

September 12, 1996

Mr. C. S. Hinnant, Vice President  
Carolina Power & Light Company  
H. B. Robinson Steam Electric Plant  
Unit No. 2  
3581 West Entrance Road  
Hartsville, South Carolina 29550

SUBJECT: ISSUANCE OF AMENDMENT NO. 175 TO FACILITY OPERATING LICENSE NO. DPR-23 REGARDING REACTOR TRIP AND ENGINEERED SAFETY FEATURE ACTUATION SYSTEMS INSTRUMENTATION - H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2 (TAC NO. M94318)

Dear Mr. Hinnant:

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 175 to Facility Operating License No. DPR-23 for the H. B. Robinson Steam Electric Plant, Unit No. 2 (HBR). This amendment changes the HBR Technical Specifications (TS) in response to your request dated December 10, 1995, as supplemented August 1, 1996, and September 4, 1996.

The amendment revises TS Section 3.5.1 and Tables 3.5-2, 3, and 4 concerning the reactor trip system, engineered safety feature actuation system, and isolation function. The TS would be revised to (1) specify actions to be taken when an instrument channel becomes inoperable, (2) add an "Applicable Conditions" column that defines the applicability and/or mode of operation of each functional unit, and (3) make editorial enhancements.

A copy of the related Safety Evaluation is enclosed. Notice of Issuance will be included in the Commission's bi-weekly Federal Register notice.

Sincerely,  
Original signed by:

Brenda L. Mozafari, Project Manager  
Project Directorate II-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket No. 50-261

Enclosures:

- 1. Amendment No. 175 to DPR-23
- 2. Safety Evaluation

cc w/enclosures:

See next page

FILENAME - G:\ROBINSON\ROB94318.AMD \*see previous concurrence

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Mr. C. S. Hinnant  
Carolina Power & Light Company

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Plant, Unit No. 2

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3581 West Entrance Road  
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AMENDMENT NO. 175 TO FACILITY OPERATING LICENSE NO. DPR-23 - H. B. ROBINSON  
STEAM ELECTRIC PLANT, UNIT NO. 2

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

CAROLINA POWER & LIGHT COMPANY

DOCKET NO. 50-261

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 175  
License No. DPR-23

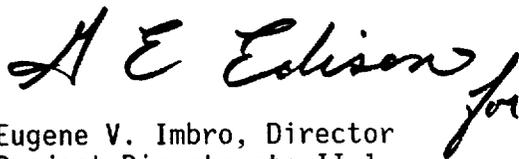
1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Carolina Power & Light Company (the licensee), dated December 10, 1995, as supplemented August 1, 1996, and September 4, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 3.B. of Facility Operating License No. DPR-23 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 175, are hereby incorporated in the license. Carolina Power & Light Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 90 days.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in black ink that reads "Eugene V. Imbro" with a stylized flourish at the end.

Eugene V. Imbro, Director  
Project Directorate II-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: September 12, 1996

ATTACHMENT TO LICENSE AMENDMENT NO. 175

FACILITY OPERATING LICENSE NO. DPR-23

DOCKET NO. 50-261

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

<u>Remove Pages</u>	<u>Insert Pages</u>
3.5-1	3.5-1
3.5-12	3.5-12
3.5-13	3.5-13
---	3.5-13a
---	3.5-13b
---	3.5-13c
3.5-14	3.5-14
3.5-15	3.5-15
---	3.5-15a
3.5-16	3.5-16
3.5-17	3.5-17
---	3.5-17a

## 3.5 INSTRUMENTATION SYSTEMS

### 3.5.1 Operational Safety Instrumentation

#### Applicability

Applies to plant operational safety instrumentation systems.

#### Objective

To provide for automatic initiation of the Engineered Safety Features in the event that principal process variable limits are exceeded, and to delineate the conditions of the plant instrumentation and safety circuits necessary to ensure reactor safety.

