

#### UNITED STATES NUCLEAR REGULATORY COMMISSION REGION IV 611 RYAN PLAZA DRIVE, SUITE 400 ARLINGTON, TEXAS 76011-8064

February 14,

R. T. Ridenoure Division Manager - Nuclear Operations Omaha Public Power District Fort Calhoun Station FC-2-4 Adm. P.O. Box 550 Fort Calhoun, Nebraska 68023-0550

#### SUBJECT: FORT CALHOUN STATION - MEETING TO DISCUSS CONTROL ELEMENT DRIVE MECHANISM (CEDM) CRACKING ISSUES

Dear Mr. Gambhir:

This refers to the meeting conducted at Arlington, Texas, on February 8, 2002, between your staff and the NRC. The participants discussed the similarities and differences between Fort Calhoun Station and Palisades Station CEDMs.

The discussions included the actions you have taken and plan to take to ensure early detection and corrective action for potential cracks in the CEDM upper housing welds. Your staff agreed to provide the results of the examinations performed on drive housings during your next refueling outage.

The attendance list and presentation slides are enclosed with this summary (Enclosures 1 and 2, respectively).

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosures will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at http://www.nrc.gov/reading-rm/adams.html (the Public Electronic Reading Room).

Sincerely,

/**RA**/

Charles S. Marschall, Chief Engineering Branch Division of Reactor Safety

Docket: 50-285 License: DPR-40 Omaha Public Power District

**Omaha Public Power District** 

Enclosures:

1. Attendance List

2. NRC Presentation

cc w/enclosures: Mark T. Frans, Manager Nuclear Licensing Omaha Public Power District Fort Calhoun Station FC-2-4 Adm. P.O. Box 550 Fort Calhoun, Nebraska 68023-0550

James W. Chase, Division Manager Nuclear Assessments Fort Calhoun Station P.O. Box 550 Fort Calhoun, Nebraska 68023-0550

David J. Bannister, Manager - Fort Calhoun Station Omaha Public Power District Fort Calhoun Station FC-1-1 Plant P.O. Box 550 Fort Calhoun, Nebraska 68023-0550

James R. Curtiss Winston & Strawn 1400 L. Street, N.W. Washington, D.C. 20005-3502

Chairman Washington County Board of Supervisors Washington County Courthouse P.O. Box 466 Blair, Nebraska 68008

Sue Semerena, Section Administrator Nebraska Health and Human Services System Division of Public Health Assurance Consumer Services Section 301 Centennial Mall, South P.O. Box 95007 Lincoln, Nebraska 68509-5007 Omaha Public Power District

Electronic distribution from ADAMS by RIV: Regional Administrator (EWM) DRP Director (KEB) DRS Director (ATH) Senior Resident Inspector (WCW) Branch Chief, DRP/C (KMK) Senior Project Engineer, DRP/C (vacant) Staff Chief, DRP/TSS (PHH) RITS Coordinator (NBH) RidsNrrDipmLipb

SRI:EMB	C:EMB				
CJPaulk	CSMarschall				
/ <b>RA</b> / T	/RA/				
02/11/02	02/14/02				
OFFICIAL RECORD COPY		T=	Telephone	E=E-mail	F=Fax

#### **ENCLOSURE 1**

#### ATTENDANCE LIST

#### Licensee:

- D. Bannister, Plant Manager
- M. Frans, Manager, Nuclear Licensing
- W. Gates, Vice President, Nuclear
- R. Lisowyj, Senior Nuclear Design Engineer
- J. McManis, Manager, Design Engineering
- R. Phelps, Division Manager, Nuclear Engineering
- R. Ridenoure, Division Manager, Nuclear Operations
- K. Woods, Senior Nuclear Design Engineer

#### NRC:

- L. Berger, Branch Secretary
- K. Brockman, Director, Division of Reactor Projects (DRP)
- A. Howell, Director, Division of Reactor Safety (DRS)
- K. Kennedy, Chief, Projects Branch C, DRP
- C. Marschall, Chief, Engineering and Maintenance Branch, DRS
- E. Merschoff, Regional Administrator, Region IV
- C. Paulk, Senior Reactor Inspector, DRS
- W. Walker, Senior Resident Inspector
- A. Wang, Project Manager, Office of Nuclear Reactor Regulation
- L. Willoughby, Resident Inspector

#### **ENCLOSURE 2**

Licensee Presentation

#### Control Element Drive Mechanism Housing Reliability

A presentation to the NRC by the Fort Calhoun Station Staff February 8, 2002, Arlington, Texas

### Agenda

- Introductions Gary Gates
- Agenda Review Ralph Phelps
- Summary of Fort Calhoun Station Initiatives - Joe McManis
- CEDM Housing Mechanics Discussion -Kirby Woods

#### Agenda

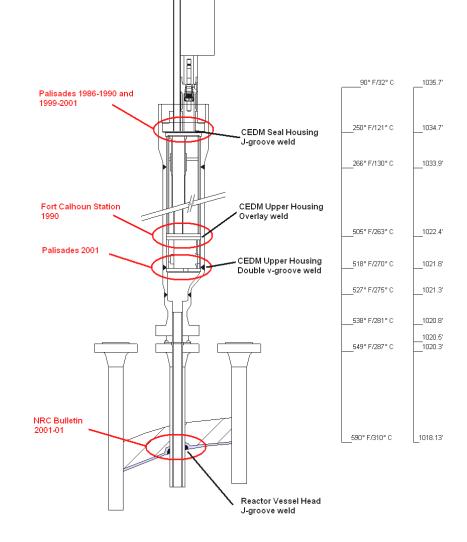
- CEDM Housing Materials Characterization Discussion - Bob Lisowyj
- Conclusions Ralph Phelps
- Closing Ross Ridenoure

