

September 6, 1994

ket No. 50-261

Mr. C. S. Hinnant, Vice President
Carolina Power & Light Company
H. B. Robinson Steam Electric Plant,
Unit No. 2
Post Office Box 790
Hartsville, South Carolina 29550-0790

Dear Mr. Hinnant:

SUBJECT: ISSUANCE OF AMENDMENT NO. 150 TO FACILITY OPERATING LICENSE NO. DPR-23 REGARDING REACTOR VESSEL SURVEILLANCE SCHEDULE - H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2 (TAC NO. M88046)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 150 to Facility Operating License No. DPR-23 for the H. B. Robinson Steam Electric Plant, Unit No. 2. This amendment consists of changes to the Technical Specifications in response to your request dated October 14, 1993.

The amendment would revise TS 4.2.2 to remove the schedule for withdrawal of reactor vessel material surveillance specimens. Guidance on the proposed TS change was provided by Generic Letter (GL) 91-01, "Removal of the Schedule for the Withdrawal of Reactor Vessel Material Specimens from Technical Specifications," issued on January 4, 1991.

A copy of the related Safety Evaluation is enclosed. Notice of Issuance will be included in the Commission's bi-weekly Federal Register notice.

Sincerely,
Original Signed by:
Brenda L. Mozafari, Project Manager
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 150 to DPR-23
- 2. Safety Evaluation

cc w/enclosures:

See next page

OFFICE	LA:PD2-1	PM:PD2-1	PD:PD2-1	OGC N20
NAME	PAnderson	BMOzafari	DMatthews	M20BLA
DATE	08/2/94	08/2/94	09/6/94	07/22/94
OFF	EMCB			
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DATE	8/2/94	/ /	/ /	/ /

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

Docket No. 50-261

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Sincerely,

A handwritten signature in cursive script that reads "Brenda Mozafari".

Brenda L. Mozafari, Project Manager
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 150 to DPR-23
2. Safety Evaluation

cc w/enclosures:
See next page

Mr. C. S. Hinnant
Carolina Power & Light Company

H. B. Robinson Steam Electric
Plant, Unit No. 2

cc:

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Hartsville, South Carolina 29551-0790

AMENDMENT NO. 150 TO FACILITY OPERATING LICENSE NO. DPR-23 - H. B. ROBINSON
STEAM ELECTRIC PLANT, UNIT NO. 2

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ACRS (10)
OPA
OC/LFDCB
E. Merschoff, R-II

cc: Robinson Service List

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 150 , are hereby incorporated in the license. Carolina Power & Light Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Brenda Mozafari
for David B. Matthews, Director
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 6, 1994



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

CAROLINA POWER & LIGHT COMPANY

DOCKET NO. 50-261

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 150
License No. DPR-23

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Carolina Power & Light Company (the licensee), dated October 14, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 3.B. of Facility Operating License No. DPR-23 is hereby amended to read as follows:

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ATTACHMENT TO LICENSE AMENDMENT NO. 150

FACILITY OPERATING LICENSE NO. DPR-23

DOCKET NO. 50-261

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

<u>Remove Pages</u>	<u>Insert Pages</u>
4.2-6	4.2-6
4.2-7	--

Class 2 and Class 3 components were chosen based on Regulatory Guide 1.26 and ANSI N18.2 and N18.2a "Nuclear Safety Criteria for the Design of stationary Pressurized Water Reactor Plants."

The Surveillance Requirements for inspection of the steam generator tubes ensure that the structural integrity of this portion of the RCS will be maintained. The program for inservice inspection of steam generator tubes is based on a modification of Regulatory Guide 1.83, Revision 1. Inservice inspection of steam generator tubing is essential in order to maintain surveillance of the conditions of the tubes for evidence of mechanical damage or progressive degradation. Inservice inspection of steam generator tubing also provides a means of characterizing the nature and cause of any tube degradation so that corrective measures can be taken.

