November 14, 1984

Docket No. 50-261

Mr. E. E. Utley, Executive Vice President
Power Supply and Engineering & Construction
Carolina Power and Light Company
Post Office Box 1551
Raleigh, North Carolina 27602

Dear Mr. Utley:

Distribution	Docket file
NRC PDR	Local PDR
ORB#1 Rdg.	Gray file (4)
DEisenhut	CParrish
GRequa	OELD
EJordan	JPartlow
PMcKee	LHarmon
TBarnhart (4)	WJones
DBrinkman	ACRS(10)
RDiggs	RBallard

SUBJECT: CORRECTIONS TO AMENDMENT NO. 85 TECHNICAL SPECIFICATION

The enclosed Technical Specification pages are corrections for our Amendment No. 85. Amendment No. 85 dated October 4, 1984 issued a Technical Specification in compliance with Appendix I of 10 CFR Part 50. The Technical Specification had been under our review for several years. Recently we also issued Amendment No. 84, October 4, 1984, which incorporated organizational changes and Amendment No. 83, September 10, 1984 which involved surveillance requirements in accordance with Section XI of the ASME Border and Pressure Vessels Code.

Because of timing of the incoming requests versus our issuance of these amendments, Amendment No. 85 (containing requests from past several years) inadvertently deleted items already approved by Amendments Nos. 83 and 84 which contained our approval of more recent requests. One page, 4.10-5, was inadvertently missing from and one page 4.18-1 was inadvertently deleted by Amendment NO. 85.

The corrected pages involved and enclosed, herewith, are:

Appendix A revision sheet
pg. 3.5-13
pg. 4.1-6
pg. 4.10-5
pg. 6.5-7a
pg. 6.5-14
pg. 6.5-15

We apologize for any inconvenience this may have caused you.

Sincerely,

/s/GRequa

Glode Requa, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosure:
As stated

cc w/enclosure:
See next page

*See previous white for concurrence

ORB#1:DL	ORB#1:DL	C-ORB#1:DL
CParrish*	GRequa*	Svarga
11/ /84	11/ /84	11/14/84

Docket No. 50-261

Distribution	Docket file
NRC PDR	Local PDR
ORB#1 Rdg.	Gray file (4)
DEisenhut	CParrish
GRequa	OELD
EJordan	JPartlow
PMcKee	LHarmon
TBarnhart (4)	WJones
DBrinkman	ACRS(10)
RDiggs	RBallard

Mr. E. E. Utley, Executive Vice President
Power Supply and Engineering & Construction
Carolina Power and Light Company
Post Office Box 1551
Raleigh, North Carolina 27602

Dear Mr. Utley:

SUBJECT: CORRECTIONS TO AMENDMENT NO. 85 TECHNICAL SPECIFICATION

The enclosed Technical Specification pages are corrections for our Amendment No. 85. Amendment No. 85 dated October 4, 1984 issued a Technical Specification in compliance with Appendix I of 10 CFR Part 50. The Technical Specification had been under our review for several years. Recently we also issued Amendment No. 84, October 4, 1984, which incorporated organizational changes and Amendment No. 83, September 10, 1984 which involved surveillance requirements in accordance with Section XI of the ASME Border and Pressure Vessels Cod.

Because of timing of the incoming requests versus our issuance of these amendments, Amendment No. 85 (containing requests from past several years) inadvertently deleted items already approved by Amendments Nos. 83 and 84 which contained our approval of more recent requests. One page, 4.10-5, was inadvertently missing from and one page 4.18-1 was inadvertently deleted by Amendment NO. 85.

The corrected pages involved and enclosed, herewith, are:

Appendix A revision sheet
pg. 3.5-13
pg. 4.1-6
pg. 4.10-5
pg. 6.5-7a
pg. 6.5-14
pg. 6.5-15

We apologize for any inconvenience this may have caused you.

Sincerely,

Glode Requa, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosure:
As stated

cc w/enclosure:
See next page

ORB#1:DL	ORB#1:DL	C-ORB#1:DL
CParrish	GRequa	SVarga
11/8 /84	11/9 /84	11/ /84

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

November 14, 1984

Docket No. 50-261

Mr. E. E. Utley, Executive Vice President
Power Supply and Engineering & Construction
Carolina Power and Light Company
Post Office Box 1551
Raleigh, North Carolina 27602

Dear Mr. Utley:

SUBJECT: CORRECTIONS TO AMENDMENT NO. 85 TECHNICAL SPECIFICATION

The enclosed Technical Specification pages are corrections for our Amendment No. 85. Amendment No. 85 dated October 4, 1984 issued a Technical Specification in compliance with Appendix I of 10 CFR Part 50. The Technical Specification had been under our review for several years. Recently we also issued Amendment No. 84, October 4, 1984, which incorporated organizational changes and Amendment No. 83, September 10, 1984 which involved surveillance requirements in accordance with Section XI of the ASME Border and Pressure Vessels Code.