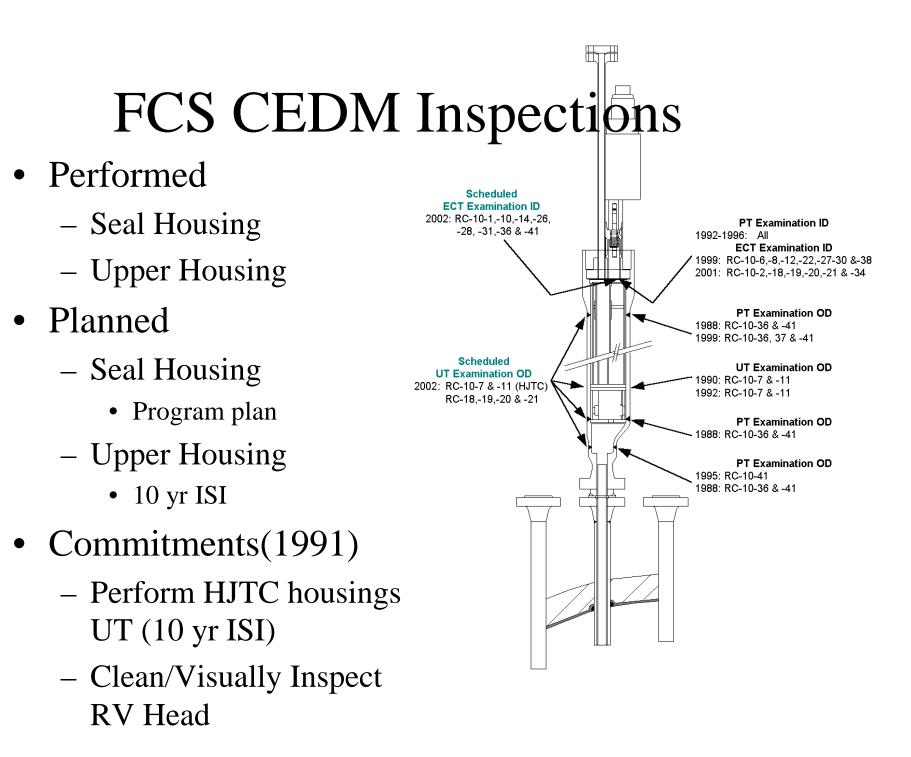
#### Joe McManis, P.E. - Manager, Design Engineering

Summary of Fort Calhoun Station Initiatives Regarding CEDM Reliability

# Operating Experience

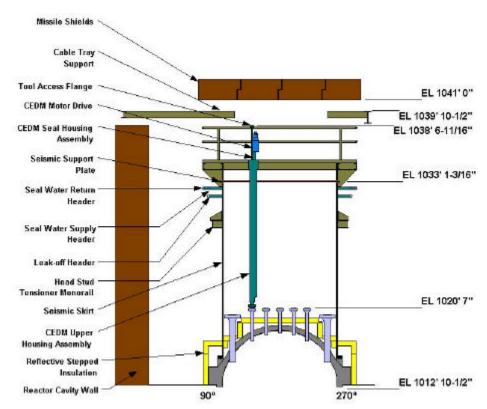
- CEDM Housing Assemblies
  - Seal Housing
    - J-groove weld
      - Palisades 1986-1990 & 1999-2001
  - Upper Housing
    - Overlay weld
      - FCS 1990
    - V-groove weld
      - Palisades 2001
  - VHP Nozzles
    - J-groove weld
- -INSIGHT-





# VHP Nozzle

- Oconee 3 Event
  - Circumferential Cracking
  - Bulletin 2001-01
- Effective Full Power Year
  - Time-at-Temperature
    Model
  - FCS 17.9 EFPY
- Low Probability of Rupture and/or Excessive Leakage
- Effective Visual Inspection during May



### FCS Susceptibility Assessment

- Environment
  - Chlorides
  - Oxygen
  - Stagnancy
- Residual Stress
  - Longitudinal
  - Circumferential
- Temperature
- Conclusion
  - Seal Housing
    Highest Risk

Susceptibility Assessment									
Description		Environment		Tensile Residual Stresses		Temp.			
Location	Туре	Chlorides	Oxygen	Axial	Ноор				
Seal Housing 1034'-7"	j-groove	High	High	Low	High	Moderate			
<b>Upper Housing</b> <i>Autoclave Flange</i> 1033'-9"	double v-groove	High	Moderate	Moderate <sup>1</sup>	N/A	Moderate			
Upper Housing Pipe 1022'-4"	overlay	Low	Low	Moderate <sup>1</sup>	Low	Moderate			
Modified Eccentric Reducer Large Diameter 1021'-8"	double v-groove	Low	Low	Moderate <sup>1</sup>	N/A	Moderate			
Modified Eccentric Reducer Small Diameter 1021'-3"	double v-groove	Low	Low	Moderate <sup>1</sup>	N/A	Moderate			

Note: 1 - This moderate stress peaks at the weld root and dissipates within the weld area.

