



50-261

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

October 24, 1997

Mr. J. S. Keenan, Vice President
Carolina Power & Light Company
H. B. Robinson Steam Electric Plant
Unit No. 2
3581 West Entrance Road
Hartsville, South Carolina 29550

SEP
Approved To DL

SUBJECT: ISSUANCE OF AMENDMENT NO. 176 TO FACILITY OPERATING LICENSE
NO. DPR-23 REGARDING CONVERSION TO IMPROVED STANDARD
TECHNICAL SPECIFICATIONS - H. B. ROBINSON STEAM ELECTRIC PLANT,
UNIT NO. 2 (TAC NO. M96440)

Dear Mr. Keenan:

The Commission has issued the enclosed Amendment No. 176 to Facility Operating License No. DPR-23 for the H. B. Robinson Steam Electric Plant, Unit No. 2 (HBR). The amendment consists of the changes described below.

The amendment reflects full conversion from your current Technical Specifications (TS) to a set of TS based on NUREG-1431, "Standard Technical Specifications Westinghouse Plants," Revision 0, dated September 1992 (including approved travellers used in the issuance of Revision 1, dated April 1995). This is in response to your application dated August 27, 1996, as supplemented by letters dated December 18, 1996, January 17, February 18, March 27, April 4, April 25, April 29, May 30, June 2, June 13, June 18, August 4, August 8, September 10, October 2 (RNP RA/97-0216), October 2 (RNP RA/97-0207), October 13, and October 21, 1997.

In addition, the instrument setpoint methodology used to determine trip setpoints and allowable values in the conversion to the improved TS is approved. The NRC staff reviewed the instrument setpoint methodology and has concluded that it is acceptable.

The NRC staff reviewed the proposed revision to the TS submitted with the August 27, 1996, letter regarding the proposed changes that would allow one safety injection (SI) pump and all three charging pumps to be operable for a reactor coolant system (RCS) temperature range of $175^{\circ}\text{F} \leq T_{\text{cold}} \leq 350^{\circ}\text{F}$. These changes would provide HBR with an automatic, single train emergency core cooling system (ECCS) injection capability in Mode 4 consistent with NUREG-1431, "Standard Technical Specifications Westinghouse Plants," Revision 1. Because the affected temperature range is within the required operation range of the low temperature overpressure protection (LTOP) system and because the LTOP analysis of record did not account for injection from an SI pump, the licensee performed a new LTOP overpressurization events analysis for the proposed configuration. The NRC staff has concluded that the new analysis supports the revised TS.

The staff reviewed the proposed revision to the TS submitted with the August 27, 1996, letter, as further discussed in your letters dated April 4 and April 29, 1997, regarding a more restrictive change in minimum containment pressure. That change was determined

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staff to be beyond the scope of the conversion to improved technical specifications. The staff has concluded that the analysis provided by Carolina Power & Light Company supports this revised TS and that the change enhances plant safety. As discussed in the accompanying Safety Evaluation, the staff has made a final determination that no significant hazards consideration is involved for this proposed amendment and that the amendment should be issued as allowed by the criteria contained in 10 CFR 50.91.

Appendix B TS contain environmental reporting requirements which were relocated to Appendix B as an interim action in 1976 pending reissuance of comprehensive Appendix B Environmental TS. These TS are being relocated to the Offsite Dose Calculation Manual (ODCM). These requirements are comparable to portions of other Radiological Environmental Monitoring TS that are also being separately relocated because they do not relate to mitigating a design basis accident or transient. These TS are not required to be in the TS under 10 CFR 50.36 and do not meet any of the four criteria in the "NRC Final Policy Statement on Technical Specification Improvements for Nuclear Power Reactors," published on July 22, 1993 (58 FR 39132). The NRC staff has concluded that appropriate controls have been established for all of the current specifications, information, and requirements that are being moved to licensee-controlled documents. A license condition is adopted herewith, to require incorporation of these matters in the appropriate licensee-controlled document(s). Following implementation, the NRC will audit the removed provisions to ensure that an appropriate level of control has been achieved. Accordingly, these TS, information, and requirements, as described in detail in this Safety Evaluation, may be relocated from current TS and placed in the Updated Final Safety Analysis Report or other licensee-controlled documents as specified in Carolina Power & Light Company letters dated September 10, 1997, and October 13, 1997.

A copy of the related Safety Evaluation is also enclosed. Also enclosed is the Notice of Issuance which has been forwarded to the Office of the Federal Register for publication.

Sincerely,

Original signed by D. Trimble for:

Brenda L. Mozafari, Project Manager
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-261

Enclosures:

1. Amendment No. 176 to DPR-23
2. Safety Evaluation
3. Notice

cc w/enclosures:

See next page

FILENAME - G:\ROBINSON\ROB96440.AMD *See previous concurrence

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AMENDMENT NO.176 TO FACILITY OPERATING LICENSE NO. DPR-23 - H. B. ROBINSON
STEAM ELECTRIC PLANT, UNIT NO. 2

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