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A Member of the
Constellation Energy Group

February 15, 2002

U. S. Nuclear Regulatory Commission
Washington, DC 20555

ATTENTION: Document Control Desk

SUBJECT: Calvert Cliffs Nuclear Power Plant
Unit No. 1; Docket No. 50-317
Core Operating Limits Report for Unit 1, Cycle 15, Revision 6

Pursuant to Calvert Cliffs Nuclear Power Plant Technical Specification 5.6.5, the attached Core Operating Limits Report for Unit 1, Cycle 15, Revision 6 (Attachment 1) is provided for your records.

Please replace the Unit 1 Core Operating Limits Report in its entirety, with the attached Revision 6.

Should you have questions regarding this matter, we will be pleased to discuss them with you.

Very truly yours,

CHC/DJM/bjd

Attachment: (1) Core Operating Limits Report for Unit 1, Cycle 15, Revision 6

cc: R. S. Fleishman, Esquire
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A001

ATTACHMENT (1)

CORE OPERATING LIMITS REPORT

UNIT 1, CYCLE 15, REVISION 6

Calvert Cliffs Nuclear Power Plant, Inc.
February 15, 2002

CALVERT CLIFFS NUCLEAR POWER PLANT

CORE OPERATING LIMITS REPORT

for

UNIT 1, CYCLE 15

REVISION 6

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RESPONSIBLE ENGINEER / DATE

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CORE OPERATING LIMITS REPORT

CALVERT CLIFFS UNIT 1, CYCLE 15

The following limits are included in this Core Operating Limits Report:

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INTRODUCTION

This report provides the cycle-specific core operating limits for operation of Calvert Cliffs Unit 1, Cycle 15. It contains the limits for:

Shutdown Margin (SDM)
Moderator Temperature Coefficient (MTC)
Control Element Assembly (CEA) Alignment
Regulating Control Element Assembly (CEA) Insertion Limits
Linear Heat Rate (LHR)
Total Planar Radial Peaking Factor (F_{xy}^T)
Total Integrated Radial Peaking Factor (F_r^T)
Axial Shape Index (ASI)
Reactor Protective System (RPS) Instrumentation - Operating
Boron Concentration

In addition, this report contains a number of figures which provides limits on the parameters listed above. If any of the limits contained in this report are exceeded, corrective action will be taken as defined in the Technical Specifications.

This report has been prepared in accordance with the requirements of the Technical Specifications. The cycle specific limits have been developed using the NRC-approved methodologies given in the "List of Approved Methodologies" section of this report and the Technical Specifications.

COLR Revision 0

COLR Revision 0 was issued to support the Unit 1 Cycle 15 core onload and operation of Unit 1 Cycle 15 in Modes 5 and 6. Safety Evaluation SE00424 Rev. 0000 provided justification for this revision.

COLR Revision 1

COLR Revision 1 was issued to incorporate those limits required for operation of Unit 1 Cycle 15 in Modes 1 through 4. No revision bars are included due to the significant number of changes required to incorporate the Modes 1 through 4 COLR limits. Safety Evaluation SE00407 Rev. 0000 provided justification for this revision.

COLR Revision 2

3.9.1 Boron Concentration was revised to include a Mode 6 temperature restriction of 140 °F. Safety Evaluation SE00431 Rev. 0000 provided justification for this revision.

COLR Revision 3

COLR Figures 3.2.1-2 & 3.2.5 are revised to reflect a 20% LCO monitoring floor for LHR and DNB ASI limits (SE00433, Rev. 0000).

Figures 3.2.1-2, 3.2.1-3, & 3.2.5 are revised to provide clarification in reference to NEOP-13. Section 3.2.1 & Figure 3.2.1-3 were also revised to provide clarity. Finally, Figures 3.2.2 & 3.2.3 were revised to correct an inaccuracy in the peaking factor Tech Specs (SE00451, Rev. 0000).

COLR Revision 4

Figures 3.2.1-1 and 3.2.1-3 are revised to lower the allowable PLHR from 14.3 kw/ft to 13.9 kw/ft. Figures 3.2.1-3, 3.2.2, & 3.2.3 are revised to expand the tradeoff curves below 80% power. Finally, Figure 3.2.5 and the list of references are revised to implement the ABB-NV setpoints. (SE00451, Rev. 0000).

COLR Revision 5

Figure 3.2.3 is revised to implement a more restrictive F_r^T due to ROPM issues discovered mid-cycle (SE00472, Rev. 0).

COLR Revision 6

Issued to revise the U1C15 Refueling Boron Concentration to eliminate CEA crediting beyond a cycle burnup of 16731 MWD/T. (SE00477) (ES200000484-006)

DEFINITIONS

Axial Shape Index (ASI)

ASI shall be the power generated in the lower half of the core less the power generated in the upper half of the core, divided by the sum of the power generated in the lower and upper halves of the core.

$$ASI = \frac{\textbf{lower} - \textbf{upper}}{\textbf{lower} + \textbf{upper}} = Y_E$$

The Axial Shape Index (Y_I) used for the trip and pretrip signals in the Reactor Protection System (RPS) is the above value (Y_E) modified by an appropriate multiplier (A) and a constant (B) to determine the true core axial power distribution for that channel.

$$Y_I = AY_E + B$$

Total Integrated Radial Peaking Factor - F_r^T

The Total Integrated Radial Peaking Factor is the ratio of the peak pin power to the average pin power in an unrodded core.

Total Planar Radial Peaking Factor - F_{xy}^T

The Total Planar Radial Peaking Factor is the maximum ratio of the peak to average power density of the individual fuel rods in any of the unrodded horizontal planes.

CYCLE SPECIFIC LIMITS FOR UNIT 1, CYCLE 15

3.1.1 Shutdown Margin (SDM) (SR 3.1.1.1)

Tavg > 200 °F - Modes 3 and 4:

The shutdown margin shall be equal to or greater than the limit line of COLR Figure 3.1.1.

Tavg ≤ 200 °F - Mode 5:

The shutdown margin shall be ≥ 3.0% Δρ.

3.1.3 Moderator Temperature Coefficient (MTC) (SR 3.1.3.2)

The Moderator Temperature Coefficient (MTC) shall be less negative than -3.0×10^{-4} Δρ/°F at rated thermal power.

3.1.4 Control Element Assembly (CEA) Alignment (Action 3.1.4.B.1)

The allowable time to realign a CEA may be provided by the full core power distribution monitoring system (Better Axial Shape Selection System - BASSS) or COLR Figure 3.1.4, "Allowable Time to Realign CEA Versus Initial Total Integrated Radial Peaking Factor (F_r^T)." If COLR Figure 3.1.4 is used, the pre-misaligned F_r^T value used to determine the allowable time to realign the CEA shall be the latest measurement taken within 5 days prior to the CEA misalignment. If no measurements have been taken within 5 days prior to the misalignment and the full core power distribution monitoring system is unavailable then the time to realign is zero (0) minutes.

3.1.6 Regulating Control Element Assembly (CEA) Insertion Limits (SR 3.1.6.1 and SR 3.1.6.2)

The regulating CEA groups insertion limits are shown on COLR Figure 3.1.6.

3.2.1 Linear Heat Rate (LHR) (SR 3.2.1.2 and SR 3.2.1.4)

The linear heat rate shall not exceed the limits shown on COLR Figure 3.2.1-1.

The axial shape index power dependent control limits are given in COLR Figure 3.2.1-2.

<u>When using the excore detector monitoring system (SR 3.2.1.2):</u>

The alarm setpoints are equal to the ASI limits, therefore when the alarms are adjusted, they provide indication to the operator when ASI is not within the limits.

The axial shape index alarm setpoints are shown as a function of fraction of thermal power on COLR Figure 3.2.1-2. A scaling factor (N-Factor) vs. F_{xy}^T is shown in COLR Figure 3.2.1-3. The fraction of thermal power shown in COLR Figure 3.2.1-2 must be scaled by the N-Factor to determine the axial shape index alarm setpoints as a function of fraction of rated thermal power.

CYCLE SPECIFIC LIMITS FOR UNIT 1, CYCLE 15

When using the incore detector monitoring system (SR 3.2.1.4):

The alarm setpoints are adjusted to protect the Linear Heat Rate limits shown on COLR Figure 3.2.1-1 and uncertainty factors are appropriately included in the setting of these alarms.

The uncertainty factors for the incore detector monitoring system are:

1. A measurement-calculational uncertainty factor of 1.062,
2. An engineering uncertainty factor of 1.03,
3. A linear heat rate uncertainty factor of 1.002 due to axial fuel densification and thermal expansion, and

4.a For measured thermal power less than or equal to 50 percent but greater than 20 percent of rated full core power a thermal power measurement uncertainty factor of 1.035.