### Key Differences

- Fort Calhoun Station has:
  - Time
    - CEDM through-wall cracks
  - Operation
    - Maintenance outage
  - Thermal Sleeve
    - Palisades extends to their reactor vessel flange
  - Program Plan

# Material Reliability Focus

- Operating Experience
  - Seal Housing
  - Upper Housing
- Program Plan
  - Predictive and Environmental Model
  - Inspection, Remediation and/or Repair Plan
- Overview of Issues
  - Responses
  - Actions

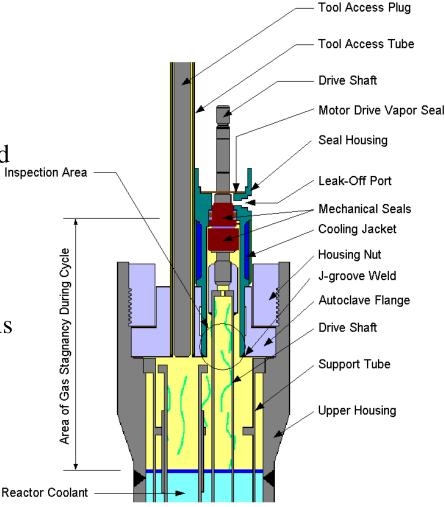
# Introduction of Program Managers

- Two Individuals Providing Oversight of the Issue
- Kirby Woods PE, MS, Worked on Issue since early 1998
- Bob Lisowyj PhD, Worked on Issue since 1990

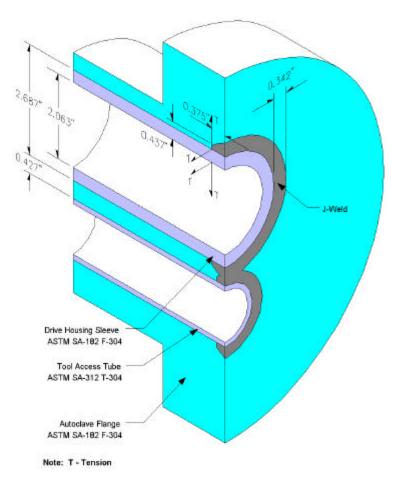
#### Kirby D. Woods - Senior Nuclear Design Engineer

CEDM Housing Mechanics Discussion

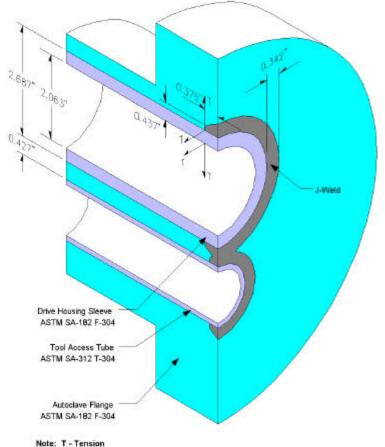
- Longest Stagnant Area
  - Oxygen
  - Leachable Chloride
    - Flexitallic gasket, o-rings and graphitar (~50 ppm)
- Residual Stress
  - Fabrication Records
    - Residual Stress (33 to 75 mils shrinkage)
- Temperature
- Operating Experience
- -PRECURSOR-



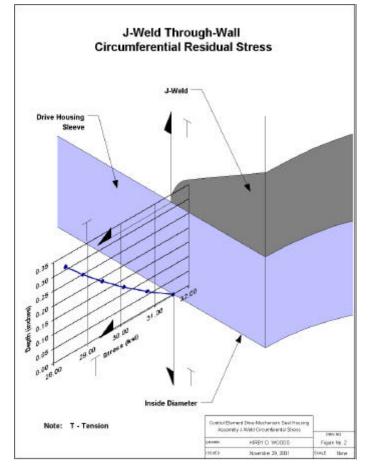
- Interpretation
  - Residual Stress
    - J-groove weld
    - Transverse shrinkage
      - Axial stress
      - Hoop stress
  - Temperature
    - ~250° F/121 ° C
      - Palisades experience
    - Inverted Bath Tub Curve