#### Specification

- 3.5.1.1 The Engineered Safety Features initiation instrumentation setting limits shall be as stated in Table 3.5-1.
- 3.5.1.2 For on-line testing or in the event of a subsystem instrumentation channel failure, plant operation at rated power shall be permitted to continue in accordance with Tables 3.5-2 through 3.5-5.
- 3.5.1.3 In the event the number of channels in service listed in Table 3.5-5 falls below the limits given in the column entitled Minimum Channels Operable, operation shall be limited according to the requirement shown in Column 2.
- 3.5.1.4 The containment ventilation isolation function is only required when containment integrity is required.
- 3.5.1.5 In the event the number of operable channels of a particular functional unit listed in Tables 3.5-2, 3, or 4 falls below the limits given in the column entitled Total Number of Channels, operation shall be limited according to the requirement shown in Column 3.

TABLE 3.5-2

REACTOR TRIP INSTRUMENTATION LIMITING OPERATING CONDITIONS

NO.	FUNCTIONAL UNIT	1	2	3	APPLICABLE CONDITIONS
		TOTAL NO. OF CHANNELS	MINIMUM CHANNELS OPERABLE	OPERATOR ACTION IF COLUMN 1 OR 2 CANNOT BE MET	
1.	Manual	2	2	ACTION 1	Reactor Critical Hot/Cold Shutdown *
		2	2	ACTION 8	
2.	Nuclear Flux Power Range				
	A. High Setpoint	4	3	ACTION 2	Reactor Critical
	B. Low Setpoint	4	3	ACTION 2	Reactor Critical **
3.	Nuclear Flux Intermediate Range	2	2	ACTION 3	Reactor Critical **
4.	Nuclear Flux Source Range				
	A. Startup	2	2	ACTION 4	Reactor Critical ***
	B. Shutdown	2	1	ACTION 5	Hot/Cold Shutdown
	C. Shutdown	2	2	ACTION 8	Hot/Cold Shutdown *
5.	Overtemperature $\Delta T$	3	2	ACTION 6	Reactor Critical
6.	Overpower $\Delta T$	3	2	ACTION 6	Reactor Critical
7.	Low Pressurizer Pressure	3	2	ACTION 6	*****
8.	Hi Pressurizer Pressure	3	2	ACTION 6	Reactor Critical
9.	Pressurizer Hi Water Level	3	2	ACTION 6	*****
10.	Low Reactor Coolant Flow				
	A. Single Loop	3/loop	2/loop	ACTION 6	$\geq 45\%$ of rated power
	B. Two Loop	3/loop	2/loop	ACTION 6	****

TABLE 3.5-2 (Continued)

REACTOR TRIP INSTRUMENTATION LIMITING OPERATING CONDITIONS

<u>NO.</u>	<u>FUNCTIONAL UNIT</u>	<u>1</u> <u>TOTAL NO.</u> <u>OF</u> <u>CHANNELS</u>	<u>2</u> <u>MINIMUM</u> <u>CHANNELS</u> <u>OPERABLE</u>	<u>3</u> <u>OPERATOR ACTION IF</u> <u>COLUMN 1 OR 2</u> <u>CANNOT BE MET</u>	<u>APPLICABLE</u> <u>CONDITIONS</u>
11.	Turbine Trip				
	A. Auto Stop Oil Pressure	3	2	ACTION 6	*****
	B. Turb Stop Valves	2	2	ACTION 6	*****
12.	Lo Lo Steam Generator Water Level	3/SG	2/SG	ACTION 6	Reactor Critical
13.	Underfrequency 4 KV System	3	2	ACTION 6	Reactor Critical
14.	Undervoltage on 4 KV System	3	2	ACTION 7	Reactor Critical
15.	Control Rod Misalignment Monitor				
	A. ERFIS Rod Position Deviation	1	1	ACTION 9	Reactor Critical
	B. Quadrant Power Tilt Monitor (upper and lower ex-core neutron detectors) "Detector Current Comparator"	1	1	ACTION 10	>50% of rated power

