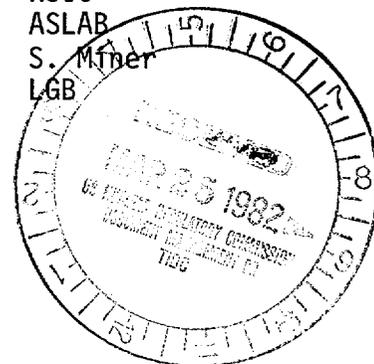


March 18, 1982

DISTRIBUTION

- Docket File
- NRC PDR
- Local PDR
- ORB 1 File
- D. Eisenhut
- C. Parrish
- W. Ross
- OELD
- OI&E (2)
- T. Barnhart (4)
- L. Schneider (1)
- D. Brinkman
- ACRS (10)
- Clare Miles (OPA)
- R. Diggs
- R. Ballard

NSIC
ASLAB
S. Miner
LGB



See Correction letter of
4/29/82

Docket No. 50-261

Mr. J. A. Jones
Vice President
Carolina Power and Light Company
336 Fayetteville Street
Raleigh, North Carolina 27602

Dear Mr. Jones:

The Commission has issued the enclosed Amendment No. 67 to Facility Operating License No. DPR-23 for the H. B. Robinson Steam Electric Plant, Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated May 9, 1980, as supplemented January 8, 1982.

The amendment revises the Radiological Technical Specifications to clarify the meaning of "Operable" and to incorporate a general action statement to complement those currently required to meet Limiting Conditions of Operation. The wording of the Technical Specification proposed in your January 8, 1982 letter has been discussed with members of your staff. With their approval, the wording has been modified by the inclusion of time limits for changing operational modes.

This amendment also revises the wording of Section 6.8.1 of the H. B. Robinson Unit 2 Technical Specifications to reflect the use of more recent versions of ANSI Standard N18.7 and Regulatory Guide 1.33 in the preparation of administrative policies and procedures. These changes were requested in your letter of May 9, 1980.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

51

William J. Ross, Project Manager
Operating Reactors Branch No. 1
Division of Licensing

Enclosures:

1. Amendment No. 67 to DPR-23
2. Safety Evaluation
3. Notice of Issuance

cc w/enclosures:
See next page

FR NOTICE
+
AMENDMENT

OFFICE	ORB 1	ORB 1	ORB 1	AD-OR	OELD	ORB 4	LGB
SURNAME	CParrish/rs	WRoss/rs	SVarga	Novak	M. KARMAK	SMiner/rs	nm
DATE	3/9/82	3/9/82	3/9/82	3/11/82	3/12/82	3/9/82	3/9/82

8204220072 820318
PDR ADDCK 05000261
P PDR

OFFICIAL RECORD COPY

Mr. J. A. Jones
Carolina Power and Light Company

cc: G. F. Trowbridge, Esquire
Shaw, Pittman, Potts and Trowbridge
1800 M Street, N.W.
Washington, D. C. 20036

James P. O'Reilly
Regional Administrator - Region II
101 Marietta Street Suite 3100
Atlanta, Georgia 30303

Hartsville Memorial Library
Home and Fifth Avenues
Hartsville, South Carolina 29550

Mr. McCuen Morrell, Chairman
Darlington County Board of Supervisors
County Courthouse
Darlington, South Carolina 29535

State Clearinghouse
Division of Policy Development
116 West Jones Street
Raleigh, North Carolina 27603

Attorney General
Department of Justice
Justice Building
Raleigh, North Carolina 27602

U. S. Nuclear Regulatory Commission
Resident Inspector's Office
H. B. Robinson Steam Electric Plant
Route 5, Box 266-1A
Hartsville, South Carolina 29550

Michael C. Farrar, Chairman
Atomic Safety and Licensing
Appeal Board Panel
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Richard S. Salzman
Atomic Safety and Licensing
Appeal Board Panel
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dr. W. Reed Johnson
Atomic Safety and Licensing
Appeal Board Panel
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Regional Radiation Representatives
EPA Region IV
345 Courtland Street, N.E.
Atlanta, Georgia 30308



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CAROLINA POWER AND LIGHT COMPANY

DOCKET NO. 50-261

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 67
License No. DPR-23

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Carolina Power and Light Company (the licensee) dated May 9, 1980, as supplemented January 8, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

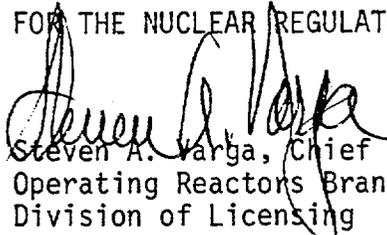
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-23 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 67, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch No. 1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 18, 1982

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 67 TO FACILITY OPERATING LICENSE NO. DPR-23

DOCKET NO. 50-261

Revise Appendix A as follows:

Remove Pages

1-2
3.1-1
6.8-1

Insert Pages

1-2
3.1-1
6.8-1

1.2.6 Refueling Operation

Any operation involving movement of core components when there is fuel in the containment vessel and the pressure vessel head is unbolted or removed.