4.b For measured thermal power greater than 50 percent of rated full core power a thermal power measurement uncertainty factor of 1.020.

3.2.2 Total Planar Radial Peaking Factor (F_{xy}^T) (SR 3.2.1.1 and SR 3.2.2.1)

The calculated value of F_{xy}^T shall be limited to $\leq$ 1.65.

The allowable combination of thermal power and F_{xy}^T are shown on COLR Figure 3.2.2.

3.2.3 Total Integrated Radial Peaking Factor (F_r^T) (SR 3.2.3.1)

The calculated value of F_r^T shall be limited to $\leq$ 1.57.

The allowable combinations of thermal power and F_r^T are shown on COLR Figure 3.2.3.

3.2.5 Axial Shape Index (ASI) (SR 3.2.5.1)

The axial shape index and thermal power shall be maintained within the limits established by the Better Axial Shape Selection System (BASSS) for CEA insertions of the lead bank of < 55% when BASSS is operable, or within the limits of COLR Figure 3.2.5 for CEA insertions specified by COLR Figure 3.1.6.

3.3.1 Reactor Protective System (RPS) Instrumentation - Operating (Reactor Trip Setpoints) (TS Table 3.3.1-1)

The Axial Power Distribution - High trip setpoint and allowable values are given in COLR Figure 3.3.1-1.

The Thermal Margin/Low Pressure (TM/LP) trip setpoint is given in COLR Figures 3.3.1-2 and 3.3.1-3. The allowable values are to be not less than the larger of (1) 1875 psia or (2) the value calculated from COLR Figures 3.3.1-2 and 3.3.1-3.

3.9.1 Boron Concentration (SR 3.9.1.1)

The refueling boron concentration will maintain the K_{eff} at 0.95 or less (including a 1% $\Delta K/K$ conservative allowance for uncertainties). The refueling boron concentration shall be maintained uniform. For Mode 6 operation the RCS temperature must be maintained $\leq$ 140 °F.

Refueling Boron Concentration Limits

	56 Credited CEAs	>16731 MWD/T Burnup with 0 Credited CEAs
COLR/Tech Spec Limit	2272 ppm	2272 ppm
Chemistry Sampling	23 ppm	23 ppm
Boron-10 Depletion	48 ppm	48pm
Refueling Boron Concentration Limit including Chemistry Sampling uncertainty and Boron-10 Depletion	$\geq$ 2343 ppm	$\geq$ 2343 ppm
Dilution of the Refueling Pool between Low and High Level Alarms with Refueling Pool Flooded	72 ppm	72 ppm
Additional Contingencies for Batch 1RT LFAs	20 ppm	20 ppm
Temporary Rotations of Fuel Assemblies	20 ppm	20 ppm
Refueling Boron Concentration Administrative Limit	$\geq$ 2455 ppm	$\geq$ 2455 ppm