TABLE 3.5-2 (Continued)

REACTOR TRIP INSTRUMENTATION LIMITING OPERATING CONDITIONS

<u>NO.</u>	<u>FUNCTIONAL UNIT</u>	<u>1</u> <u>TOTAL NO.</u> <u>OF</u> <u>CHANNELS</u>	<u>2</u> <u>MINIMUM</u> <u>CHANNELS</u> <u>OPERABLE</u>	<u>3</u> <u>OPERATOR ACTION IF</u> <u>COLUMN 1 OR 2</u> <u>CANNOT BE MET</u>	<u>APPLICABLE</u> <u>CONDITIONS</u>
16.	Low Steam Generator Level Coincident With Steam Flow/Feedwater Flow Mismatch	2 Level and 2 Stm/ Feed Flow Mismatch Per SG	1 Level and 2 Stm/ Feed Flow Mismatch Per SG <u>OR</u> 2 Level and 1 Stm/ Feed Flow Mismatch Per SG	ACTION 6	Reactor Critical

REACTOR TRIP INSTRUMENTATION LIMITING OPERATING CONDITIONSTABLE NOTATIONS

- \* With the reactor trip breakers closed.
- \*\* Below the P-10 (Low Setpoint Power Range Neutron Flux Interlock) setpoint.
- \*\*\* Below the P-6 (Intermediate Range Neutron Flux Interlock) setpoint.
- \*\*\*\* Above the P-10 (Low Setpoint Power Range Neutron Flux Interlock) setpoint or P-7 (Turbine First Stage Pressure Interlock) setpoint and below the P-8 (Low Setpoint Power Range Neutron Flux Interlock) setpoint.
- \*\*\*\*\* Above the P-10 (Low Setpoint Power Range Neutron Flux Interlock) setpoint or P-7 (Turbine First Stage Pressure Interlock) setpoint.

ACTION STATEMENTS

- ACTION 1 With the number of OPERABLE channels one less than the Minimum Channels OPERABLE requirement, restore the inoperable channel to OPERABLE status within 12 hours, or be in the Hot Shutdown Condition within the next 8 hours.
- ACTION 2 With the number of OPERABLE channels one less than the Total Number of Channels, Startup and/or Power Operation may proceed provided the following Conditions are satisfied:
- a. The inoperable channel is placed in the tripped condition within 1 hour.
  - b. Either, thermal power is restricted to less than or equal to 75% of rated power and the Power Range Neutron Flux trip setpoint is reduced to less than or equal to 85% of rated power within 4 hours; or, the Quadrant Power Tilt Ratio is monitored within 12 hours and every 12 hours thereafter, using the movable incore detectors to confirm that the normalized symmetric power distribution is consistent with the indicated Quadrant Power Tilt Ratio.
- ACTION 3 With the number of channels OPERABLE one less than the Minimum Channels OPERABLE requirement and with the thermal power level:
- a. Below the P-6 (Intermediate Range Neutron Flux Interlock) setpoints, restore the inoperable channel to OPERABLE status prior to increasing thermal power above the P-6 setpoint.
  - b. Above the P-6 (Intermediate Range Neutron Flux Interlock) setpoint but below 10% of rated power, restore the inoperable channel to OPERABLE status prior to increasing thermal power above 10% of rated power.

