

December 10, 1992

Docket No. 50-261

Mr. R. A. Watson
Senior Vice President
Nuclear Generation
Carolina Power & Light Company
Post Office Box 1551
Raleigh, North Carolina 27602

Dear Mr. Watson:

SUBJECT: ISSUANCE OF AMENDMENT NO. 143 TO FACILITY OPERATING LICENSE NO. DPR-23 REGARDING CALIBRATION OF CONTAINMENT HIGH-RANGE RADIATION MONITOR - H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2, (TAC NO. M83714)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 143 to Facility Operating License No. DPR-23 for the H. B. Robinson Steam Electric Plant, Unit No. 2. This amendment consists of changes to the Technical Specifications in response to your request dated June 5, 1992.

The amendment clarifies Item 44 of Technical Specifications Table 4.1-1, which specifies the minimum frequencies for checks, calibrations and tests of the containment vessel high-range monitors (R-32A and B) by revising the footnote so that the calibration method is clearly identified as an acceptable alternative to the NRC-preferred calibration technique described in NUREG-0737, but which does not preclude the use of the NUREG-0737 methodology.

A copy of the related Safety Evaluation is enclosed. Notice of Issuance will be included in the Commission's bi-weekly Federal Register notice.

Sincerely,

ORIGINAL SIGNED BY:

Brenda L. Mozafari, Project Manager
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 143 to DPR-23
2. Safety Evaluation

cc w/enclosures: 150030
See next page

OFC	LA:PD21:DRPE	PM:PD21:DRPE	D:PD21:DRPE		OGC
NAME	Little	B Mozafari:tms	E Adamsam	L Cunningham	S. How
DATE	11/30/92	11/19/92	12/10/92	11/23/92	11/7/92

Document Name: ROB83714.AMD

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Mr. R. A. Watson
Carolina Power & Light Company

H. B. Robinson Steam Electric
Plant, Unit No. 2

cc:

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AMENDMENT NO. 143 TO FACILITY OPERATING LICENSE NO. DPR-23
H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2

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Docket File

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cc: Robinson Service List

ATTACHMENT TO LICENSE AMENDMENT NO. 143

FACILITY OPERATING LICENSE NO. DPR-23

DOCKET NO. 50-261

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Remove Pages

4.1-9a

Insert Pages

4.1-9a

TABLE 4.1-1 (Continued)

48.	Reactor Vessel Level Instrumentation System (RVLIS)	M	R	N.A.	
49.	Incore Thermocouple Temperature Instrumentation	M	R	N.A.	
+	Containment Water Level Monitor - NUREG-0737 Item II.F.1.5				
++	Containment Pressure Monitor - NUREG-0737 Item II.F.1.4				
+++	Containment Hydrogen Monitor - NUREG-0737 Item II.F.1.6				
++++	Containment High-Range Radiation Monitor - NUREG-0737 Item II.F.1.3				
#	CP&L's letter dated April 28, 1982, S. R. Zimmerman to S. A. Varga, provides an acceptable alternate calibration methodology.				
S	-	At least once per 12 hours	Q	-	At least once per 92 days
D	-	At least once per 24 hours	S/U	-	Prior to each reactor startup if not performed in the previous seven (7) days
W	-	At least once per 7 days	R	-	At least once per 18 months
B/W	-	At least once per 14 days	N.A.	-	Not applicable
M	-	At least once per 31 days			

4.1-9a



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

CAROLINA POWER & LIGHT COMPANY

DOCKET NO. 50-261

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 143
License No. DPR-23

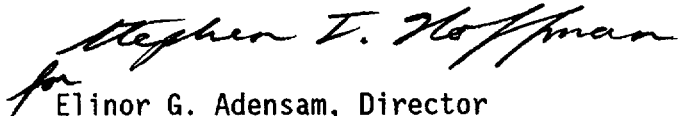
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Carolina Power & Light Company (the licensee), dated June 5, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 3.B. of Facility Operating License No. DPR-23 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 143 , are hereby incorporated in the license. Carolina Power & Light Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



for
Elinor G. Adensam, Director
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 10, 1992



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 143 TO FACILITY OPERATING LICENSE NO. DPR-23
CAROLINA POWER & LIGHT COMPANY
H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2
DOCKET NO. 50-261

1.0 INTRODUCTION

By letter dated June 5, 1992, the Carolina Power & Light Company (licensee) submitted a request for changes to the H. B. Robinson Steam Electric Plant, Unit No. 2, Technical Specifications (TS). The requested changes would clarify a footnote associated with calibration of the containment vessel (CV) high-range radiation monitors R-32A and R-32B.

The proposed amendment would change a footnote to TS Table 4.1-1 associated with the refueling interval calibration of the CV high-range radiation monitors. The current footnote limits the licensee to an alternative calibration methodology as described in the licensee's letter dated April 28, 1982. The change would allow the licensee the flexibility to use the special calibration methodology provided in the NRC's guidance of NUREG-0737, Item II.F.1, Attachment 3, and associated Table II.F.1-3.

2.0 EVALUATION

On November 1, 1983, the NRC issued Generic Letter (GL) 83-37, "NUREG-0737 Technical Specifications," that provided guidance and model TS for the containment high-range radiation monitors. The NRC-preferred calibration methodology involves in situ calibration. In response to the GL, by letter dated February 7, 1984, the licensee submitted proposed TS for the CV high-range monitor. The licensee's proposed TS met the criteria of Table II.F.1-3 of NUREG-0737 with the exception that the licensee would perform the calibration of the instruments in the plant's test lab rather than in the installed location. The footnote to Item 44 of TS Table 4.1-1 referenced the alternative methodology provided by the licensee on April 28, 1982. That letter was in response to an NRC request that the licensee ensure that the entire channel was functioning after re-installation following calibration at the test facility. In the safety evaluation related to Amendment No. 94 issued on August 29, 1985, the NRC approved the licensee's TS, including the exception to perform the calibration in accordance with the April 28, 1982, letter.

Recently the licensee became aware that the literal reading of the TS footnote required that the CV high-range monitors be calibrated in accordance with the April 28, 1982, letter. Although the NRC-preferred calibration method is that described in NUREG-0737, either calibration method will ensure the operability

and availability of the CV high-range monitors. The licensee has proposed wording which clearly reflects the acceptability of the approved alternative calibration while allowing the option to use the NRC-preferred in situ calibration. The NRC finds the revised footnote to be acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the State of South Carolina official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes the surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (57 FR 28196). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: B. Mozafari

Date: December 10, 1992