Because of timing of the incoming requests versus our issuance of these amendments, Amendment No. 85 (containing requests from past several years) inadvertently deleted items already approved by Amendments Nos. 83 and 84 which contained our approval of more recent requests. One page, 4.10-5, was inadvertently missing from and one page 4.18-1 was inadvertently deleted by Amendment No. 85.

The corrected pages involved and enclosed, herewith, are:

Appendix A revision sheet
pg. 3.5-13
pg. 4.1-6
pg. 4.10-5
pg. 6.5-7a
pg. 6.5-14
pg. 6.5-15

We apologize for any inconvenience this may have caused you.

Sincerely,

Glode Requa, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosure:
As stated

cc w/enclosure:
See next page

Mr. E. E. Utley
Carolina Power and Light Company

H. B. Robinson Steam Electric
Plant 2

cc: G. F. Trowbridge, Esquire
Shaw, Pittman, Potts and Trowbridge
1800 M Street, N.W.
Washington, DC 20036

Regional Radiation Representative
EPA Regiona IV
345 Courtland Street, N.E.
Atlanta, GA 30308

Mr. McCuen Morrell, Chairman
Darlington County Board of Supervisors
County Courthouse
Darlington, South Carolina 29535

State Clearinghouse
Division of Policy Development
116 West Jones Street
Raleigh, North Carolina 27603

Attorney General
Department of Justice
Justice Building
Raleigh, North Carolina 27602

U.S. Nuclear Regulatory Commission
Resident Inspector's Office
H. B. Robinson Steam Electric Plant
Route 5, Box 413
Hartsville, South Carolina 29550

James P. O'Reilly
Regional Administrator - Region II
U.S. Nuclear Regulatory Commission
Sutie 2900
101 Marietta Street
Atlanta, GA 30303

Mr. R. Morgan
General Manager
H. B. Robinson Steam Electric Plant
Post Office Box 790
Hartsville, South Carolina 29550

Mr. Dayne H. Brown, Chief
Radiation Protection Branch
Division of Facility Services
Department of Human Resources
P.O. Box 12200
Raliegh, North Carolina 27605

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 85 FACILITY OPERATING LICENSE NO. DPR-23

DOCKET NO. 50-261

Revise Appendix A as follows:

Remove Pages	Insert Pages
i thru vi (Table of Contents)	i thru vii (Table of Contents)
1-3 thru 1-4	1-3 thru 1-8
3.5-1 thru 3.5.14	3.5-1 thru 3.5-30
3.9-1 thru 3.9-4	3.9-1 thru 3.9-11
----------	3.16-1 thru 3.16-8
----------	3.17-1 thru 3.17-13
4.1-5	4.1-5
4.10-1 thru 4.10-3	4.10-1 thru 4.10-10
----------	----------
----------	4.19-1 thru 4.19-7
----------	4.20-1 thru 4.20-4
----------	4.21-1 thru 4.21-2
6.5.1	6.5.1
6.5.1a	6.5.1a
6.5.7	6.5.7
6.5.7a	6.5.7a
6.5.14	6.5.14
6.5.15	6.5.15
6.9-3 thru 6.9-9	6.9-3 thru 6.9-16
6.10-2	6.10-2
6.14-1	6.14-1
----------	6.15-1
----------	6.16-1
	6.17-1 thru 6.17-2

TABLE 3.5-2 (Cont'd)

REACTOR TRIP INSTRUMENTATION LIMITING OPERATING CONDITIONS

NO.	FUNCTIONAL UNIT	1 MINIMUM OPERABLE CHANNELS	2 MINIMUM DEGREE OF REDUNDANCY	3 OPERATOR ACTION IF CONDITIONS OF COLUMN 1 OR 2 CANNOT BE MET
12.	Lo Lo Steam Generator Water Level	2	1	Maintain Hot Shutdown
13.	Underfrequency 4 KV System	2	1	Maintain Hot Shutdown
14.	Undervoltage on 4 KV System	2	1	Maintain Hot Shutdown
15.	Control Rod Misalignment Monitor****			
	a. Rod Position Deviation	1	0	Log individual rod position once/hour, and after a load change >10% or after >30 inches of control rod motion
	b. Quadrant Power Tilt Monitor (upper and lower ex-core neutron detectors)	1	0	Log individual upper and lower ion chamber currents once/hour and after a load change >10% or after >30 inches of control rod motion
16.	Steam Flow/Feedwater Flow Mismatch	1	0	Maintain Hot Shutdown
17.	Low Steam Generator Water Level	1	0	Maintain Hot Shutdown

* For zero power physics testing, it is permissible to take one channel out of service.
** When two of four power channels are greater than 10% full power, hot shutdown is not required.
*** When one of two intermediate range channels is greater than 1E-10 amps, hot shutdown is not required.
**** If both rod misalignment monitors (a and b) are inoperable for two hours or more, the nuclear overpower trip shall be reset to 93 percent of rated power in addition to the increased surveillance noted.
R.P. = Rated Power

TABLE 4.1-1 (Continued)