REACTOR TRIP INSTRUMENTATION LIMITING OPERATING CONDITIONSTABLE NOTATIONS

- ACTION 4 With the number of OPERABLE channels one less than the Minimum Channels OPERABLE requirement, suspend all operations involving positive reactivity changes.
- ACTION 5 With the number of OPERABLE channels one less than the Minimum Channels OPERABLE requirement, verify compliance with Shutdown Margin within 1 hour and at least once per 12 hours thereafter.
- ACTION 6 With the number of OPERABLE channels one less than the Total Number of Channels, Startup and/or Power Operation may proceed until performance of the next required operational test provided the inoperable channel is placed into the tripped condition within 1 hour.
- ACTION 7 With the number of OPERABLE channels one less than the Total Number of Channels, place the inoperable channel into the tripped condition within 1 hour, and restore the inoperable channel to OPERABLE status within 7 days or be in at least the Hot Shutdown Condition within the next 8 hours.
- ACTION 8 With the number of OPERABLE channels one less than the Minimum Channels OPERABLE requirement, restore the inoperable channel to OPERABLE status within 48 hours or open the Reactor Trip Breakers within the next hour.
- ACTION 9 Log individual rod position within 1 hour and every hour thereafter, and following load changes of >10% of rated power, or after >30 inches of control rod motion. In addition to the above ACTIONS, if both rod misalignment monitors (15.A and 15.B) are inoperable with reactor power >50% of rated power for 2 hours or more, the nuclear overpower trip shall be reset to  $\leq 93\%$  of rated power.
- ACTION 10 Log individual upper and lower ion chamber currents within 1 hour and every hour thereafter, and following load changes of >10% of rated power, or after >30 inches of control rod motion. In addition to the above ACTIONS, if both rod misalignment monitors (15.A and 15.B) are inoperable with reactor power >50% of rated power for two hours or more, the nuclear overpower trip shall be reset to  $\leq 93$  percent of rated power.

TABLE 3.5-3

ENGINEERED SAFETY FEATURES INSTRUMENTATION LIMITING OPERATING CONDITIONS

<u>NO.</u>	<u>FUNCTIONAL UNIT</u>	<u>1 TOTAL NO. OF CHANNELS</u>	<u>2 MINIMUM CHANNELS OPERABLE</u>	<u>3 OPERABLE ACTION IF COLUMN 1 OR 2 CANNOT BE MET</u>	<u>APPLICABLE CONDITIONS</u>
1.	SAFETY INJECTION				
A.	Manual	2	2	ACTION 11	>200°F
B.	High Containment Pressure (Hi Level)	3	2	ACTION 12	>200°F
C.	High Differential Pressure between Any Steam Line and the Steam Header	3/Steam Line	2/Steam Line	ACTION 12	#
D.	Pressurizer Low Pressure	3	2	ACTION 12	#
E.	High Steam Flow In 2/3 Steam Lines Coincident with Low T <sub>avg</sub> in 2/3 loops	2/Steam Line and 1 T <sub>avg</sub> Loop	1/Steam Line and 1 T <sub>avg</sub> in 2 Loops OR 2/Steam Line and 1 T <sub>avg</sub>	ACTION 12	≥350°F ##
F.	High Steam Flow In 2/3 Steam Lines Coincident with Low Steam Pressure in 2/3 lines	2/Steam Line and 1 Press/Line	1/Steam Line and 1 Press in 2 Lines OR 2/Steam Line and 1 Press	ACTION 12	≥350°F ##
2.	CONTAINMENT SPRAY				
A.	Manual	2	2	ACTION 13	>200°F
B.	High Containment Pressure (Hi Hi Level)	3/Set	2/Set	ACTION 12	>200°F

TABLE 3.5-3 (Continued)

ENGINEERED SAFETY FEATURES INSTRUMENTATION LIMITING OPERATING CONDITIONS

<u>NO.</u>	<u>FUNCTIONAL UNIT</u>	<u>1 TOTAL NO. OF CHANNELS</u>	<u>2 MINIMUM CHANNELS OPERABLE</u>	<u>3 OPERABLE ACTION IF COLUMN 1 OR 2 CANNOT BE MET</u>	<u>APPLICABLE CONDITIONS</u>
3.	LOSS OF POWER				
A.	480V Emerg. Bus Undervoltage (Loss of Voltage)	2/Bus	1/Bus	ACTION 14	Reactor Critical
B.	480V Emerg. Bus Undervoltage (Degraded Voltage)	3/Bus	2/Bus	ACTION 14	Reactor Critical ###