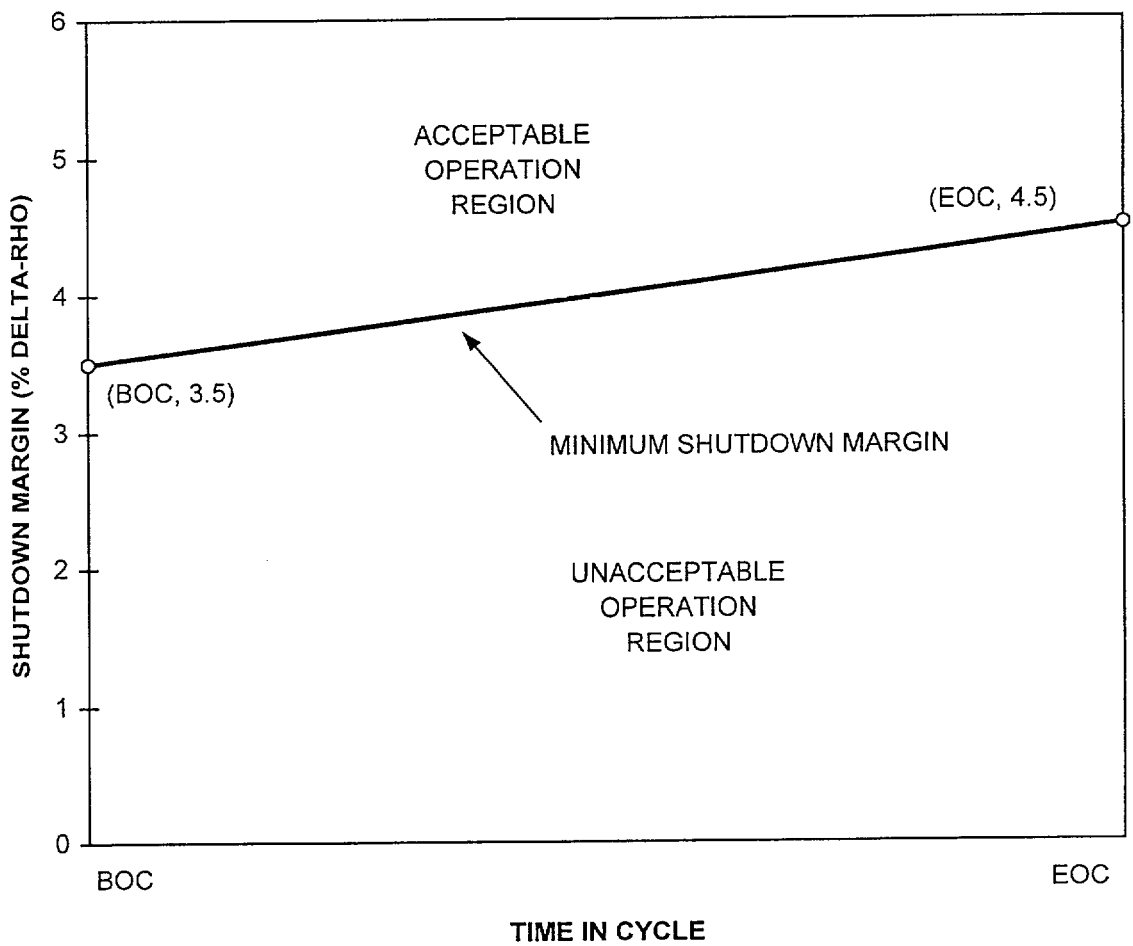

Figure 3.1.1

Shutdown Margin vs. Time in Cycle

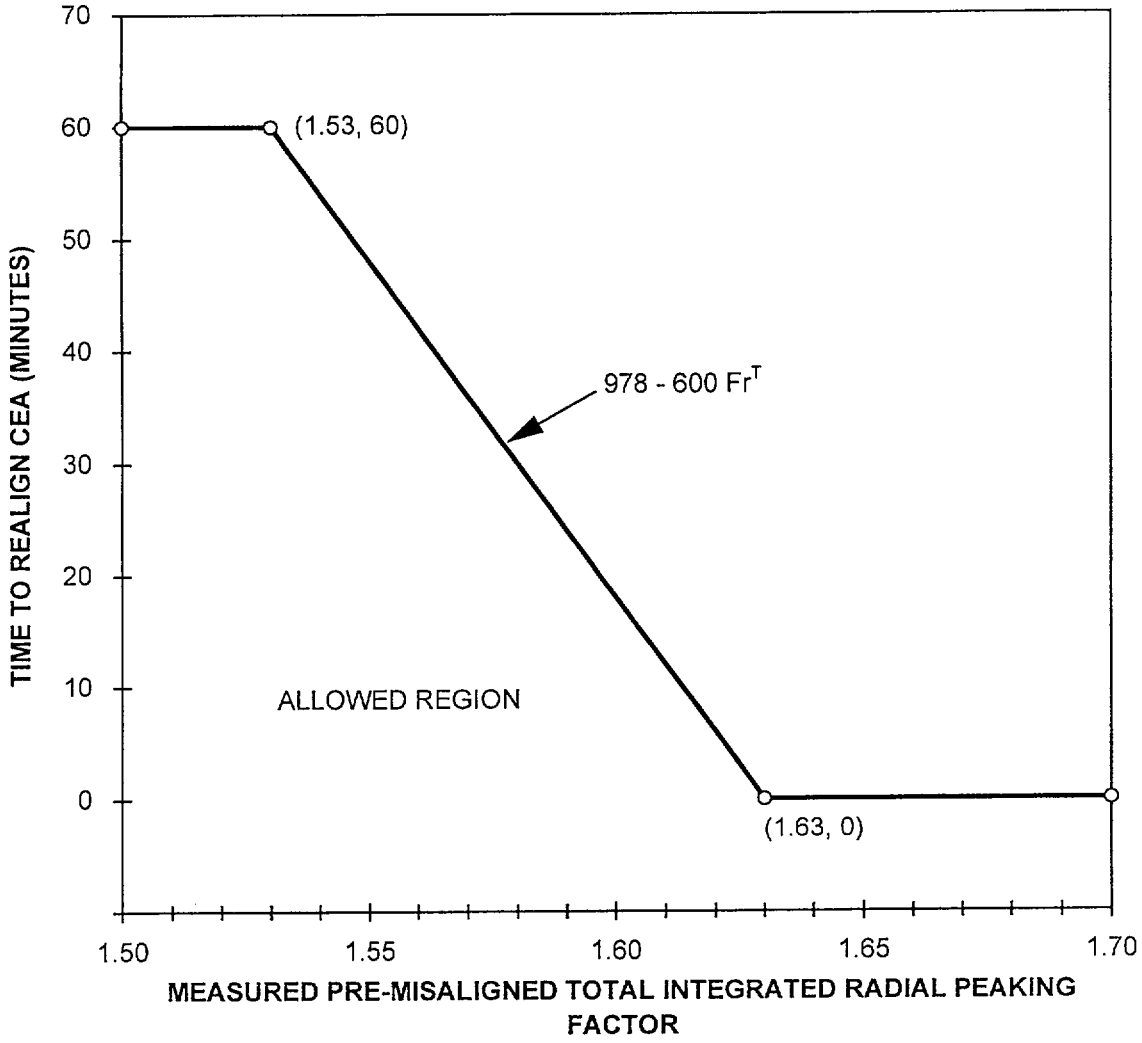

Figure 3.1.4

Allowable Time to Realign CEA Versus Initial Total Integrated Radial Peaking Factor (F_r^T)

