

October 12, 1984

Docket No. 50-261

Mr. E. E. Utley, Executive Vice President  
Power Supply and Engineering & Construction  
Carolina Power and Light Company  
Post Office Box 1551  
Raleigh, North Carolina 27602

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Dear Mr. Utley:

The Commission has issued the enclosed Amendment No. 86 to Facility Operating License No. DPR-23 for the H. B. Robinson Steam Electric Plant Unit No. 2. This amendment consists of changes to the Technical Specifications in response to your request dated January 9, 1984.

The amendment would revise the Technical Specifications to correct inconsistencies regarding AFW pump automatic initiation.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular monthly Federal Register notice.

Sincerely,

/s/GRequa

Glode Requa, Project Manager  
Operating Reactors Branch #1  
Division of Licensing

Enclosures:

1. Amendment No. 86 to DPR-23
2. Safety Evaluation

cc: w/enclosures  
See next page

ORB#1:DL  
CParrish  
10/10/84

ORB#1:DL  
GRequa;ps  
10/10/84

C-ORB#1:DL  
DBrinkman  
10/10/84

OELD  
R Bachmann  
10/11/84

AD:OR:DL  
GInas  
10/11/84



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

October 12, 1984

Docket No. 50-261

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Power Supply and Engineering & Construction  
Carolina Power and Light Company  
Post Office Box 1551  
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Sincerely,

A handwritten signature in cursive script that reads "Glode Requa".

Glode Requa, Project Manager  
Operating Reactors Branch #1  
Division of Licensing

Enclosures:

1. Amendment No. 86 to DPR-23
2. Safety Evaluation

cc: w/enclosures  
See next page

Mr. E. E. Utley  
Carolina Power and Light Company

H. B. Robinson Steam Electric  
Plant 2

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H. B. Robinson Steam Electric Plant  
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

CAROLINA POWER AND LIGHT COMPANY

DOCKET NO. 50-261

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 86  
License No. DPR-23

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Carolina Power and Light Company (the licensee) dated January 9, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-23 is hereby amended to read as follows:

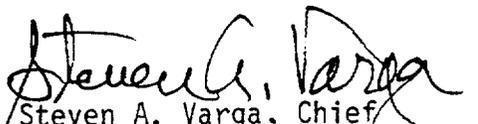
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(B) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 86, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Steven A. Varga, Chief  
Operating Reactors Branch #1  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: October 12, 1984

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 86 FACILITY OPERATING LICENSE NO. DPR-23

DOCKET NO. 50-261

Revise Appendix A as follows:

Remove Page

3.4-5

Insert Page

3.4-5

TABLE 3.4-1

AUXILIARY FEEDWATER FLOW AUTOMATIC INITIATION\*

<u>NO.</u>	<u>FUNCTIONAL UNIT</u>	<u>1 MINIMUM CHANNELS OPERABLE</u>	<u>2 MINIMUM DEGREE OF REDUNDANCY</u>	<u>3 OPERATOR ACTION IF CONDITIONS OF COLUMN 1 OR 2 CANNOT BE MET</u>
1.	Steam Gen. Water Level-low-low a. Start Motor-Driven Pumps b. Start Turbine-Driven Pump	2/Steam Generator 2/Steam Generator	1/Steam Generator 1/Steam Generator	Maintain Hot Shutdown Maintain Hot Shutdown
2.	Undervoltage-4KV Busses 1 & 4 Start Turbine-Driven Pump (15 Second Time Delay Pickup)	2 Per Bus	0	Note 1
3.	S.I. Start Motor-Driven Pumps	See Table 3.5-3, Item No.1		
4.	Station Blackout Start Motor-Driven Pumps (40 Second Time Delay Prior to Starting MD AFW Pumps on Black- out Sequence)	2 Per Bus	0	Note 2
5.	Trip of Main Feedwater Pumps Start Motor-Driven Pumps	1/Pump	0	Note 2

\* This table is applicable whenever the RCS is >350°F except Item 5. Item 5 is applicable only when the RCS is at normal operating temperature and the reactor is critical.

Note 1: 4KV Busses 1, 2, and 4 each have two undervoltage relays. One relay on each of the three busses provides an input to the reactor trip logic. Both relays on Busses 1 and 4 provide inputs to the SD AFW pump start logic. If the undervoltage relay on Busses 1 or 4 that provides the input to the reactor trip logic fails, follow the requirements of Table 3.5-2 Item 14 in addition to the following. If either 4KV undervoltage relay on Busses 1 or 4 fails, within 4 hours insert the equivalent of an undervoltage signal from the affected relay in the SD AFW pump start circuit and repair the affected relay within 7 days. If the affected relay is not repaired in the 7 days, then commence a normal plant shutdown to hot standby.

Note 2: Restore the inoperable channel to operable status within 48 hours. If the inoperable channel is not restored to an operable status within 48 hours, then commence a normal plant shutdown and cooldown to <350°F.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 86 TO FACILITY OPERATING LICENSE NO. DPR-23  
CAROLINA POWER AND LIGHT COMPANY  
H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2  
DOCKET NO. 50-261

Introduction

By letter dated January 9, 1984, the licensee submitted a request for a license amendment concerning Technical Specification (TS) revisions to correct errors and to achieve consistency within the TS. The changes concern the Automatic Feedwater (AFW) pump initiation. NRC Inspection Report 83-09 (IER 83-09), dated April 26, 1983, requested that the licensee clarify discrepancies between Table 3.4-1 and other existing AFW TS.

Evaluation

The licensee has requested a revision in the H. B. Robinson Unit 2 Technical Specification Table 3.4-1 to require the AFW flow automatic initiation system be operable whenever the RCS is greater than 350°F.

Currently, the notes to Table 3.4-1 imply that the AFW pump automatic initiation feature is required above 200°F whereas TS 3.4.1 and Table 3.5-5 require operability of other AFW system components above 350°F. Therefore, the licensee requested that the temperature requirement in Table 3.4-1 be changed to 350°F to be consistent with the other AFW TS requirements. The 4 kV Bus undervoltage signal to start the turbine-driven AFW pump (Item 2) should also be operable under the same conditions. The only item in Table 3.4-1 which should not be under this temperature specification is the AFW actuation signal from trip of both main feedwater pumps (Item 5). It should not be changed because this signal is defeated prior to securing both main feed pumps at no load temperature in order to prevent the undesirable condition of an AFW auto start and the subsequent steam generator blowdown valve auto closure.

We have reviewed the licensee's request and find that the requested changes would provide consistency within the Technical Specifications thus eliminating confusion due to two-temperature requirements. In addition, the change would be consistent with the Westinghouse Standard Technical Specifications, NUREG-0452. Therefore, the staff finds these changes acceptable.

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Environmental Consideration

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: October 12, 1984

Principal Contributor:

G. Requa