ENGINEERED SAFETY FEATURES INSTRUMENTATION LIMITING OPERATING CONDITIONSTABLE NOTATIONS

- # Above Low Pressure SI Block Permit interlock.
- ## Trip function may be blocked below Low  $T_{avg}$  Interlock setpoint.
- ### The reactor may remain critical below the Power Operating conditions with this feature inhibited for the purpose of starting reactor coolant pumps.
- ACTION 11 With the number of OPERABLE channels one less than the Minimum Channels OPERABLE requirement, restore the inoperable channel to OPERABLE status within 48 hours or be in at least the Hot Shutdown Condition within the next 8 hours and the Cold Shutdown Condition within the following 30 hours.
- ACTION 12 With the number of OPERABLE channels one less than the Total Number of Channels. Power Operation may proceed until performance of the next required operational test provided the inoperable channel is placed into the tripped condition within 1 hour.
- ACTION 13 With the number of OPERABLE channels one less than the Minimum Channels OPERABLE requirement, restore the inoperable channel to OPERABLE status within 1 hour or be in at least the Hot Shutdown Condition within the next 8 hours and the Cold Shutdown Condition within the following 30 hours.
- ACTION 14 With the number of OPERABLE channels one less than the Total Number of Channels; place the inoperable channel into the blocked condition within 1 hour, and restore the inoperable channel to OPERABLE status within 48 hours or be in at least the Hot Shutdown Condition within the next 8 hours and the Cold Shutdown Condition within the following 30 hours.

TABLE 3.5-4

ISOLATION FUNCTIONS INSTRUMENTATION LIMITING OPERATING CONDITIONS

<u>NO.</u>	<u>FUNCTIONAL UNIT</u>	<u>1</u> <u>TOTAL NO.</u> <u>OF</u> <u>CHANNELS</u>	<u>2</u> <u>MINIMUM</u> <u>CHANNELS</u> <u>OPERABLE</u>	<u>3</u> <u>OPERABLE ACTION IF</u> <u>COLUMN 1 OR 2</u> <u>CANNOT BE MET</u>	<u>APPLICABLE</u> <u>CONDITIONS</u>
1.	CONTAINMENT ISOLATION				
A.	Phase A				
i.	Safety Injection			See Item No. 1 of Table 3.5-3 for all Safety Injection initiating functions and requirements	
ii.	Manual	2	2	ACTION 11	>200°F
B.	Phase B			See Item No. 2 of Table 3.5-3 for all Containment Spray initiating functions and requirements	
C.	Ventilation Isolation				
i.	High Containment Activity. Gaseous	1	0	ACTION 15	During Containment Purge
ii.	High Containment Activity. Particulate	1	0	ACTION 15	During Containment Purge
iii.	Phase A			See Item No. 1.A of Table 3.5-4 for all Phase A initiating functions and requirements	

TABLE 3.5-4 (Continued)

ISOLATION FUNCTIONS INSTRUMENTATION LIMITING OPERATING CONDITIONS

<u>NO.</u>	<u>FUNCTIONAL UNIT</u>	<u>1</u> <u>TOTAL NO.</u> <u>OF</u> <u>CHANNELS</u>	<u>2</u> <u>MINIMUM</u> <u>CHANNELS</u> <u>OPERABLE</u>	<u>3</u> <u>OPERABLE ACTION IF</u> <u>COLUMN 1 OR 2</u> <u>CANNOT BE MET</u>	<u>APPLICABLE</u> <u>CONDITIONS</u>
2.	STEAM LINE ISOLATION				
	A. High Steam Flow in 2/3 Steam Lines Coincident with Low T <sub>avg</sub> in 2/3 Loops	See Item No. 1.E of Table 3.5-3 for initiating functions and requirements			
	B. High Steam Flow in 2/3 Steam Lines Coincident with Low Steam Pressure in 2/3 lines	See Item No. 1.F of Table 3.5-3 for initiating functions and requirements			
	C. High Containment Pressure (Hi Hi Level)	See Item No. 2.B of Table 3.5-3 for initiating functions and requirements			
	D. Manual	1/Line	1/Line	ACTION 16	≥350°F
3.	FEEDWATER LINE ISOLATION				
	A. Safety Injection	See Item No. 1 of Table 3.5-3 for all Safety Injection initiating functions and requirements			