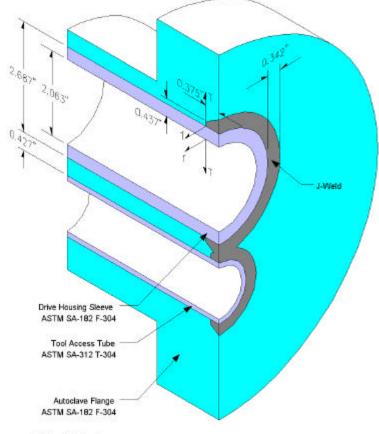




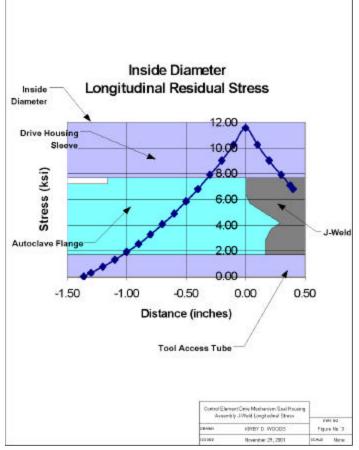
• Weld Induced Shrinkage

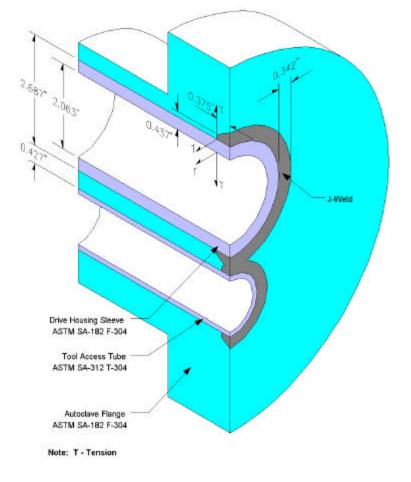


• Longitudinal Driver

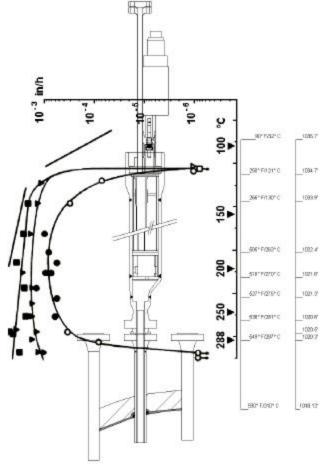


Note: T - Tension

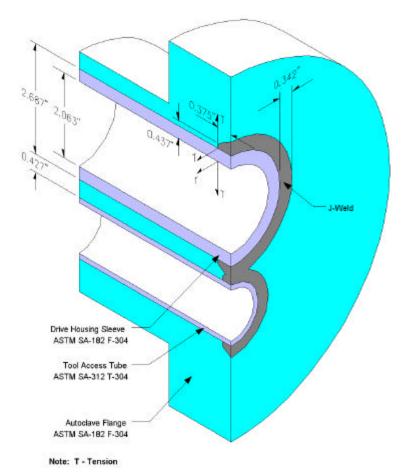




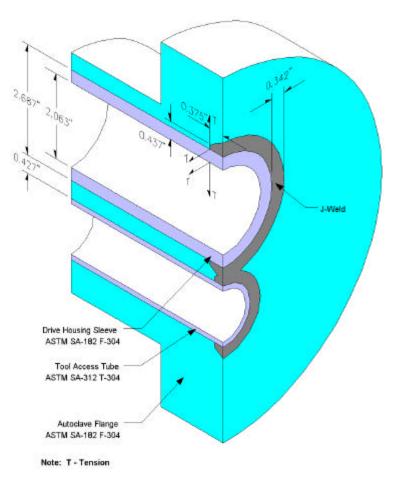
• Circumferential Driver



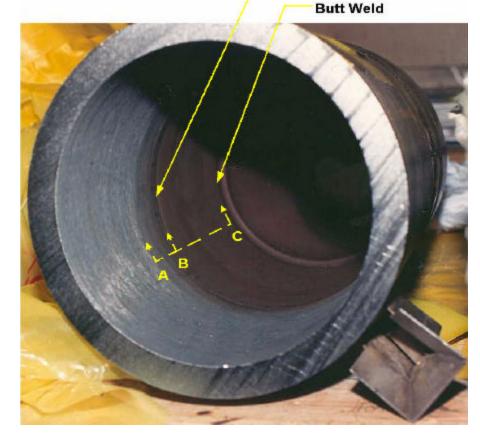
• Temperature Relationship



- Interpretation
  - Residual Stress
    - J-groove weld
    - Transverse shrinkage
      - Axial stress
      - Hoop stress
  - Temperature
    - ~250° F/121 ° C
      - Palisades experience
    - Inverted Bath Tub Curve
- High Susceptibility

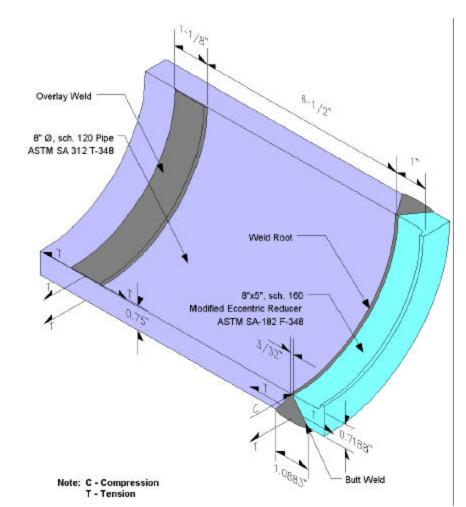


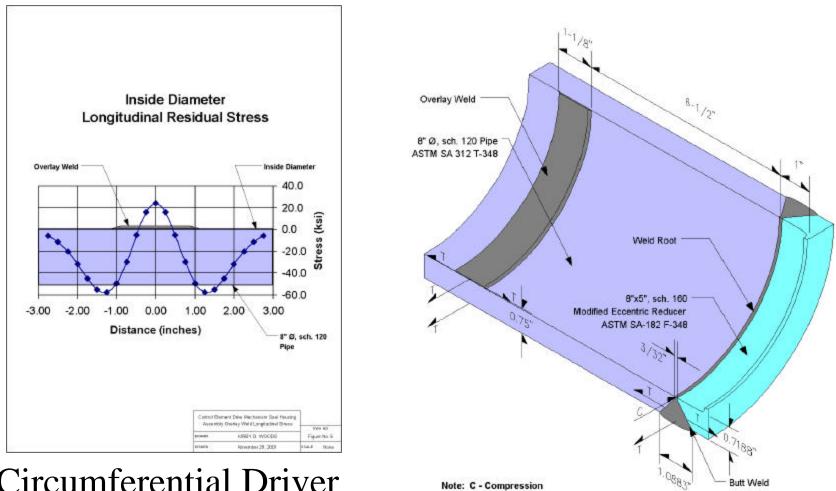
- Through-wall Crack 1990
  - non-vented housing
    - RC-10-9 & 13 Spare
    - 100% stagnant
    - Line 'B' showing maximum level of stagnancy
  - Source of Tensile Stress
    - Overlay Weld
- -WORST CASE-



**Overlay Weld** 

- Interpretation
  - Residual Stress
    - Overlay weld
    - Double v-groove weld
      - Weld pass
        - » jacketed
  - Temperature
    - ~505°F/263°C
      - FCS experience
      - ~50 °F delta

