

August 26, 1985

Docket No. 50-261

DISTRIBUTION

Mr. E. E. Utley, Senior Executive Vice President
Power Supply and Engineering & Construction
Carolina Power and Light Company
Post Office Box 1551
Raleigh, North Carolina 27602


Docket File
L PDR
ORB#1 Rdg
OELD
GRequa
SECY
EJordan
JPartlow
WJones
ACRS 10
CMiles

NRC PDR
Gray File
HThompson
HEmami
CParrish
LHarmon
BGrimes
TBarnhart 4
MVirgilio
RDiggs
RBallard

Dear Mr. Utley:

The Commission has issued the enclosed Amendment No.93 to Facility Operating License No. DPR-23 for the H. B. Robinson Steam Electric Plant Unit No. 2. This amendment consists of changes to the Technical Specifications in response to your request dated April 30, 1985.

The amendment revises Technical Specifications Table 3.5-1 Items 6.a and 6.b to increase the voltage setpoint tolerances for loss of voltage and degraded grid voltage relays and increase the loss of voltage relay trip time. This completes our review of this item (TAC No. 57738).

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

/s/GRequa

Glode Requa, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosures:

- 1. Amendment No. 93 to DPR-23
- 2. Safety Evaluation

cc: w/enclosures
See next page

ORB#1:DL
CParrish
8/5/85

ORB#1:DL
GRequa,ps
8/1/85

BC-ORB#1:DL
SVarga
8/2/85

OELD
8/2/85

AD-OR:DL
GLainas
8/2/85

8509060198 850826
PDR ADOCK 05000261
P PDR

Mr. E. E. Utley
Carolina Power & Light Company

H. B. Robinson 2

cc:
G. F. Trowbridge, Esquire
Shaw, Pittman, Potts and Trowbridge
1800 M Street, N.W.
Washington, DC 20036

Mr. Dayne H. Brown, Chief
Radiation Protection Branch
Division of Facility Services
Department of Human Resources
P.O. Box 12200
Raleigh, North Carolina 27605

Mr. McCuen Morrell, Chairman
Darlington County Board of Supervisors
County Courthouse
Darlington, South Carolina 29535

State Clearinghouse
Division of Policy Development
116 West Jones Street
Raleigh, North Carolina 27603

Attorney General
Department of Justice
Justice Building
Raleigh, North Carolina 27602

U.S. Nuclear Regulatory Commission
Resident Inspector's Office
H. B. Robinson Steam Electric Plant
Route 5, Box 413
Hartsville, South Carolina 29550

Regional Administrator, Region II
U.S. Nuclear Regulatory Commission
Suite 2900
101 Marietta Street
Atlanta, Georgia 30303

Mr. R. Morgan
General Manager
H. B. Robinson Steam Electric Plant
Post Office Box 790
Hartsville, South Carolina 29550



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CAROLINA POWER AND LIGHT COMPANY

DOCKET NO. 50-261

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 93
License No. DPR-23

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Carolina Power and Light Company (the licensee) dated April 30, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-23 is hereby amended to read as follows:

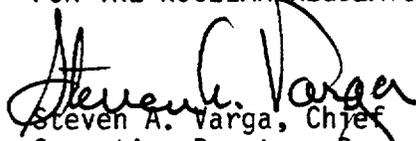
8509060204 850826
PDR ADDCK 05000261
P PDR

(B) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 93, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 26, 1985

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 93 FACILITY OPERATING LICENSE NO. DPR-23

DOCKET NO. 50-261

Revise Appendix A as follows:

Remove Pages

Insert Pages

3.5-10

3.5-10

3.5-11

3.5-11

TABLE 3.5-1

ENGINEERED SAFETY FEATURE SYSTEM INITIATION INSTRUMENT SETTING LIMITS

<u>NO.</u>	<u>FUNCTIONAL UNIT</u>	<u>CHANNEL ACTION</u>	<u>SETTING LIMIT</u>
1.	High Containment Pressure (HI Level)	Safety Injection*	≤ 5 psig
2.	High Containment Pressure (HI-HI Level)	a. Containment Spray** b. Steam Line Isolation	≤ 25 psig
3.	Pressurizer Low Pressure	Safety Injection*	≥ 1700 psig
4.	High Differential Pressure Between any Steam Line and the Steam Line Header	Safety Injection*	≤ 150 psi
5.	High Steam Flow in 2/3 Steam Lines*** Coincident with Low T_{avg} or Low Steam Line Pressure	a. Safety Injection* b. Steam Line Isolation	$< 40\%$ (at zero load) of full steam flow $< 40\%$ (at 20% load) of full steam flow $< 110\%$ (at full load) of full steam flow $> 541^\circ\text{F } T_{avg}$ ≥ 600 psig steam line pressure
6.	Loss of Power a. 480V Emerg. Bus Undervoltage (Loss of Voltage) Time Delay	Trip Normal Supply Breaker	328 Volts $\pm 10\%$ < 1 sec when voltage is reduced to zero

TABLE 3.5-1 (Continued)

ENGINEERED SAFETY FEATURE SYSTEM INITIATION INSTRUMENT SETTING LIMITS

<u>NO.</u>	<u>FUNCTIONAL UNIT</u>	<u>CHANNEL ACTION</u>	<u>SETTING LIMIT</u>
6. (Cont'd)	b. 480V Emerg. Bus Undervoltage (Degraded Voltage) Time Delay	Trip Normal Supply Breaker	415 Volts \pm 4 Volts 10.0 Second Delay \pm 0.5 sec.
7.	Containment Radioactivity High	Ventilation Isolation	The alarm is set with a method described in the ODCM.

-
- * Initiates also containment isolation (Phase A), feedwater line isolation and starting of all containment fans.
 - ** Initiates also containment isolation (Phase B).
 - *** Derived from equivalent WP measurements.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 93 TO FACILITY OPERATING LICENSE NO. DPR-23

CAROLINA POWER AND LIGHT COMPANY

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2

DOCKET NO. 50-261

I. Introduction

By letter dated April 30, 1985 Carolina Power and Light Company (CP&L) proposed changes to the existing Technical Specification Table 3.5-1, Items 6.a and 6.b. CP&L states that the proposed changes when implemented will increase the voltage setpoint tolerances for loss of voltage and degraded grid voltage relays for 0.3% and 0.24% to 10% to 0.96%, respectively. In addition, the loss of voltage relays trip time at zero volts will be modified from the present 0.75 ± 0.25 seconds to equal or less than 1 second. The voltage setpoint for the degraded grid voltage relays is also proposed to be increased from 412 volts to 415 volts.

II. Evaluation

The licensee states that the proposed tolerances are more realistic to maintain due to the inherent characteristics of these relays and do not impact their safety functions. With the proposed tolerances, the sensing voltage for the loss of voltage relay can drop to a minimum of 295 volts under the worst condition. However, this voltage is higher than the contactor drop-out voltage (291) volts) and does not prevent the safety equipment from performing their respective functions under worst case transient conditions. The proposed increase of voltage and tolerance for the degraded grid voltage relays from $412 \pm 1V$ to $415 \pm 4V$ does not change the magnitude of the minimum trip value, since, in both cases, the setpoint minimum voltage under the worst case condition remain the same, 411 volts.

III. Summary

Based on the above (II) discussions we find the proposed Technical Specification changes acceptable.

IV. Environmental Consideration

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in

8509060213 850826
PDR ADDCK 05000261
P PDR

individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

V. Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: August 26, 1985

Principal Contributor:

H. Emami