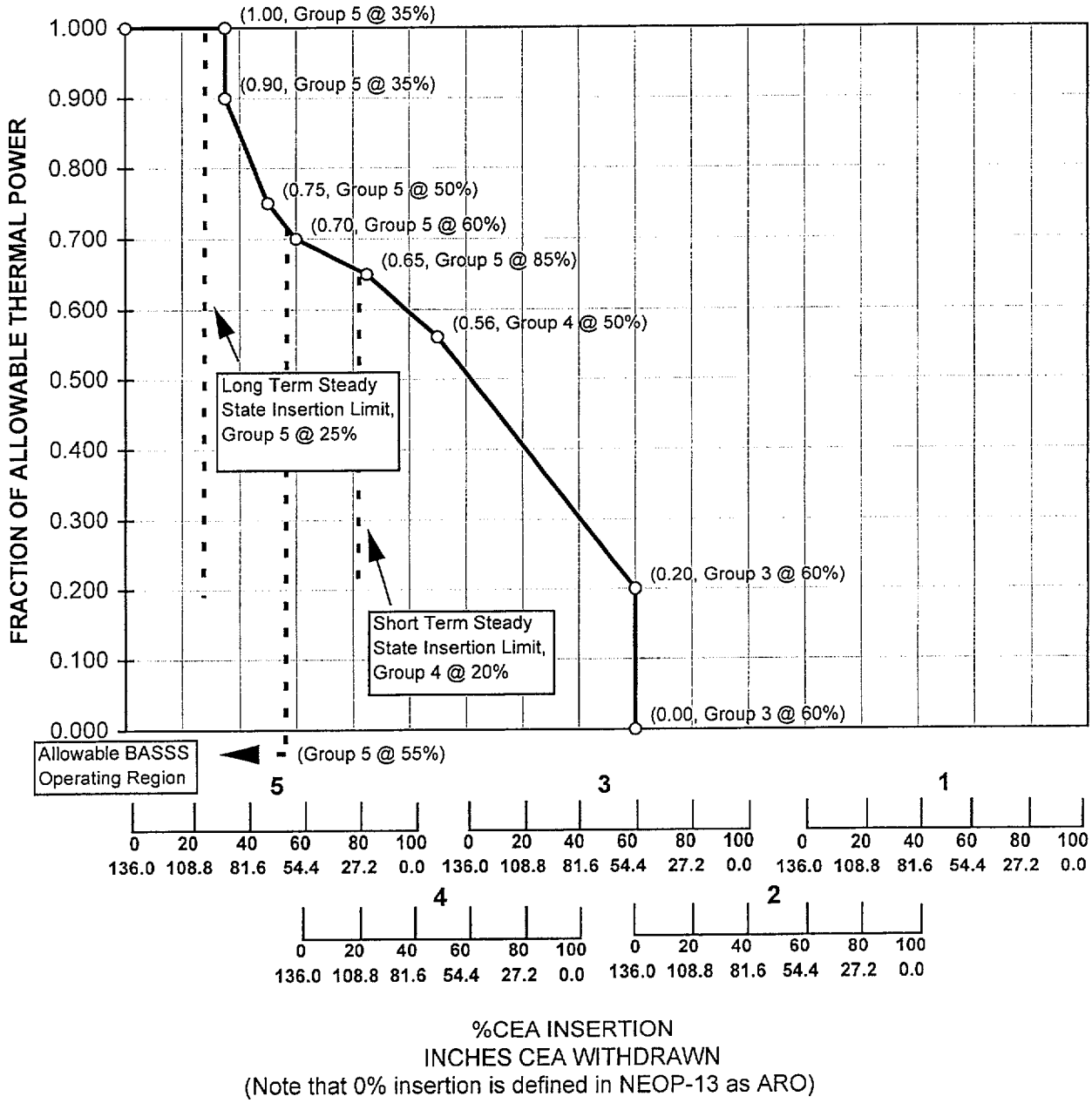

Figure 3.1.6

CEA Group Insertion Limits vs. Fraction of Rated Thermal Power

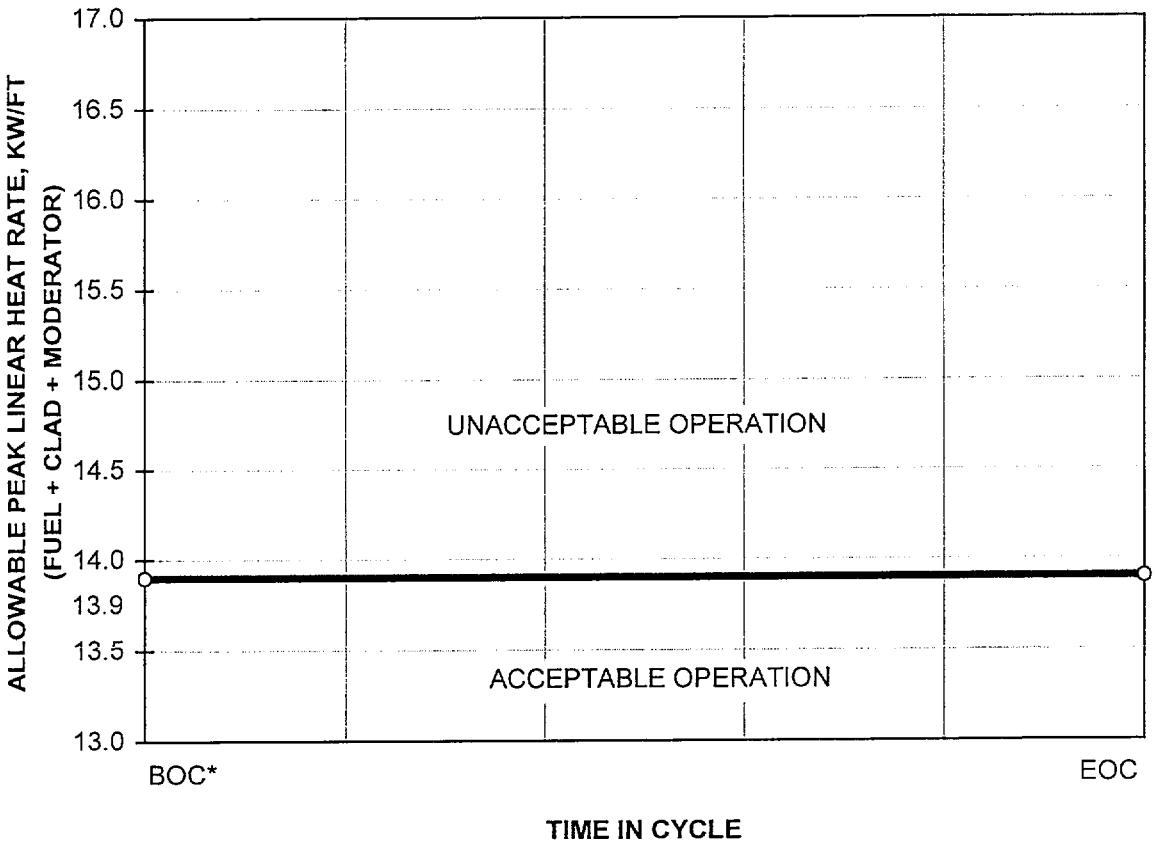

Figure 3.2.1-1

Allowable Peak Linear Heat Rate vs. Time in Cycle

***Allowable PLHR was changed from 14.3 kw/ft to 13.9 kw/ft mid-cycle**

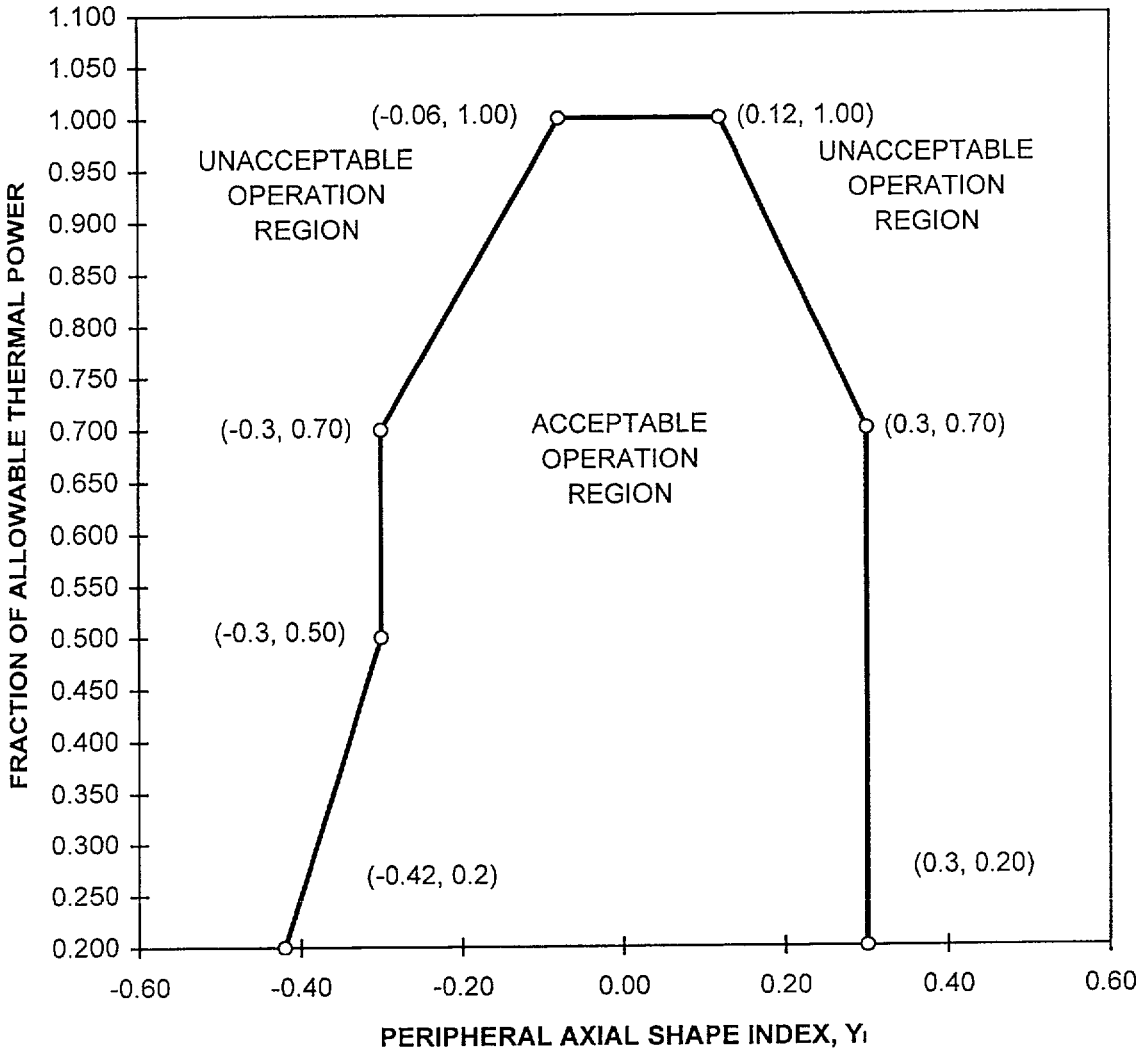

Figure 3.2.1-2

Linear Heat Rate Axial Flux Offset Control Limits

(LCO Limits are not needed below 20% thermal power)

(See NEOP-13 for Administrative Limits)

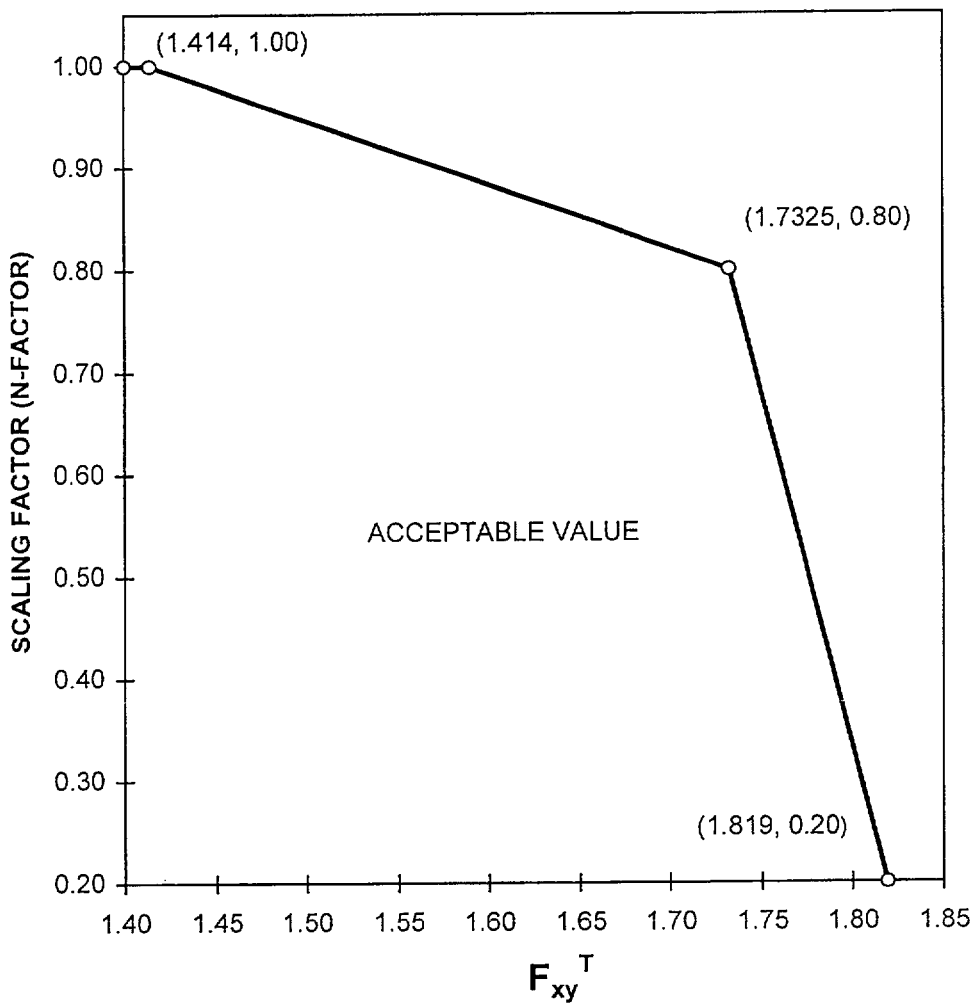

Figure 3.2.1-3

Total Planar Radial Peaking Factor (F_{xy}^T) vs. Scaling Factor (N-Factor)

(See NEOP-13 for Administrative Limits)