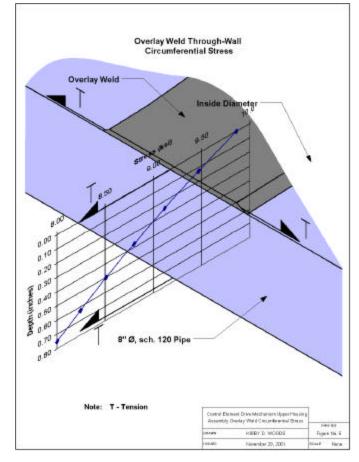




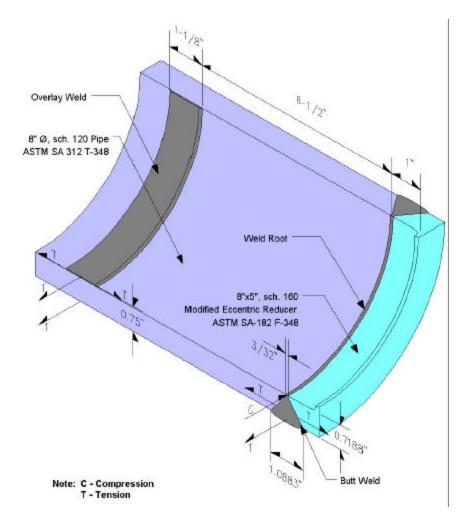
Note: C - Compression T - Tension

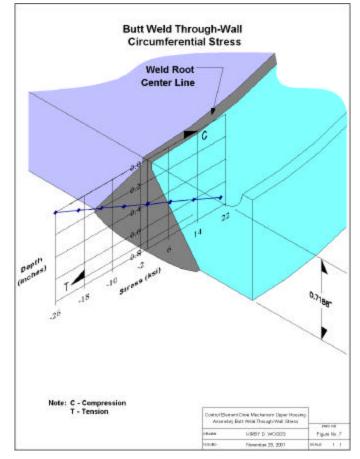
Butt Weld

• Circumferential Driver

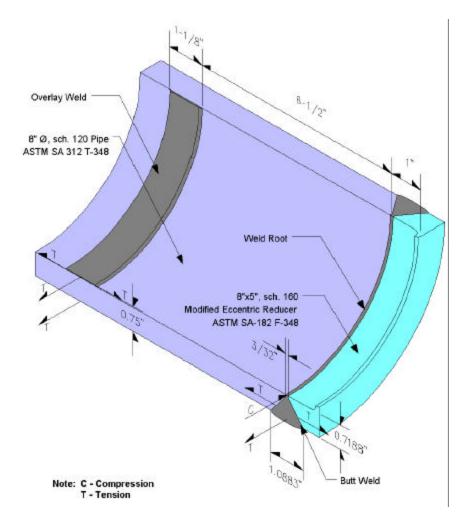


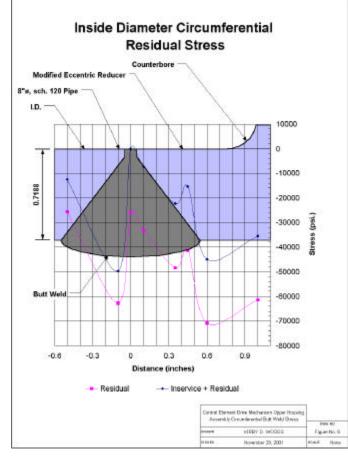
• Longitudinal Driver



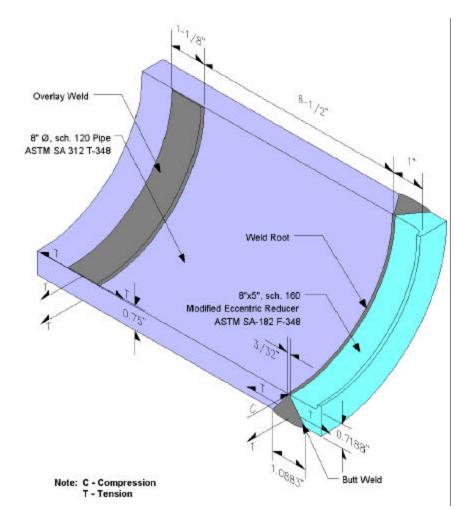


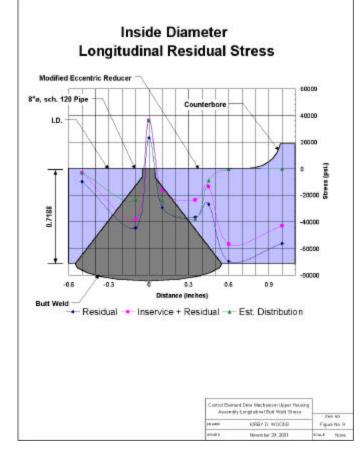
• Through-Wall Affect



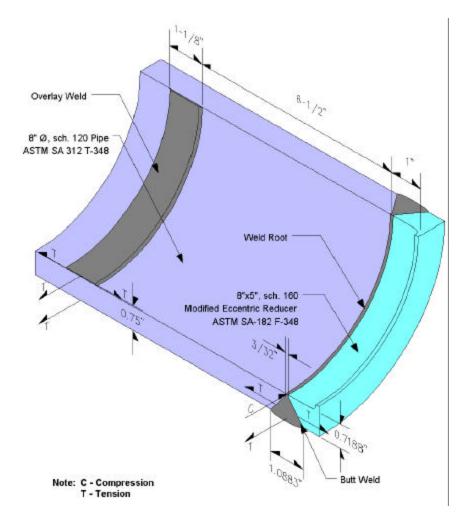


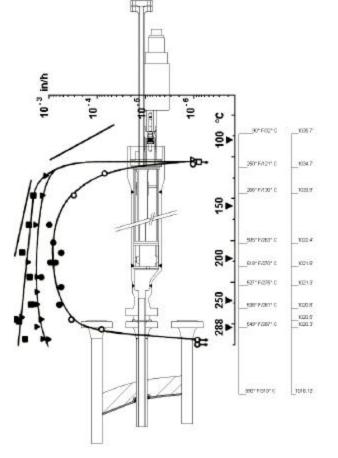
• Longitudinal Driver



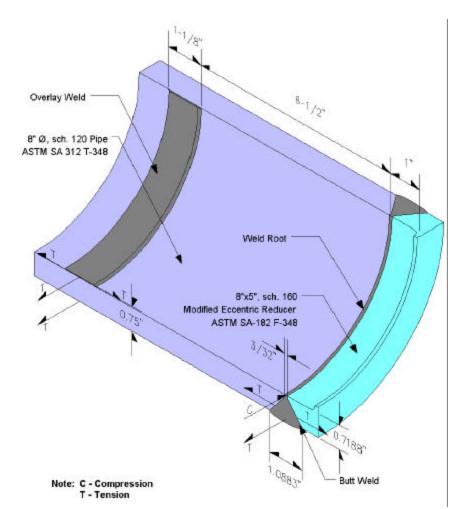


• Circumferential Driver

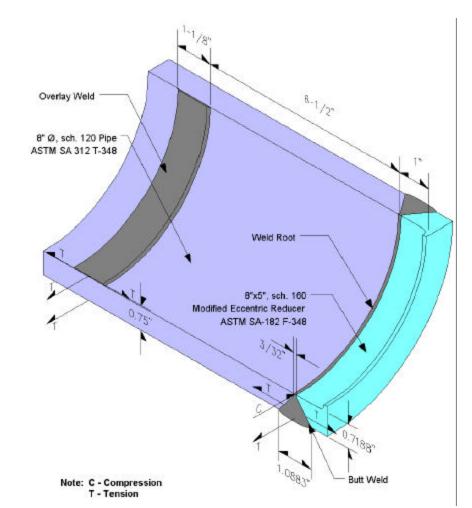




• Temperature Relationship



- Interpretation
  - Residual Stress
    - Overlay weld
    - Double v-groove weld