Wastage-type defects will be minimized with proper chemistry treatment of the secondary coolant. If defects or significant degradations should develop in service, this condition is expected to be detected during inservice steam generator tube examinations. Plugging will be required for all tubes with imperfections exceeding the plugging limit. Steam generator tube inspections by means of eddy current testing have demonstrated the capability to reliably detect degradation that has penetrated 20% of the original tube wall thickness.

Whenever the results of any steam generator tubing inservice inspection fall into Category C-3, these results will be reported to the Commission prior to resumption of plant operation. Such cases will be considered by the Commission on a case-by-case basis and may result in a requirement for analysis, laboratory examinations, tests, additional eddy-current inspection, and revision of the Technical Specifications.

4.2.2 Materials Irradiation Surveillance Specimens

The reactor vessel material surveillance specimens shall be removed and examined to determine changes in their material properties, as required by Appendix H to 10CFR50.

4.2.3 Primary Pump Flywheels

The flywheels shall be visually examined at the first refueling after each ten year inspection. At the fourth refueling after each ten year inspection and at each fourth refueling thereafter, the outside surfaces shall be examined by ultrasonic methods.

References

- (1) FSAR, Section 4.4
- (2) FSAR, Volume 4, Tab VII, Question VI.C



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 150 TO FACILITY OPERATING LICENSE NO. DPR-23
CAROLINA POWER & LIGHT COMPANY
H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2
DOCKET NO. 50-261

1.0 INTRODUCTION

By letter dated October 14, 1993, Carolina Power & Light Company (the licensee) submitted a request for a change to the H. B. Robinson Steam Electric Plant, Unit No. 2, (HBR) Technical Specifications (TS). The requested change would revise TS 4.2.2 to remove the schedule for withdrawal of reactor vessel material surveillance specimens. Guidance on the proposed TS change was provided by Generic Letter (GL) 91-01, "Removal of the Schedule for the Withdrawal of Reactor Vessel Material Specimens from Technical Specifications," issued on January 4, 1991.

2.0 EVALUATION

As stated in Generic Letter 91-01, a program for reactor vessel material surveillance ensures the availability of data to update the inservice operating temperature and pressure limits. This program will assist in fulfilling the requirements of Appendix H to 10 CFR, Part 50, to prevent brittle fracture of the reactor vessel. The surveillance requirements associated with these limits specify the withdrawal schedule for the reactor vessel material specimens.

Section II.B.3 of Appendix H to 10 CFR, Part 50, requires the submittal of a proposed withdrawal schedule with a technical justification as specified in 10 CFR 50.4 and approval by the NRC before implementation. The surveillance capsule withdrawal schedule for HBR was designed to meet ASTM E 185-66, applicable at the time the reactor vessel was purchased and approved when the plant was licensed for full power operation on May 20, 1970. If a licensee proposes schedule changes that are not consistent with ASTM E 185-79 or -82, the changes would likely be deemed to involve an unreviewed safety question under the current regulatory framework and would require NRC approval by a license amendment prior to implementation, as provided by 10 CFR 50.59(c).

The licensee stated in the submittal of October 14, 1993, that the surveillance schedule will be placed in the HBR Updated Final Safety Analysis Report (UFSAR). Inclusion of the withdrawal schedule in the UFSAR will provide a readily available reference for NRC inspectors and other staff. In addition, the licensee proposes a revision to TS 4.2.2 to indicate that the specimens must be removed and examined to determine changes in their material properties, as required by Appendix H to 10 CFR Part 50.

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The removal of the schedule from the TS will not result in any loss of regulatory control because changes to this schedule are controlled by the requirements of Appendix H to 10 CFR Part 50. The amendment involves no addition, deletion, or change to any plant hardware or equipment.

The NRC staff finds that the proposed change to TS 4.2.2 is consistent with the guidance provided in GL 91-01 with respect to the removal of the schedule for the withdrawal of reactor vessel specimens from the TS. The change is, likewise, consistent with the regulations, and is, therefore, acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the State of South Carolina official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a surveillance requirement. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (58 FR 59745). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: B. L. Mozafari

Date: September 6, 1994