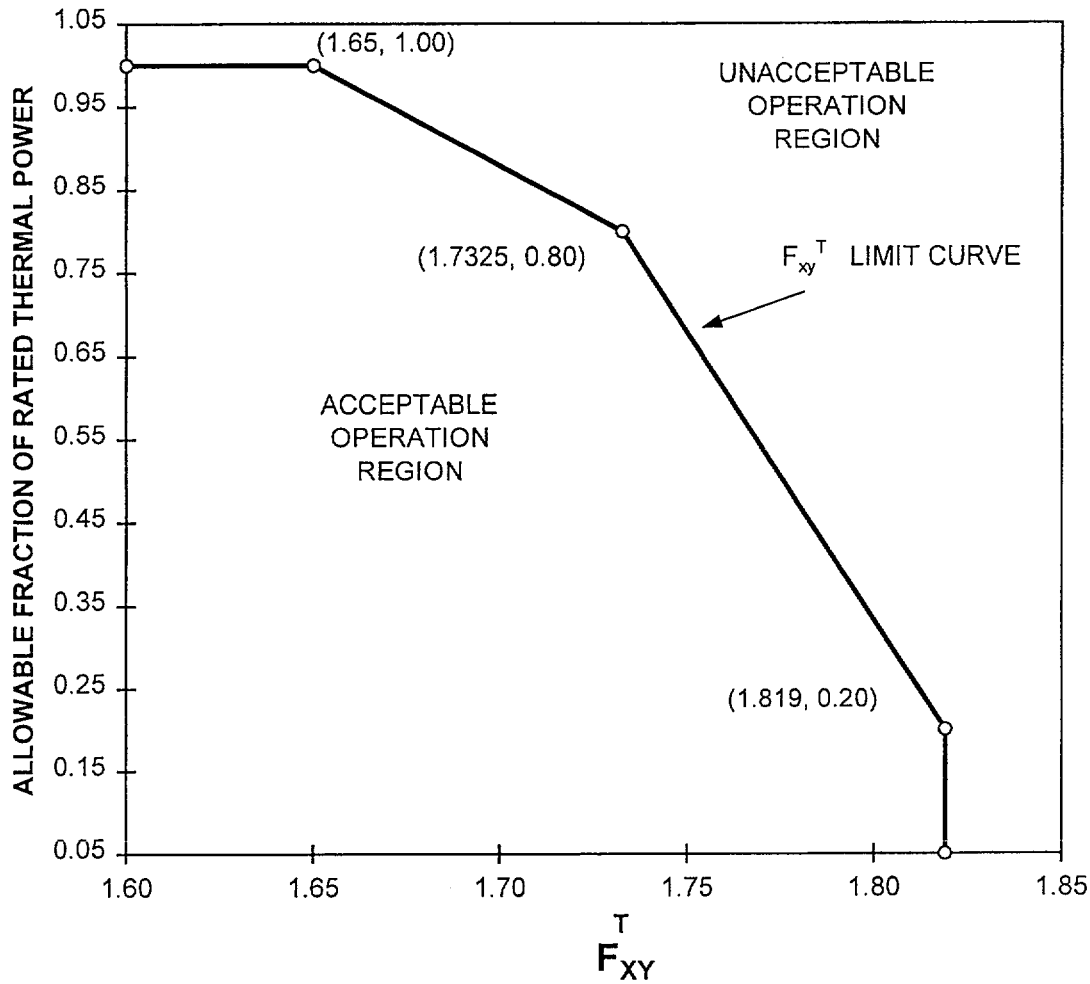

Figure 3.2.2

Total Planar Radial Peaking Factor (F_{xy}^T) vs. Allowable Fraction of Rated Thermal Power

While operating with F_{xy}^T greater than 1.65, withdraw CEAs to or above the Long Term Steady State Insertion Limits (Figure 3.1.6)

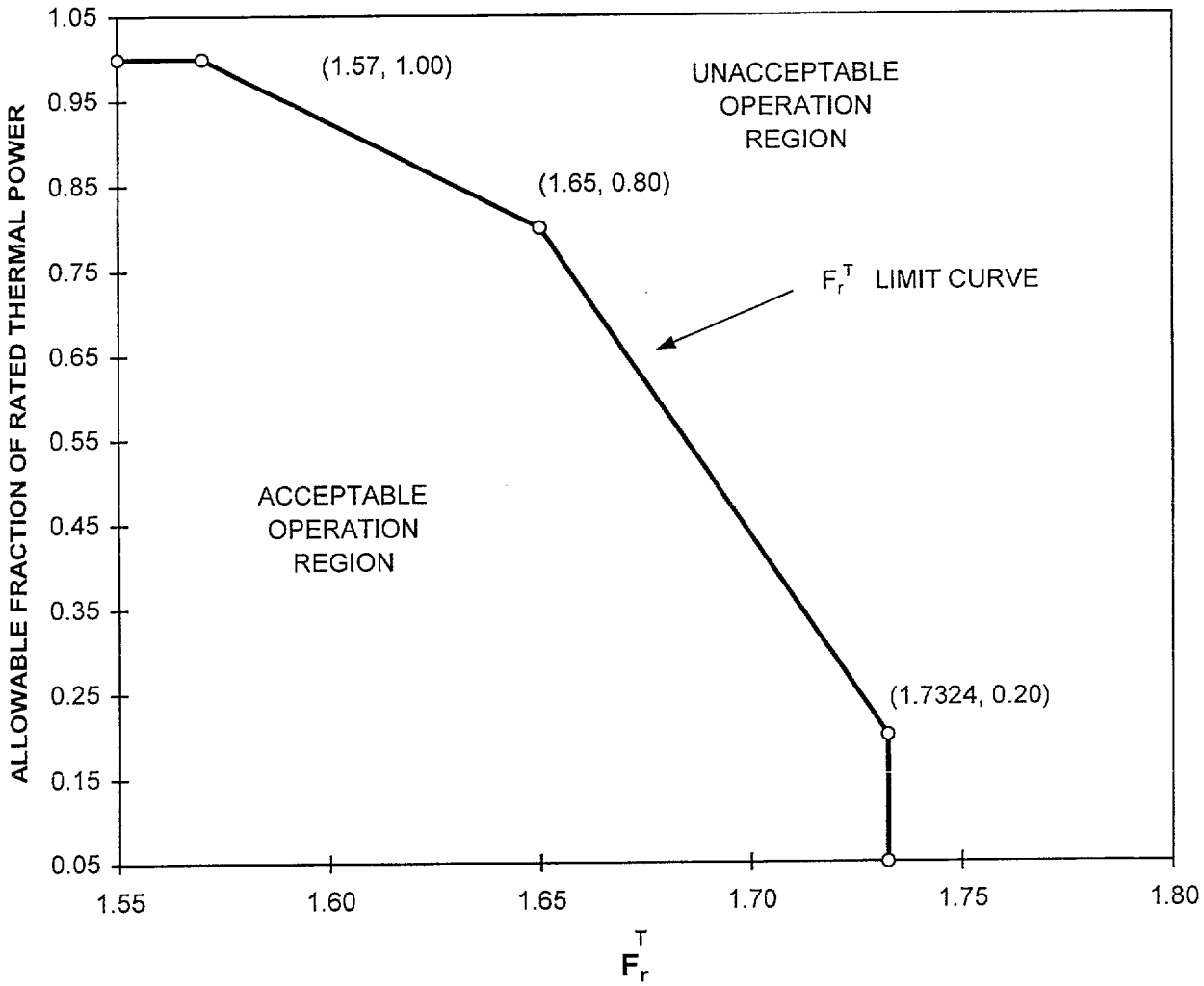

Figure 3.2.3

Total Integrated Radial Peaking Factor (F_r^T) vs. Allowable Fraction of Rated Thermal Power

While operating with F_r^T greater than 1.57, withdraw CEAs to or above the Long Term Steady State Insertion Limits (Figure 3.1.6)