      - Weld pass
        - » jacketed
  - Temperature
    - ~505°F/263°C
      - FCS experience
      - ~50 °F delta
- Medium/Low Susceptibility



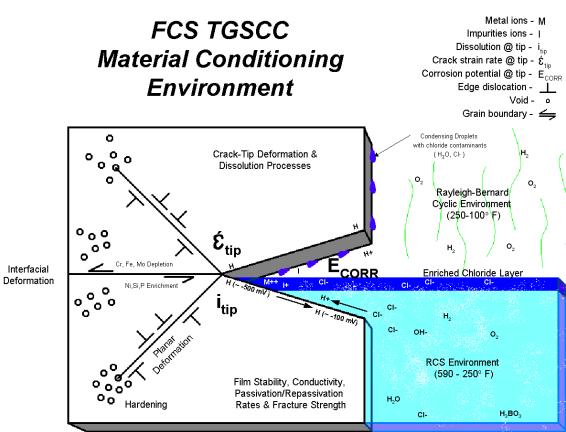
#### Bob Lisowyj - Senior Nuclear Design Engineer

CEDM Housing Materials Characterization Discussion

#### Program Plan

- Predictive
  - Crack Model
  - Environmental Model
- Plan
  - InspectionCriteria
  - Remediation
  - Repair

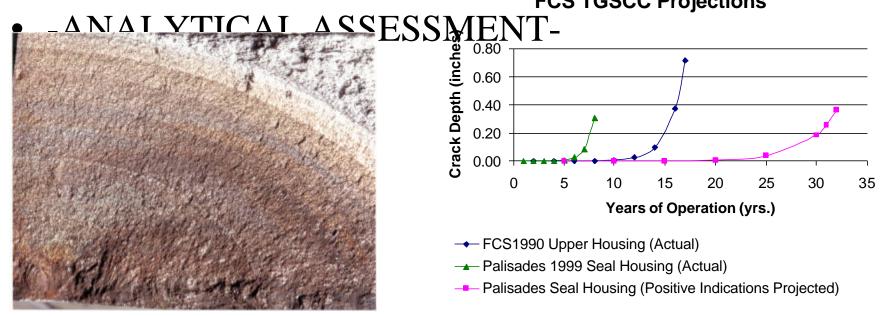




Woods - Lisowyj modified S. Bruemmer diagram

#### Predictive Crack Model

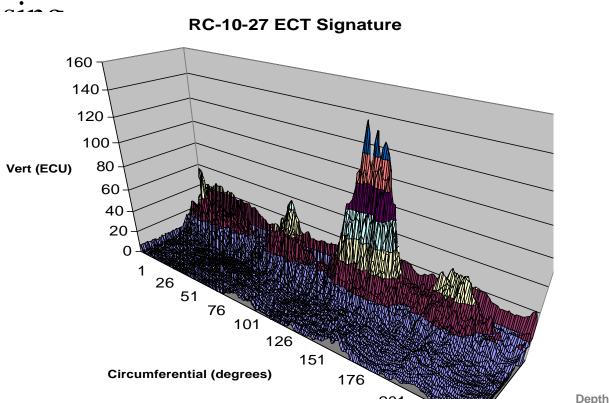
- Palisades and FCS Weibull Curves
  - Material difference Type 304 versus Type 347
- FCS Upper Housing Beachmarks
  - 10 yrs incubation & 7 yrs cracking FCS TGSCC Projections