TABLE 3.5-4 (Continued)

ISOLATION FUNCTIONS INSTRUMENTATION LIMITING OPERATING CONDITIONS

TABLE NOTATIONS

- ACTION 15 With less than the Total Number of Channels, Power Operations may continue provided the Containment Ventilation Purge and Exhaust valves are maintained closed.
- ACTION 16 With the number of channels one less than the Total Number of Channels, restore the inoperable channel to OPERABLE status within 48 hours or declare the associated valve inoperable and take the ACTION required by Specification 3.4.3.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 175 TO FACILITY OPERATING LICENSE NO. DPR-23  
CAROLINA POWER & LIGHT COMPANY  
H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2  
DOCKET NO. 50-261

## 1.0 INTRODUCTION

By letter dated December 10, 1995, as supplemented August 1, 1996, and September 4, 1996, the Carolina Power & Light Company (licensee) submitted a request for changes to the H. B. Robinson Steam Electric Plant, Unit No. 2 (HBR), Technical Specifications (TS). The requested changes would revise TS Section 3.5.1 and Tables 3.5-2, 3, and 4 concerning the reactor trip system (RTS), engineered safety feature actuation system (ESFAS), and isolation function. The TS would be revised to (1) specify actions to be taken when an instrument channel becomes inoperable, (2) add an "Applicable Conditions" column that defines the applicability and/or mode of operation of each functional unit, and (3) make editorial enhancements. The August 1, 1996, and September 4, 1996, letters provided administrative changes to the TS pages that did not change the initial proposed no significant hazards consideration determination.

The current TS at HBR were implemented prior to the development and issuance of NUREG-0452, Revision 3, "Standard Technical Specifications for Westinghouse Pressurized Water Reactors," dated Fall 1981. As such, many of the operator actions specified in NUREG-0452 for inoperable equipment were not contained in the HBR TS. The proposed changes will upgrade the RTS, ESFAS, and isolation function instrumentation to more closely agree with NUREG-0452, Revision 3. Currently, the TS Section 3.5.1 and Tables 3.5-2, 3, and 4 do not specify discrete operator actions whenever an instrument channel becomes inoperable and do not clearly define the applicability for each functional unit.

## 2.0 EVALUATION

The current TS Section 3.5.1 and Tables 3.5-2, 3, and 4 describe the limiting condition during which RTS and ESFAS instrumentation may be inoperable. The TS section requires that when the number of channels of a particular subsystem in service falls below the limits given in the table columns entitled "Minimum Operable Channels," or the "Minimum Degree of Redundancy" cannot be achieved, operation shall be limited according to the requirement shown in column 3 of the tables. The proposed changes make editorial enhancements, delineate discrete actions whenever an instrument channel becomes inoperable, and add an "Applicable Conditions" column that defines the applicability and/or mode of operation of each functional unit.

The NRC staff has reviewed the proposed changes and has concluded that the changes are acceptable because they clarify the applicability of RT and ESFAS requirements and agree with the action statements described in NUREG-0452, Revision 3. The NRC staff finds the proposed changes, therefore, acceptable.

### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the State of South Carolina official was notified of the proposed issuance of the amendment. The State official had no comments.

### 4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (61 FR 5812). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: F. Gee, HICB

Date: September 12, 1996