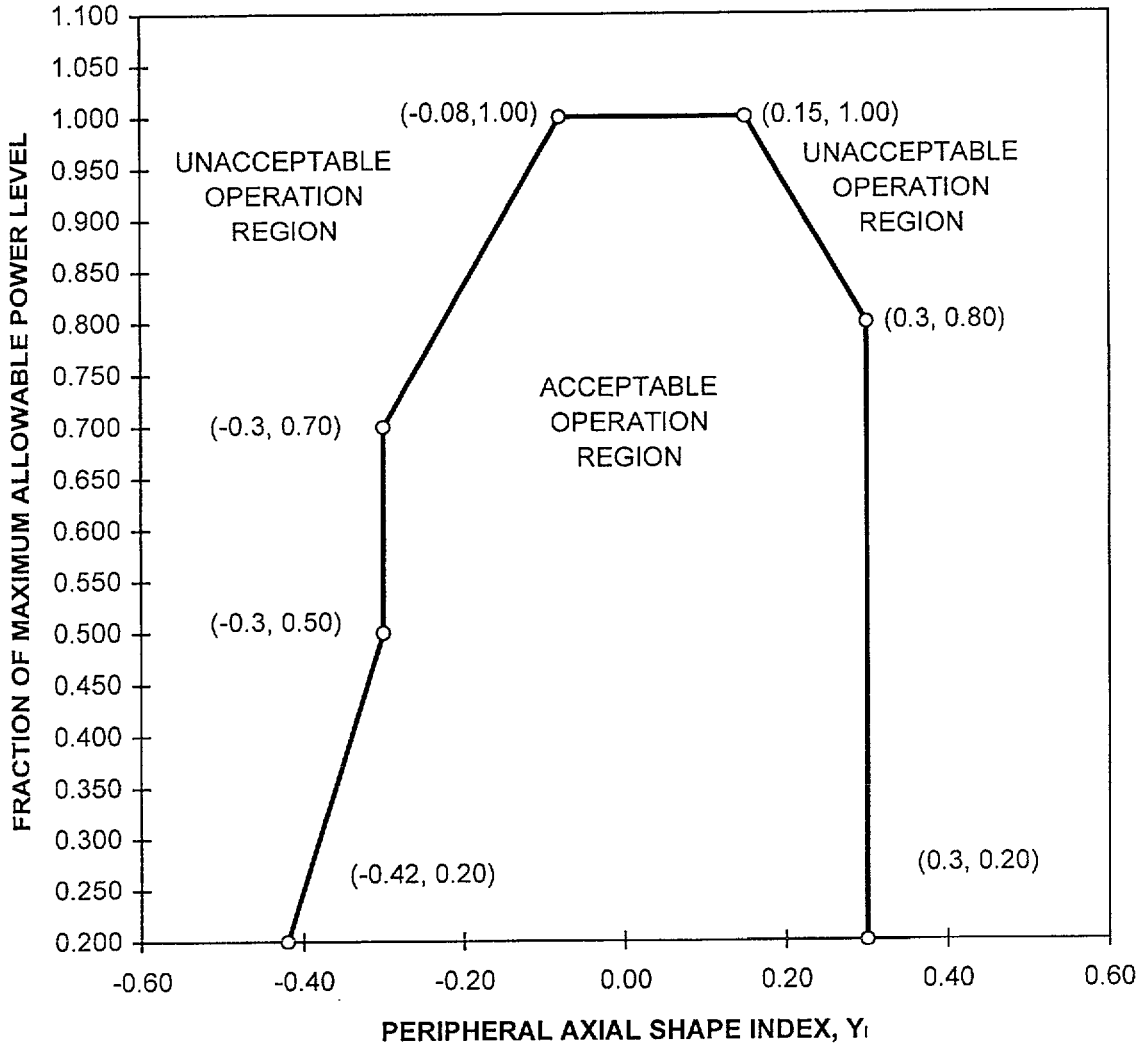

Figure 3.2.5

DNB Axial Flux Offset Control Limits

(See NEOP-13 for Administrative Limits)

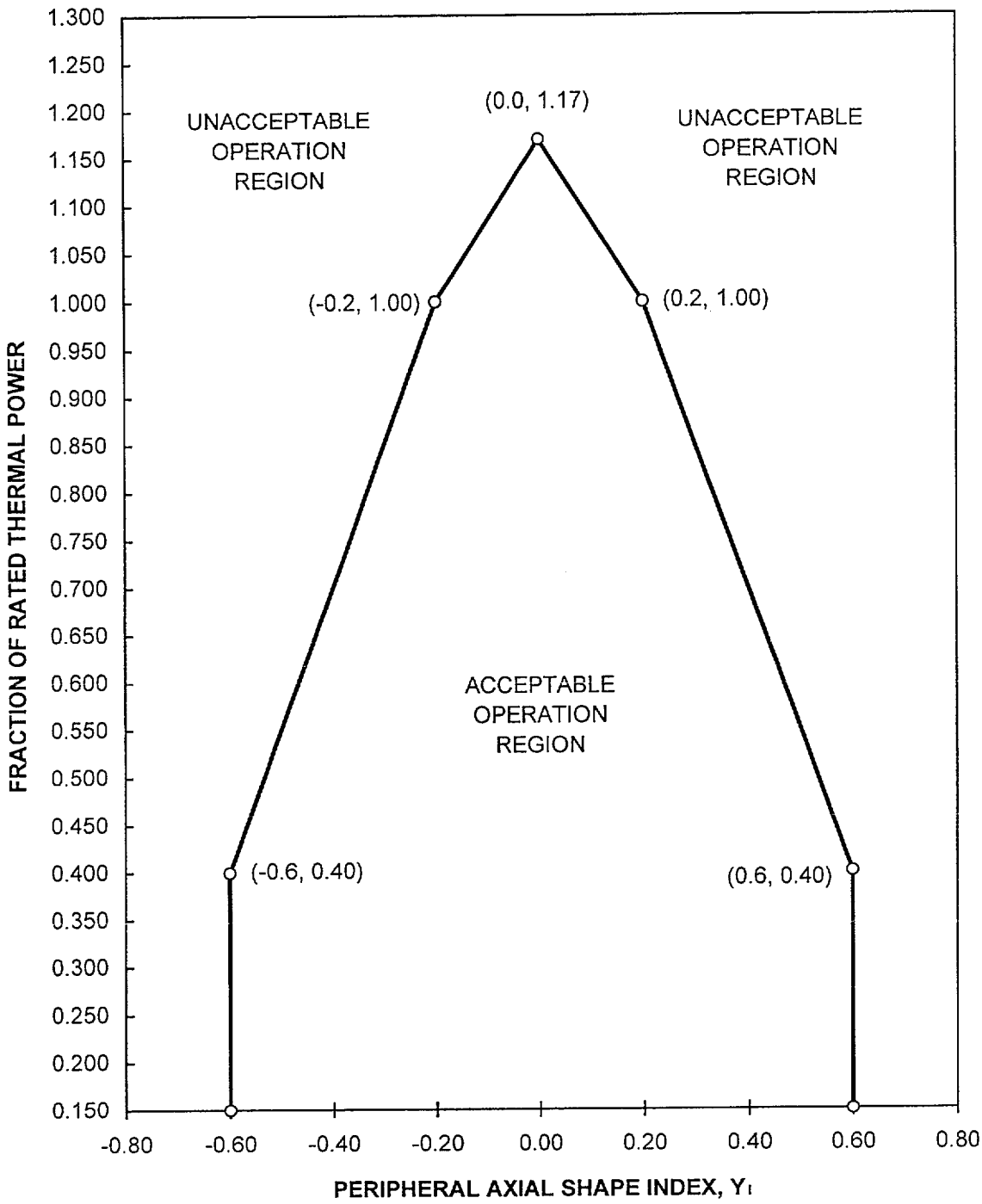

Figure 3.3.1-1

Axial Power Distribution - High Trip Setpoint
Peripheral Axial Shape Index vs. Fraction of Rated Thermal Power