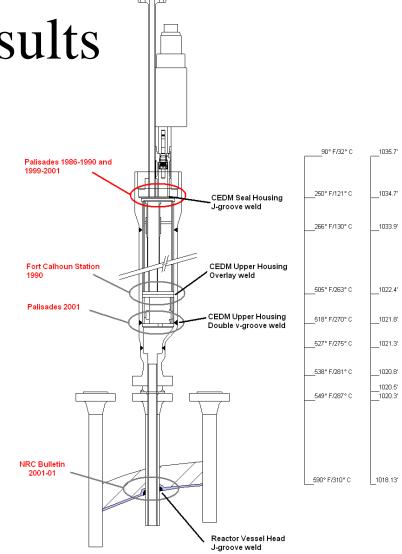
FCS Weld Overlay Failure

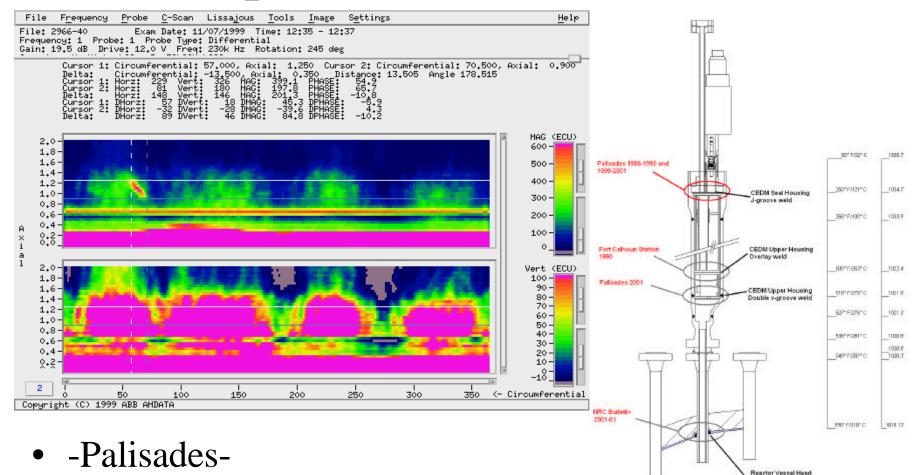
### FCS 1999 RFO Signature

- CEDM Seal Houring ECT Reported
  - Six of Eight Ins
  - Permeability Cl "
- FCS and Palisad
  - Similar
    - Geometry
    - Signatures
- Assessment
  - Possible SCC History
- -INSPECTION DATA-

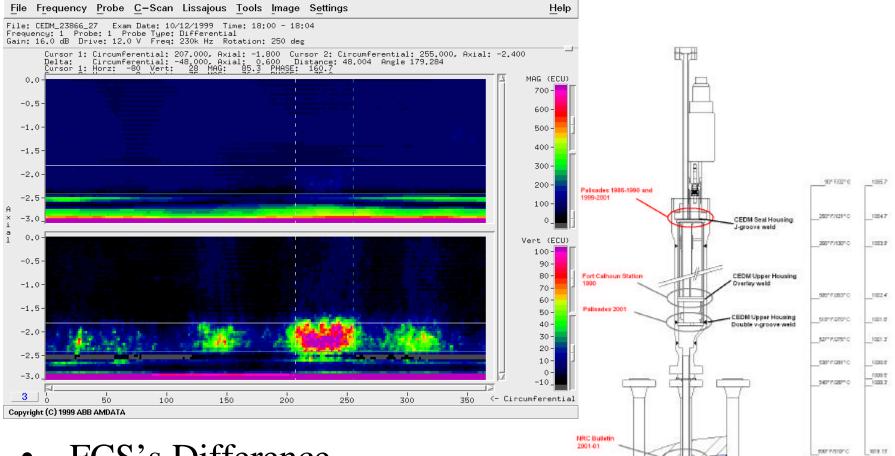


- Palisades ECT sample
  - Positive Signatures
    - Cracks
    - Permeability Areas
- FCS 1999 ECT examination
  - Positive Signatures
    - Permeability Areas





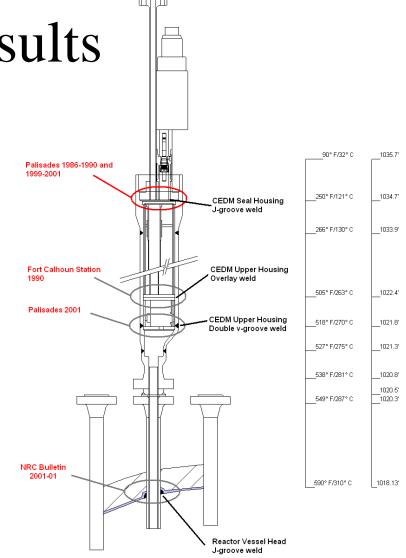
J-groove weld



Reactor Vessel Head J-groove weld

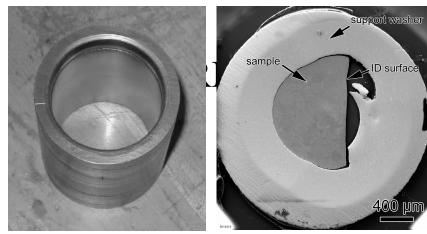
• -FCS's Difference-

- Palisades ECT sample
  - Positive Signatures
    - Cracks
    - Permeability Areas
- FCS 1999 ECT examination
  - Positive Signatures
    - Permeability Areas
- -QUESTIONING-

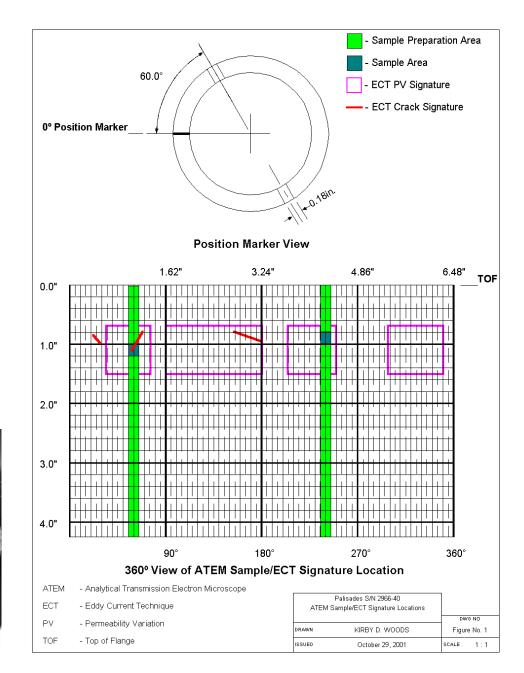


# Sample

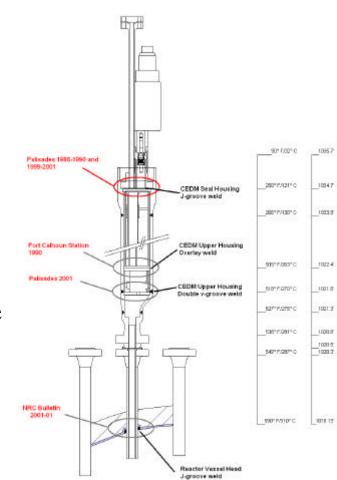
- EPRI assistance
- Validation of ECT technique
- Drive Housing Sleeve preparation for ATEM

