

Docket No. 50-261

April 4, 1984

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Mr. E. E. Utley, Executive Vice President
Power Supply and Engineering & Construction
Carolina Power and Light Company
Post Office Box 1551
Raleigh, North Carolina 27602

Dear Mr. Utley:

The Commission has issued the enclosed Amendment No. 79 to Facility Operating License No. DPR-23 for the H. B. Robinson Steam Electric Plant Unit No. 2. This amendment consists of changes to the Technical Specifications in response to your request dated January 11, 1983.

The amendment revises the Technical Specifications to reduce the containment internal pressure limit from 2 psig to 1 psig.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular monthly Federal Register notice.

Sincerely,

ORIGINAL SIGNED BY

Glode Requa, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosures:

1. Amendment No. 79 to DPR-23
2. Safety Evaluation

cc: w/enclosures
See next page

ORB#1:DL *CP*
CParrish
3/26/84

ORB#1:DL *GR*
GRequa:ps
3/23/84

G-ORB#1:DL
GRequa
3/26/84

ORB#1:DL
GRequa
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AD:DL
GLa'inas
4/2/84

Mr. E. E. Utley
Carolina Power and Light Company

H. B. Robinson Steam Electric
Plant 2

cc: G. F. Trowbridge, Esquire
Shaw, Pittman, Potts and Trowbridge
1800 M Street, N.W.
Washington, DC 20036

Regional Radiation Representative
EPA Region IV
345 Courtland Street, N.E.
Atlanta, GA 30308

Mr. McCuen Morrell, Chairman
Darlington County Board of Supervisors
County Courthouse
Darlington, South Carolina 29535

State Clearinghouse
Division of Policy Development
116 West Jones Street
Raleigh, North Carolina 27603

Attorney General
Department of Justice
Justice Building
Raleigh, North Carolina 27602

U.S. Nuclear Regulatory Commission
Resident Inspector's Office
H. B. Robinson Steam Electric Plant
Route 5, Box 4313
Hartsville, South Carolina 29550

James P. O'Reilly
Regional Administrator - Region II
U.S. Nuclear Regulatory Commission
101 Marietta Street
Atlanta, GA 30303

Mr. R. Morgan
General Manager
H. B. Robinson Steam Electric Plant
Post Office Box 790
Hartsville, South Carolina 29550



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CAROLINA POWER AND LIGHT COMPANY

DOCKET NO. 50-261

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 79
License No. DPR-23

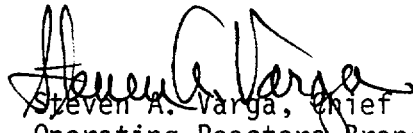
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Carolina Power and Light Company (the licensee) dated January 11, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-23 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.79 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 4, 1984

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 79 FACILITY OPERATING LICENSE NO. DPR-23

DOCKET NO. 50-261

Revise Appendix A as follows:

Remove Pages

3.6-1
3.6-2a

Insert Pages

3.6-1
3.6-2a

3.6 CONTAINMENT SYSTEM

Applicability

Applies to the integrity of reactor containment.

Objective

To define the operating status of the reactor containment for plant operation.

Specification

3.6.1 Containment Integrity

- a. The containment integrity (as defined in 1.7) shall not be violated unless the reactor is in the cold shutdown condition.
- b. The containment integrity shall not be violated when the reactor vessel head is removed unless a shutdown margin greater than 10% $\Delta k/k$ is constantly maintained.
- c. Positive reactivity changes shall not be made by rod drive motion when the containment integrity is not intact except for rod drop tests or disconnecting part length rods, in which case the shutdown margin is maintained $\geq 1\% \Delta k/k$.
- d. Positive reactivity changes shall not be made by boron dilution when the containment integrity is not intact unless the shutdown margin is maintained $\geq 1\% \Delta k/k$.

3.6.2 Internal Pressure

If the internal pressure exceeds 1 psi or the internal vacuum exceeds 1.0 psi, the condition shall be corrected within eight (8) hours or the

Regarding internal pressure limitations, the containment design pressure of 42 psig would not be exceeded if the internal pressure before a major loss-of-coolant accident were as much as 2 psig.⁽¹⁾ The containment is designed to withstand an internal vacuum of 2.0 psi.⁽²⁾

References

- (1) FSAR Section 14.3.4
- (2) FSAR Section 5.1.2.3

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 79 TO FACILITY OPERATING LICENSE NO. DPR-23
CAROLINA POWER AND LIGHT COMPANY
H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2
DOCKET NO. 50-261

Introduction

By letter dated January 11, 1983, the licensee proposed certain changes to the Technical Specifications for H. B. Robinson, Unit 2, concerning the allowable containment internal pressure. The licensee has proposed these changes to make the Technical Specifications consistent with the current safety analysis for the plant.

Technical Specification (TS) 3.6.2 currently requires that action be taken to reduce the containment internal pressure if it exceeds 2 psig, or to shut down the plant within eight hours. The 2 psig limit is based on a previous safety analysis which calculated a peak accident pressure inside containment of 37.8 psig. Since the containment design pressure is 42 psig, the pre-accident containment pressure could have been as high as 4 psig; when this initial value is added to the peak accident pressure (37.8 psig), the containment pressure would not exceed its design value. The TS limit of 2 psig was originally chosen to provide additional safety margin.

The licensee has since discovered an error in the calculation of the containment net free volume stated in the facility FSAR. This volume was recalculated and found to be less than originally stated. As a result, calculated peak accident pressure is now 40 psig; the previous value was 37.8 psig. Since the containment design pressure is 42 psig,

the containment internal pressure at the onset of a LOCA must be no more than 2 psig to prevent exceeding the containment design pressure.

The licensee has proposed that TS 3.6.2 be revised to reduce the containment internal pressure limit from 2 psig to 1 psig, to provide a 1 psig safety margin. The associated basis in the TS would also be revised to indicate that the TS 3.6.2 limit of 1 psig is based on using 50% of the calculated difference of 2 psig between the peak accident pressure and the design pressure.

The staff finds the proposed changes to the facility Technical Specifications to be consistent with the licensee's revised safety analysis and, therefore, acceptable.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be

endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: April 4, 1984

Principal Contributor:

J. Pulsipher