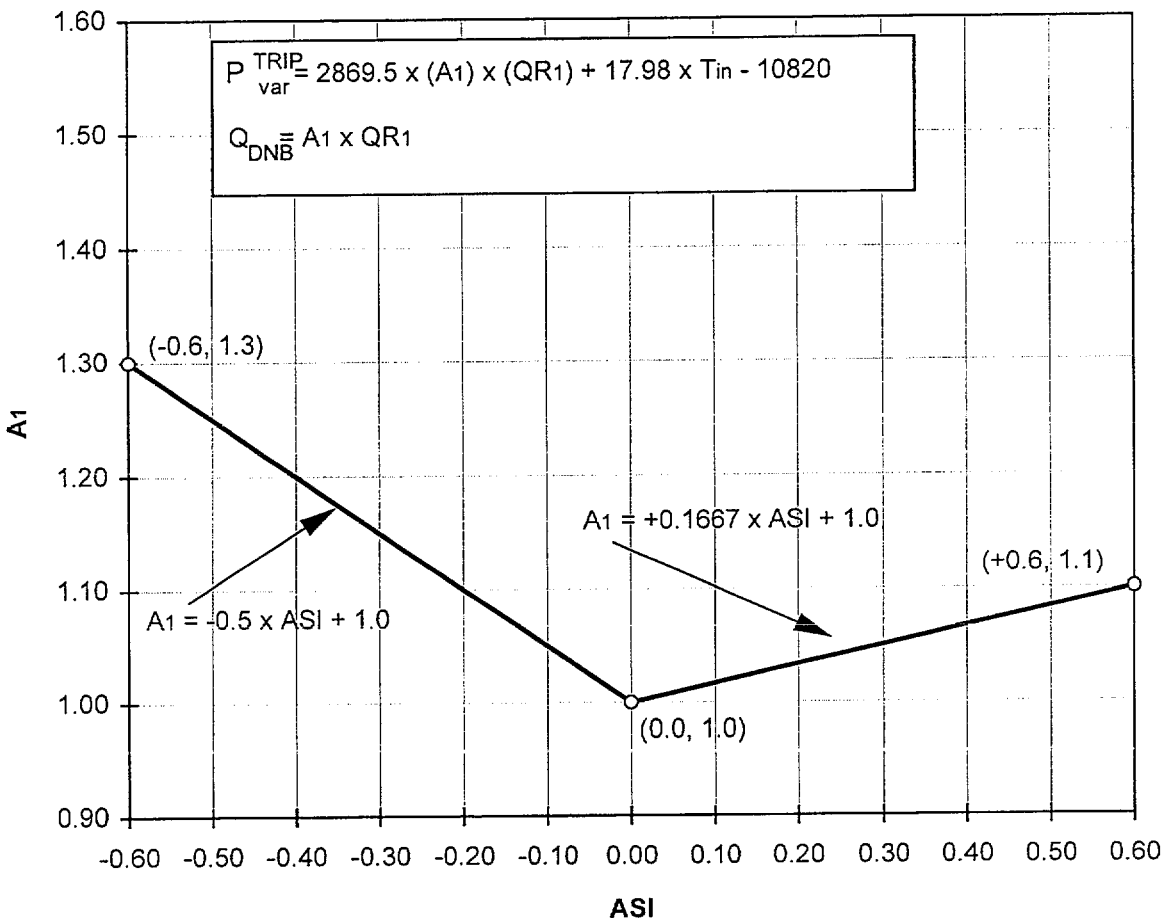

Figure 3.3.1-2

Thermal Margin/Low Pressure Trip Setpoint - Part 1
(ASI vs. A_1)

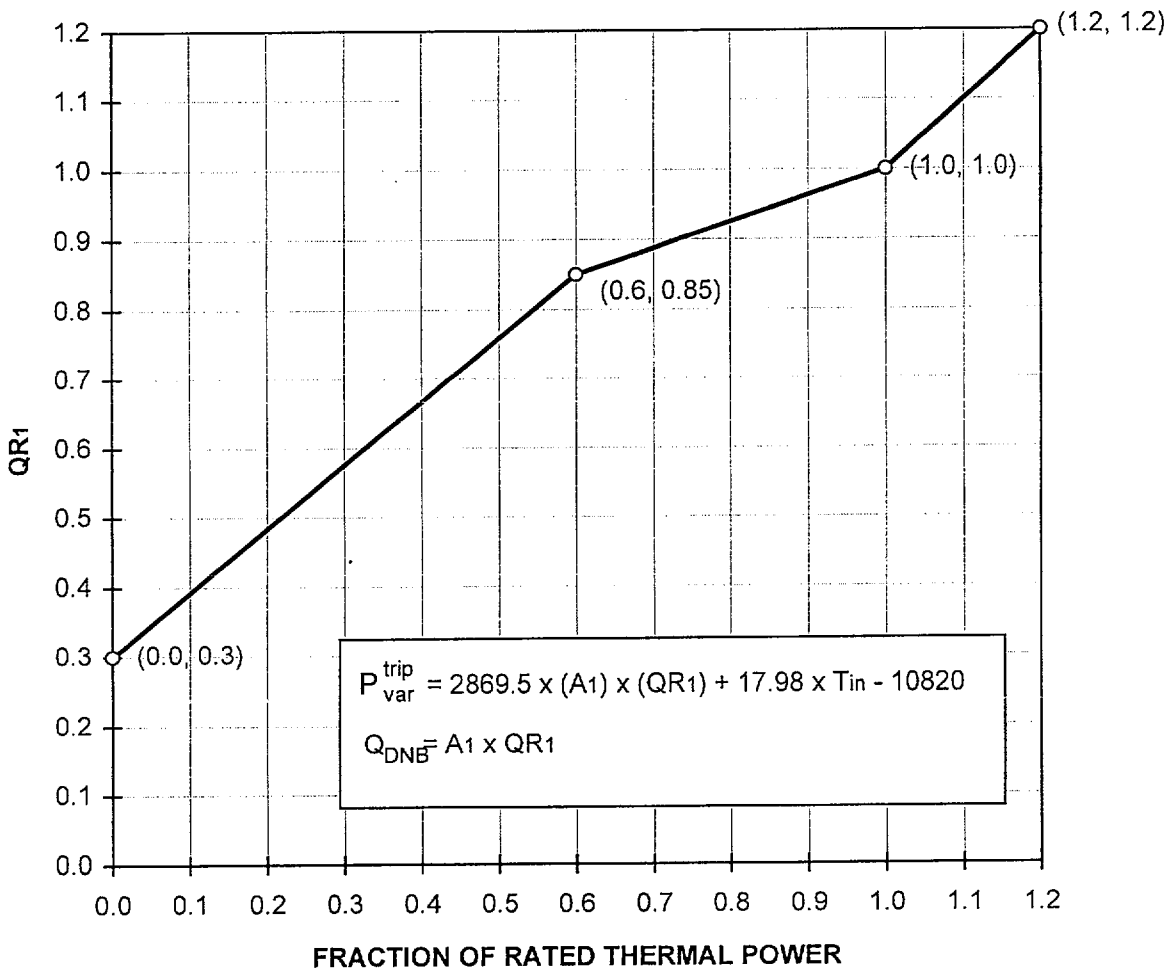

Figure 3.3.1-3

Thermal Margin/Low Pressure Trip Setpoint - Part 2
(Fraction of Rated Thermal Power vs. QR_1)

LIST OF APPROVED METHODOLOGIES

(1) CENPD-199-P, Rev 1-P-A, "C-E Setpoint Methodology: C-E Local Power Density and DNB LSSS and LCO Setpoint Methodology for Analog Protection Systems," January 1986 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.1, 3.2.2, 3.2.3, 3.2.5)

(2) CEN-124(B)-P, "Statistical Combination of Uncertainties Methodology Part 1: C-E Calculated Local Power Density and Thermal Margin/Low Pressure LSSS for Calvert Cliffs Units I and II," December 1979 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.2, 3.2.3)

(3) CEN-124(B)-P, "Statistical Combination of Uncertainties Methodology Part 2: Combination of System Parameter Uncertainties in Thermal Margin Analyses for Calvert Cliffs Units 1 and 2," January 1980 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.3, 3.2.5)

(4) CEN-124(B)-P, "Statistical Combination of Uncertainties Methodology Part 3: C-E Calculated Departure from Nucleate Boiling and Linear Heat Rate Limiting Conditions for Operation for Calvert Cliffs Units 1 and 2," March 1980 (Methodology for Specifications 3.1.6, 3.2.1, 3.2.2, 3.2.3, 3.2.5)

(5) CEN-191(B)-P, "CETOP-D Code Structure and Modeling Methods for Calvert Cliffs Units 1 and 2," December 1981 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.3, 3.2.5)

(6) Letter from Mr. D. H. Jaffe (NRC) to Mr. A. E. Lundvall, Jr. (BG&E), dated June 24, 1982, Unit 1 Cycle 6 License Approval (Amendment No. 71 to DPR-53 and SER) [Approval to CEN-124(B)-P (three parts) and CEN-191(B)-P)]

(7) CEN-348(B)-P, "Extended Statistical Combination of Uncertainties," January 1987 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.3, 3.2.5)

(8) Letter from Mr. S. A. McNeil, Jr. (NRC) to Mr. J. A. Tiernan (BG&E), dated October 21, 1987, Docket Nos. 50-317 and 50-318, "Safety Evaluation of Topical Report CEN-348(B)-P, Extended Statistical Combination of Uncertainties"

(9) CENPD-161-P-A, "TORC Code, A Computer Code for Determining the Thermal Margin of a Reactor Core," April 1986 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.3, 3.2.5)

(10) CENPD-162-P-A, "Critical Heat Flux Correlation of C-E Fuel Assemblies with Standard Spacer Grids Part 1, Uniform Axial Power Distribution," April 1975 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.3, 3.2.5)

(11) CENPD-207-P-A, "Critical Heat Flux Correlation for C-E Fuel Assemblies with Standard Spacer Grids Part 2, Non-Uniform Axial Power Distribution," December 1984 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.3, 3.2.5)