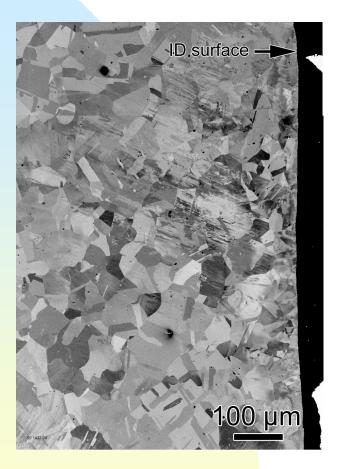


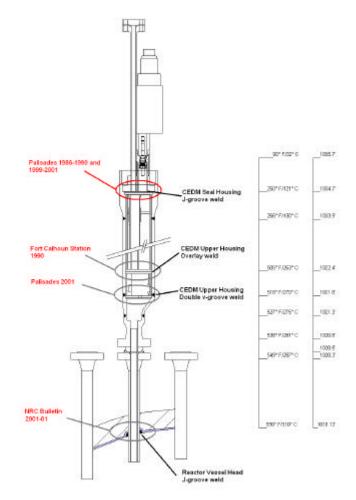
Palisades Drive Housing Sleeve



- Palisades Sample
  - Seal housing
    - SA312 F304
  - Scanning Electron Microscope
    - Transgranular
      - Small Branching
    - 0.095" deep
  - Energy Dispersive Spectroscope
    - Cl & S contaminants
    - Fe,Cr & Ni oxides



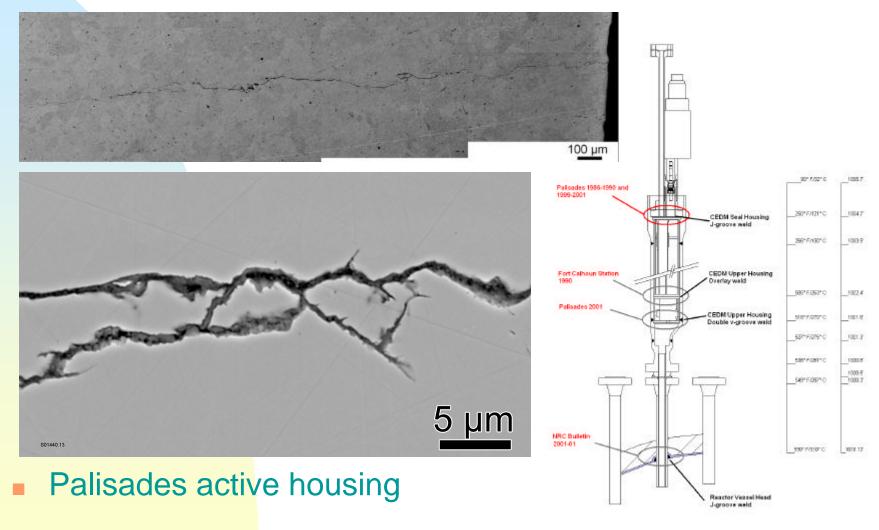




Palisades active housing

**OPPD** Presentation

2/08/02

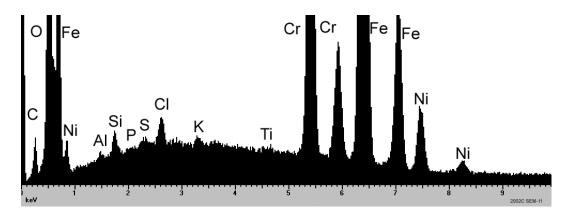


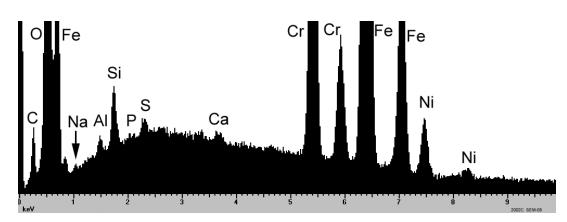
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#### **OPPD** Presentation

#### SEM/EDS Analyses

- Impurities
  - Na, Al, Si, Ca, K and Cl in spots
- Oxides
  - Si, S and CI near crack tip





Palisades active housing

2/08/02

**OPPD** Presentation

#### TEM/EDS Analyses

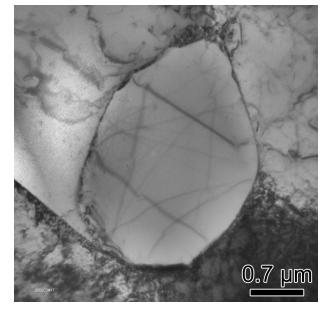
- 3 to 5 µm particles identified
- Delta ferrite pattern
  - Observation

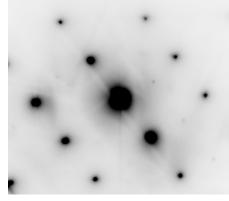
#### EDS Analyses: wt.%

SiCrMnFeNimatrix:0.620.22bal.9.2

ferrite: 0.5 28.4 1.6 bal. 4.8

Palisades active housing





011 Plane

2/08/02

**OPPD** Presentation