

December 30, 1986

Docket No. 50-261

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Mr. E. E. Utley, Senior Executive Vice President  
Power Supply and Engineering & Construction  
Carolina Power and Light Company  
Post Office Box 1551  
Raleigh, North Carolina 27602

Dear Mr. Utley:

The Commission has issued the enclosed Amendment No. 111 to Facility Operating License No. DPR-23 for the H. B. Robinson Steam Electric Plant Unit No. 2. This amendment consists of changes to the Technical Specifications in response to your request dated August 28, 1986, and to your other request dated September 9, 1986 as supplemented by letter dated November 25, 1986.

The amendment revises Technical Specification Section 3.3.1.2 by correcting reference paragraphs in response to your request dated August 28, 1986. In addition, TS Sections 6.13.1.a and 6.13.1.b are revised by specifying a distance of 18 inches for dose rate measurements that are made to determine if an area is a High Radiation Area (HRA) or a Locked High Radiation Area (LHRA). The latter change is in response to your request dated September 9, 1986, as supplemented November 25, 1986.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

/s/

Glode Requa, Project Manager  
PWR Project Directorate #2  
Division of PWR Licensing-A  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 111 to DPR-23
2. Safety Evaluation

cc: w/enclosures  
See next page

LA:PAD#2  
D. Miller  
12/11/86

PM:PAD#2  
GRequa  
12/11/86

OGC  
M. Karman  
12/15/86

PD:PAD#2  
LRubenstein  
12/13/86



Mr. E. E. Utley  
Carolina Power & Light Company

H. B. Robinson 2

cc:

Thomas A. Baxter, Esquire  
Shaw, Pittman, Potts and Trowbridge  
2300 N Street, N.W.  
Washington, DC 20037

Mr. Dayne H. Brown, Chief  
Radiation Protection Branch  
Division of Facility Services  
Department of Human Resources  
701 Barbour Drive  
Raleigh, North Carolina 27603-2008

Mr. McCuen Morrell, Chairman  
Darlington County Board of Supervisors  
County Courthouse  
Darlington, South Carolina 29535

Mr. Robert P. Gruber  
Executive Director  
Public Staff - NCUC  
P.O. Box 29520  
Raleigh, North Carolina 27626-0520

Mr. H. A. Cole  
Special Deputy Attorney General  
State of North Carolina  
P.O. Box 629  
Raleigh, North Carolina 27602

Mr. D. E. Hollar  
Associate General Counsel  
Carolina Power and Light Company  
P.O. Box 1551  
Raleigh, North Carolina 27602

U.S. Nuclear Regulatory Commission  
Resident Inspector's Office  
H. B. Robinson Steam Electric Plant  
Route 5, Box 413  
Hartsville, South Carolina 29550

Regional Administrator, Region II  
U.S. Nuclear Regulatory Commission  
Suite 2900  
101 Marietta Street  
Atlanta, Georgia 30303

Mr. R. Morgan  
General Manager  
H. B. Robinson Steam Electric Plant  
Post Office Box 790  
Hartsville, South Carolina 29550



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

CAROLINA POWER AND LIGHT COMPANY

DOCKET NO. 50-261

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 111  
License No. DPR-23

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Carolina Power and Light Company (the licensee) dated August 28, 1986, and application dated September 9, 1986 as supplemented by letter dated November 25, 1986 comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-23 is hereby amended to read as follows:

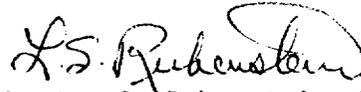
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PDR

(B) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 111, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Lester S. Rubenstein, Director  
PWR Project Directorate #2  
Division of PWR Licensing-A  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: December 30, 1986

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 111 FACILITY OPERATING LICENSE NO. DPR-23

DOCKET NO. 50-261

Revise Appendix A as follows:

Remove Pages

3.3-4  
6.13-1

Insert Pages

3.3-4  
6.13-1

valves, are not subject to the requirements of this specification.

- f. Power or air supply may be restored to any valve referenced in 3.3.1.1.g. and 3.3.1.1.h. for the purpose of valve testing or maintenance providing no more than one valve has power restored and provided that testing and maintenance is completed and power removed within 24 hours except for accumulator isolation valves (MOV 865 A,B,&C) which will have this time period limited to four hours.

6.13 HIGH RADIATION AREA

6.13.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR 20:

- a. Each High Radiation Area in which the intensity of radiation is greater than 100 mr/hr, but equal to or less than 1000mr/hr when measured at 18 inches from a source within the area, shall be barricaded and conspicuously posted as a High Radiation Area and entrance thereto shall be controlled by issuance of a Radiation Work Permit. Any individual or group of individuals permitted to enter such areas shall be provided with a radiation monitoring device which continuously indicates the radiation exposure rate.
  
- b. Each High Radiation Area in which the intensity of radiation is greater than 1000 mr/hr when measured at 18 inches from a source within the area shall be subject to the provisions of 6.13.1(a) above. In addition, locked doors shall be provided to prevent unauthorized entry into such areas and the keys shall be maintained under the administrative control of the Shift Foreman on duty and/or the Radiation Control Foreman.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 111 TO FACILITY OPERATING LICENSE NO. DPR-23

CAROLINA POWER AND LIGHT COMPANY

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2

DOCKET NO. 50-261

INTRODUCTION

By letter dated August 28, 1986, Carolina Power and Light Company (the licensee), requested a revision to the Technical Specifications to correct references cited in Section 3.3.1.2. These references were inadvertently not corrected in the licensee's submittal dated November 13, 1985 in which several paragraphs were deleted from TS Section 3.1.1 and subsequent paragraphs renumbered. Section 3.3.1.2 was not corrected at that time to reference the renumbered paragraphs. Also, by letter dated September 4, 1986, the licensee requested a change to Section 6.13.1 of the Technical Specifications. The change consisted of adding a specified distance of 18 inches at which dose rates must be measured for determining if an area is a High Radiation Area (HRA) or a Locked High Radiation Area (LHRA). The addition of the specified distance would be consistent with the Westinghouse Standard Technical Specifications (STS).

By letter dated November 25, 1986 the licensee supplemented the September 4, 1986 request by correcting an inadvertent miswording of the corrected TS page to clarify an undefined area at 1,000 mr/hr. This was clarified by adding the words ".... equal to or ...." to the second line of Section 6.13.1.a.

EVALUATION

We have reviewed the requested change to correct an error in Section 3.3.1.2. This request is merely to correct a clerical error as discussed above and consequently clarifies the TS. Therefore, we find this change acceptable. We have also reviewed the changes to Section 6.13.1 requested by the licensee's submittal dated September 4, 1986 in which the licensee proposed to add a distance of 18 inches at which the dose rate must be measured. We find that the added requirement for the distance at which the dose rate must be measured clarifies the Technical Specification and prevents misunderstanding. In addition, we have compared the licensee's requested change with the STS and find that the addition is consistent with the STS.

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Therefore, based on the above discussion, we find the requested changes to the Technical Specifications acceptable.

#### ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). This amendment also involves changes in recordkeeping, reporting or administrative procedures or requirements. Accordingly, with respect to these items, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

#### CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: December 30, 1986

Principal Contributor:  
G. Requa