Company may make changes to the approved Fire Protection Program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

As required by Generic Letter 86-10, the licensee confirmed that the NRC-approved Fire Protection Program has been incorporated into the FSAR.

The licensee confirmed that the operational conditions, remedial actions, and test requirements associated with the removed fire protection TS have been included in the Fire Protection Program incorporated into the FSAR. This is in accordance with the guidance of Generic Letter 88-12. Therefore, the staff finds the proposed changes acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the State of South Carolina official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes to the surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (57 FR 11103, April 1, 1992). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: T. Dunning
B. Mozafari
A. Singh

Date: December 7, 1992