(12) CENPD-206-P-A, "TORC Code, Verification and Simplified Modeling Methods," June 1981 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.3, 3.2.5)

(13) CENPD-225-P-A, "Fuel and Poison Rod Bowing," June 1983 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.3, 3.2.5)

(14) CENPD-266-P-A, "The ROCS and DIT Computer Code for Nuclear Design," April 1983 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.1, 3.2.2, 3.2.3, 3.2.5)

(15) CENPD-275-P-A, "C-E Methodology for Core Designs Containing Gadolinia - Urania Burnable Absorbers," May 1988 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.1, 3.2.2, 3.2.3, 3.2.5)

(16) CENPD-382-P-A, "Methodology for Core Designs Containing Erbium Burnable Absorbers," August 1993 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.1, 3.2.2, 3.2.3, 3.2.5)

(17) CENPD-139-P-A, "C-E Fuel Evaluation Model Topical Report," July 1974 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.1, 3.2.2)

(18) CEN-161-(B)-P-A, "Improvements to Fuel Evaluation Model," August 1989 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.1, 3.2.2)

(19) CEN-161-(B)-P, Supplement 1-P, "Improvements to Fuel Evaluation Model," April 1986 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.1, 3.2.2)

(20) Letter from Mr. S. A. McNeil, Jr. (NRC) to Mr. J. A. Tiernan (BG&E), dated February 4, 1987, Docket Nos. 50-317 and 50-318, "Safety Evaluation of Topical Report CEN-161-(B)-P, Supplement 1-P, Improvements to Fuel Evaluation Model" (Approval of CEN-161(B), Supplement 1-P)

(21) CEN-372-P-A, "Fuel Rod Maximum Allowable Gas Pressure," May 1990 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.1, 3.2.2)

(22) Letter from Mr. A. E. Scherer (CE) to Mr. J. R. Miller (NRC) dated December 15, 1981, LD-81-095, Enclosure 1-P, "C-E ECCS Evaluation Model Flow Blockage Analysis" (Methodology for Specifications 3.2.1, 3.2.2)

(23) CENPD-132, Supplement 3-P-A, "Calculative Methods for the C-E Large Break LOCA Evaluation Model for the Analysis of C-E and W Designed NSSS," June 1985 (Methodology for Specifications 3.2.1, 3.2.2 and approval of Letter LD-81-095, dated December 15, 1981)

(24) CENPD-133, Supplement 5, "CEFLASH-4A, a FORTRAN77 Digital Computer Program for Reactor Blowdown Analysis," June 1985 (Methodology for Specifications 3.2.1, 3.2.2)

(25) CENPD-134, Supplement 2, "COMPERC-II, a Program for Emergency Refill-Reflood of the Core," June 1985 (Methodology for Specifications 3.2.1, 3.2.2)

(26) Letter from Mr. D. M. Crutchfield (NRC) to Mr. A. E. Scherer (CE), dated July 31, 1986, "Safety Evaluation of Combustion Engineering ECCS Large Break Evaluation Model and Acceptance for Referencing of Related Licensing Topical Reports (Approval of CENPD-133, Supplement 5 and CENPD-134, Supplement 2)

(27) CENPD-135, Supplement 5-P, "STRIKIN-II, A Cylindrical Geometry Fuel Rod Heat Transfer Program," April 1977 (Methodology for Specifications 3.2.1, 3.2.2)

(28) Letter from Mr. R. L. Baer (NRC) to Mr. A. E. Scherer (CE) dated September 6, 1978, "Evaluation of Topical Report CENPD-135, Supplement 5"

(29) CENPD-137, Supplement 1-P, "Calculative Methods for the C-E Small Break LOCA Evaluation Model," January 1977 (Methodology for Specifications 3.2.1, 3.2.2)

(30) CENPD-133, Supplement 3-P, "CEFLASH-4AS, "A Computer Program for the Reactor Blowdown Analysis of the Small Break Loss of Coolant Accident," January 1977 (Methodology for Specifications 3.2.1, 3.2.2)

(31) Letter from Mr. K. Kniel (NRC) to Mr. A. E. Scherer (CE), dated September 27, 1977, "Evaluation of Topical Reports CENPD-133, Supplement 3-P and CENPD-137, Supplement 1-P"

(32) CENPD-138, Supplement 2-P, "PARCH, A FORTRAN-IV Digital Program to Evaluate Pool Boiling, Axial Rod and Coolant Heatup," January 1977 (Methodology for Specifications 3.2.1, 3.2.2)

(33) Letter from Mr. C. Aniel (NRC) to Mr. A. E. Scherer, dated April 10, 1978. "Evaluation of Topical Report CENPD-138, Supplement 2-P"

(34) Letter from Mr. A. E. Lundvall, Jr. (BG&E) to Mr. J. R. Miller (NRC) dated February 22, 1985, "Calvert Cliffs Nuclear Power Plant Unit 1; Docket No. 50-317, Amendment to Operating License DPR-53, Eighth Cycle License Application" (Section 7.3.2 contains Methodology for Specifications 3.1.1 and 3.1.3 and 3.1.6)

(35) Letter from Mr. D. H. Jaffe (NRC) to Mr. A. E. Lundvall, Jr. (BG&E), dated May 20, 1985, "Safety Evaluation Report Approving Unit 1 Cycle 8 License Application"

(36) Letter from Mr. A. E. Lundvall, Jr. (BG&E) to Mr. R. A. Clark (NRC), dated September 22, 1980, "Amendment to Operating License No. 50-317, Fifth Cycle License Application" (Section 7.1.2 contains Methodology for Specifications 3.1.1, 3.9.1)

(37) Letter from Mr. R. A. Clark (NRC) to Mr. A. E. Lundvall, Jr. (BG&E), dated December 12, 1980, "Safety Evaluation Report Approving Unit 1, Cycle 5 License Application"

(38) Letter from Mr. J. A. Tiernan (BG&E) to Mr. A. C. Thadani (NRC), dated October 1, 1986, "Calvert Cliffs Nuclear Power Plant Unit Nos. 1 & 2, Docket Nos. 50-317 & 50-318, Request for Amendment" (Methodology for Specifications 3.1.4)

(39) Letter from S. A. McNeil, Jr. (NRC) to Mr. J. A. Tiernan (BG&E), dated July 7, 1987, Docket Nos. 50-317 and 50-318, Approval of Amendments 127 (Unit 1) and 109 (Unit 2) (Support for Specification 3.1.4)

(40) CENPD-188-A, "HERMITE: A Multi-Dimensional Space-Time Kinetics Code for PWR Transients," July 1976 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.1, 3.2.2, 3.2.3, 3.2.5)

(41) The power distribution monitoring system referenced in various specifications and the BASES, is described in the following documents:

i. CENPD-153-P, Revision 1-P-A, "Evaluation of Uncertainty in the Nuclear Power Peaking Measured by the Self-Powered, Fixed Incore Detector System," May 1980 (Methodology for Specifications 3.3.1, 3.1.6, 3.2.1, 3.2.2, 3.2.3, 3.2.5)

ii. CEN-119(B)-P, "BASSS, Use of the Incore Detector System to Monitor the DNB-LCO on Calvert Cliffs Unit 1 and Unit 2," November 1979 (Referenced in Appendix B of Unit 2 Cycle 9 License Application)

iii. Letter from Mr. G. C. Creel (BG&E) to NRC Document Control Desk, dated February 7, 1989, "Calvert Cliffs Nuclear Power Plant Unit No. 2; Docket No. 50-318, Request for Amendment, Unit 2 Ninth Cycle License Application" (Appendix B contains Methodologies for Specifications 3.1.4, 3.2.2, 3.2.3, 3.2.5)

iv. Letter from Mr. S. A. McNeil, Jr. (NRC) to Mr. G. C. Creel (BG&E), dated January 10, 1990, "Safety Evaluation Report Approving Unit 2 Cycle 9 License Application"

(42) Letter from Mr. D. G. McDonald, Jr. (NRC) to Mr. R. E. Denton (BGE), dated May 11, 1995, "Approval to Use Convolution Technique in Main Steam Line Break Analysis - Calvert Cliffs Nuclear Power Plant, Unit Nos. 1 and 2 (TAC Nos. M90897 and M90898)" (Methodology for Specification 3.2.3).

(43) CENPD-387-P-A, Latest Approved Revision, "ABB Critical Heat Flux Correlations for PWR Fuel".

(44) CENPD-199-P, Supplement 2-P-A, Appendix A, Latest Approved Revision, "CE Setpoint Methodology, " June 1998.