

August 25, 1986

Docket No. 50-261

DISTRIBUTION

Mr. E. E. Utley, Senior Executive Vice President
Power Supply and Engineering & Construction
Carolina Power and Light Company
Post Office Box 1551
Raleigh, North Carolina 27602

Docket File	J. Partlow
NRC PDR	T. Barnhart (4)
Local PDR	W. Jones
PAD#2 Rdg	E. Butcher
T. Novak	N. Thompson
D. Miller	V. Benaroya
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OGC	ACRS (10)
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Dear Mr. Utley:

The Commission has issued the enclosed Amendment No.102 to Facility Operating License No. DPR-23 for the H. B. Robinson Steam Electric Plant Unit No. 2. This amendment consists of changes to the Technical Specifications and the license in response to your request dated September 12, 1985.

The amendment revises the Technical Specification to increase the total steam flow rate to reflect the as-built conditions of the new steam generators and deletes section 3.J of the Operating License imposed during the steam generator replacement program.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

Glode Requa, Project Manager
PWR Project Directorate #2
Division of PWR Licensing-A

Enclosures:

1. Amendment No.102 to DPR-23
2. Safety Evaluation

cc: w/enclosures
See next page

LA: PAD#2
DM: Miller
8/17/86

PM: PAD#2
GRequa, mc
8/12/86

PD: PAD#2
LRubenstein
8/12/86

GG
Bachmann
8/14/86

Mr. E. E. Utley
Carolina Power & Light Company

H. B. Robinson 2

cc:

Thomas A. Baxter, Esquire
Shaw, Pittman, Potts and Trowbridge
1800 M Street, N.W.
Washington, DC 20036

Mr. Dayne H. Brown, Chief
Radiation Protection Branch
Division of Facility Services
Department of Human Resources
P.O. Box 12200
Raleigh, North Carolina 27605

Mr. McCuen Morrell, Chairman
Darlington County Board of Supervisors
County Courthouse
Darlington, South Carolina 29535

State Clearinghouse
Division of Policy Development
116 West Jones Street
Raleigh, North Carolina 27603

Attorney General
Department of Justice
Justice Building
Raleigh, North Carolina 27602

U.S. Nuclear Regulatory Commission
Resident Inspector's Office
H. B. Robinson Steam Electric Plant
Route 5, Box 413
Hartsville, South Carolina 29550

Regional Administrator, Region II
U.S. Nuclear Regulatory Commission
Suite 2900
101 Marietta Street
Atlanta, Georgia 30303

Mr. R. Morgan
General Manager
H. B. Robinson Steam Electric Plant
Post Office Box 790
Hartsville, South Carolina 29550



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CAROLINA POWER AND LIGHT COMPANY

DOCKET NO. 50-261

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 102
License No. DPR-23

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Carolina Power and Light Company (the licensee) dated September 12, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-23 is hereby amended to read as follows:

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P PDR

(B) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.102, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. In addition, Facility Operating License DPR-23 is amended as follows:
 - (A) Delete paragraph 3.J.
4. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Lester S. Rubenstein, Director
PWR Project Directorate #2
Division of PWR Licensing-A

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 25, 1986

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 102 FACILITY OPERATING LICENSE NO. DPR-23

DOCKET NO. 50-261

Revise Appendix A as follows:

Remove Pages

3.4-3

Insert Pages

3.4-3

Basis

A reactor shutdown from power requires removal of core decay heat. Immediate decay heat removal requirements are normally satisfied by the steam bypass to the condenser. Therefore, core decay heat can be continuously dissipated via the steam bypass to the condenser as feedwater in the steam generator is converted to steam by heat absorption. Normally, the capability to return feedwater flow to the steam generators is provided by operation of the turbine cycle feedwater system.

The twelve main steam safety valves have a total combined rated capability of 1.022×10^7 lbs/hr. The total full power steam flow is 1.011×10^7 lbs/hr.; therefore, twelve (12) main steam safety valves will be able to relieve the total steam flow if necessary.⁽¹⁾ Following a loss of load, which represents the worst transient, steam flows are below the total capacity of the 12 safety valves. Therefore, over-pressurization of the secondary system is not possible.

In the unlikely event of complete loss of turbine-generator and offsite electrical power to the plant, decay heat removal would continue to be assured by the availability of either the steam-driven auxiliary feedwater pump or one of the two motor-driven auxiliary steam generator feedwater pumps operated from the diesel generators and steam discharge to the atmosphere via the main steam safety valves and atmospheric relief valves. One motor-driven auxiliary feedwater pump can supply sufficient feedwater for removal of decay heat from the plant.⁽²⁾ The auxiliary feedwater system essential features are those features that provide auxiliary feedwater flow to two out of three steam generators consistent with auxiliary feedwater pump operability. In order to provide a high degree of reliability all three auxiliary feedwater pumps will be operable prior to exceeding 350°F. The minimum amount of water in the condensate storage tank is the amount needed for at least two hours operation at hot standby conditions. If the outage is more than two hours, deep well or Lake Robinson water may be used.

An unlimited supply is available from the lake via either leg of the plant Service Water System for an indefinite time period.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 102 TO FACILITY OPERATING LICENSE NO. DPR-23
CAROLINA POWER AND LIGHT COMPANY
H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2
DOCKET NO. 50-261

INTRODUCTION

Carolina Power and Light Company, the licensee, requested in a letter dated September 12, 1985, revisions to their Technical Specifications for Robinson 2. These revisions result from the replacement of the steam generators at Robinson 2. The changes to the Technical Specifications reflect the as-built conditions at the plant, specifically a steam flow rate increase of 5% and deletion of obsolete references to a steam generator repair program.

In addition, the licensee requested that Section 3.J be deleted from the Operating License. Section 3.J, Steam Generator Repair Program, was added to the license by Amendment 77 on February 28, 1984, for the steam generator replacement construction program at H.B. Robinson 2.

EVALUATION

The replacement of the steam generators resulted in an increase in the total steam flow rate from 9.59×10^6 to 10.11×10^6 lb/hr. The new flow rate is 0.4% greater than the capacity stated in the basis of Section 3.4 of the Technical Specifications, which is 10.07×10^6 lb/hr. However, the licensee asserts that the actual combined relief capacity of the twelve main steam safety valves is 10.22×10^6 lb/hr, not 10.07×10^6 lb/hr. The revision to the Technical Specifications will reflect the actual relief capacity and the actual as-built steam flow rate.

The twelve main steam safety valves have a total combined capacity which exceeds the total full power steam flow rate by 110,000 lb/hr. Following a loss of load, which represents the worst case transient, steam flows are lower than the total capacity of the twelve safety valves. The twelve safety valves will be able to relieve the total steam flow if necessary and over-pressurization of the secondary system is highly unlikely. Therefore, the proposed revision to the H.B. Robinson Unit 2 Technical Specifications is approved.

Section 3.J of the Operating License, Steam Generator Repair Program, was added by Amendment 77 dated February 28, 1984. The purpose of this condition was to impose certain requirements for the duration of the Steam Generator Repair Program. The Steam Generator Repair Program was completed in early 1985; therefore the requirements of Section 3.J are no longer applicable.

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SUMMARY

Based on the above discussions, we find that the licensee's requests to revise the Technical Specifications to reflect the as-built near steam generator steam flow rate and the request to delete Section 3.J from the Operating License acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: August 25, 1986

Principal Contributors:

G. Requa
R. Karsch