	Channel Description	Check	Calibrate	Test	Remarks
9.	Analog Rod Position	S (1,2)	R	M	(1) With step counters (2) Following rod motion in excess of six inches when the computer is out of service
10.	Rod Position Bank Counters	S (1,2)	N.A.	N.A.	(1) Following rod motion in excess of six inches when the computer is out of service (2) With analog rod position
11.	Steam Generator Level	S	R	M	
12.	Charging Flow	N.A.	R	N.A.	
13.	Residual Heat Removal Pump Flow	N.A.	R	N.A.	
14.	Boric Acid Tank Level	D (1)	R	N.A.	(1) Bubbler tube rodded weekly
15.	Refueling Water Storage Tank Level	W	R	N.A.	
16.	Boron Injection Tank Level	W	R	N.A.	
17.	Volume Control Tank Level	N.A.	R	N.A.	
18.	Containment Pressure	D	R	B/W (1)	(1) Containment isolation valve signal
19.	Deleted by Amendment No. 85				
20.	Boric Acid Makeup Flow Channel	N.A.	R	N.A.	

TABLE 4.10-1

RADIOACTIVE LIQUID WASTE SAMPLING AND ANALYSIS PROGRAM

Type of Release	Sampling Frequency	Minimum Analysis Frequency	Required Activity Analysis	Required LLD[a] μ Ci/ml
Batch Waste Releases[b] 1. Monitor Tanks 2. Waste Condensate Tanks 3. Drainage of Systems	P Grab Sample	P on Grab Sample	Principal Gamma Emitters[c]	5E-07
			I-131	1E-06
	P Grab Sample One Batch/M	M on Grab Sample	Dissolved and Entrained Gases (gamma emitters)	1E-05
	P Grab Sample Each Batch and Composited[d]	M on Composite	Tritium	1E-05
			Gross Alpha	1E-07
	P Grab Sample Each Batch and Composited[d]	Q on Composite	Sr-89, Sr-90	5E-08
			Fe-55	1E-6
Continuous Releases[e] 1. Steam Generator Blowdown	D Grab Sample	W on Grab Sample	Principal Gamma Emitters[c]	5E-07
			I-131	1E-06
	M Grab Sample	M on Grab Sample	Dissolved and Entrained Gases (gamma emitters)	1E-05
	D Grab Sample and Composited[d,f]	M on Composite	Tritium	1E-05
			Gross Alpha	1E-07
	D Grab Sample and Composited[d,f]	Q on Composite	Sr-89, Sr-90	5E-08
			Fe-55	1E-6

6.5.1.6.7 In the event of disagreement between the recommendations of the Plant Nuclear Safety Committee and the actions contemplated by the General Manager, the course determined by the General Manager to be more conservative will be followed. The Manager - Robinson Nuclear Project and the Manager - Corporate Nuclear Safety will be notified within the 24 hours of the disagreement and subsequent actions.

6.5.1.6.8 The PNSC shall maintain written minutes of each meeting that, at a minimum, document the results of all PNSC activities performed under the provisions of these Technical Specifications; and copies shall be provided to the Manager - Robinson Nuclear Project and to the Manager - Corporate Nuclear Safety.

(4) The verification of compliance and implementation of the requirements of the Quality Assurance Program to meet the criteria of Appendix B, 10CFR50, at least once per 24 months.

(5) The Emergency Plan and implementing procedures at least once per 12 months.

(6) The Security Plan and implementing procedures at least once per 12 months.

(7) The Facility Fire Protection Program and implementing procedures at least once per 24 months.

(8) Any other area of facility operation considered appropriate by the Corporate Quality Assurance Performance Evaluation Unit; the Executive Vice President - Power Supply and Engineering & Construction; or the Senior Vice President - Nuclear Generation.

(9) The Radiological Environmental Monitoring Program and the results thereof at least once per 12 months.

(10) The Offsite Dose Calculation Manual and implementing procedure at least once per 24 months.

(11) The Process Control Program and implementing procedures for solidification of radioactive wastes at least once per 24 months.

(12) The performance of activities required by the Quality Assurance Program to meet the criteria of Regulatory Guide 4.15, December 1977 at least once per 12 months.

e. Distribute reports and other records to appropriate managers.

6.5.3.3 a. Audit personnel shall be independent of the area audited. Selection for auditing assignments is based on experience or training that establishes that their qualifications are commensurate with the complexity or special nature of the activities to be audited. In selecting auditing personnel, consideration shall be given to special abilities, specialized technical training, prior pertinent experience, personal characteristics, and education.

b. Qualified outside consultants or other individuals independent from those personnel directly involved in plant operation shall be used to augment the audit teams when necessary. Individuals performing the audits may be members of the audited organization; however, they shall not audit activities for which they have immediate responsibility, and while performing the audit, they shall not report to a management representative who has immediate responsibility for the activity audited.

6.5.3.4 Results of plant audits are approved by the Manager - QA Services Section and transmitted to the Executive Vice President - Power Supply and Engineering & Construction; the Senior Vice President - Nuclear Generation; Manager - Robinson Nuclear Project; General Manager; and the Vice President - Corporate Nuclear Safety & Research; and others, as appropriate within 30 days after the completion of the audit.

6.5.3.5 The Corporate Quality Assurance Audit Program shall be conducted in accordance with written, approved procedures.