1.2.7 Operating Basis Earthquake

The operating basis earthquake is that earthquake which involves a ground acceleration of 0.10 g horizontally and 0.067 g vertically.

1.2.8 Safe Shutdown Earthquake

The safe shutdown earthquake is that earthquake which involves a ground acceleration of 0.20 g horizontally and 0.133 g vertically.

1.3 OPERABLE - OPERABILITY

A system, subsystem, train, component or device shall be OPERABLE or have OPERABILITY when it is capable of performing its specified function(s). Implicit in this definition shall be the assumption that all necessary attendant instrumentation, controls, normal and emergency electrical power sources, cooling or seal water, lubrication or other auxiliary equipment that are required for the system, subsystem, train, component or device to perform its function(s) are also capable of performing their related support function(s).

When a system, subsystem, train, component or device is determined to be inoperable solely because its emergency power source is inoperable, or solely because its normal power source is inoperable, it may be considered OPERABLE for the purpose of satisfying the requirements of its applicable Limiting Condition for Operation, provided: (1) its corresponding normal or emergency power source is OPERABLE; and (2) all of its redundant system(s), subsystem(s), train(s), component(s) and device(s) are OPERABLE, or likewise satisfy the requirements of this specification.

1.4 PROTECTION INSTRUMENTATION CHANNEL

An arrangement of components and modules as required to generate a single protective action signal when required by a plant condition. A channel loses its identity where single action signals are combined.

1.5 DEGREE OF REDUNDANCY

The difference between the number of operable channels and the number of channels which when tripped will cause an automatic system trip.

3.0 LIMITING CONDITIONS FOR OPERATION

Except as otherwise provided for in each specification, if a Limiting Condition for Operation cannot be satisfied because of circumstances in excess of those addressed in the specification, the unit shall be placed in hot shutdown within eight hours and in COLD SHUTDOWN within the next 30 hours unless corrective measures are taken that permit operation under the permissible Limiting Condition for Operation statements for the specified time interval as measured from initial discovery or until the reactor is placed in a condition in which the specification is not applicable.

3.1 REACTOR COOLANT SYSTEM

Applicability

Applies to the operating status of the Reactor Coolant System.

Objective

To specify those Reactor Coolant System conditions which must be met to assure safe reactor operation.

Specification

3.1.1 Operational Components

3.1.1.1 Coolant Pumps

- a. At least one reactor coolant pump or the Residual Heat Removal System shall be in operation when a reduction is made in the boron concentration of the reactor coolant.
- b. When the reactor is critical, except for special low power tests during initial start-up testing, at least one reactor coolant pump shall be in operation.
- c. Reactor power shall not exceed 10% rated power unless at least two reactor coolant pumps are in operation.
- d. Reactor power will not exceed 45% of rated power with only two pumps in operation.
- e. A reactor coolant pump may be started (or jogged) only if there is a steam bubble in the pressurizer or the steam generator temperature is no higher than 50°F higher than the temperature of the reactor coolant system.

6.8 PROCEDURES

6.8.1 Written procedures and administrative policies shall be established, implemented, and maintained that meet or exceed the requirements and recommendations of Sections 5.2 and 5.3 of ANSI N18.7-1976 and Appendix "A" of USNRC Regulatory Guide 1.33 Rev. 2 dated February, 1978, except as provided in 6.8.2 and 6.8.3 below.

6.8.2 Proposed operating procedures, overall plant operating procedures, system descriptions, emergency procedures, fuel handling procedures, periodic test procedures, procedures for equipment maintenance which may affect nuclear safety, annunciator procedures, Fire Protection Program implementation procedures and any other procedures determined by the Plant Manager to affect nuclear safety, shall be reviewed by the PNSC and approved by the Plant Manager. Prior to implementation, proposed changes to these procedures must also be reviewed and approved in this manner.

6.8.3 Temporary changes to procedures of 6.8.2 above may be made provided:

- a. The intent of the original procedure is not altered.
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License.
- c. The change is documented, reviewed by the PNSC and approved by the Plant Manager within three weeks of implementation.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 67 TO FACILITY OPERATING LICENSE NO. DPR-23
CAROLINA POWER AND LIGHT COMPANY
H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2
DOCKET NO. 50-261

I. OPERABLE

Introduction

By letter dated April 10, 1980 the staff provided clarification to licensees regarding the usage of the terms "OPERABLE" and "OPERABILITY." Licensees were requested to revise their Technical Specifications to assure correct application of these terms in the operation of their plants. In their response of May 9, 1980, as supplemented by letter dated January 8, 1982, Carolina Power and Light Company (the licensee) proposed to revise the definition of these terms to include the key factors in the staff's clarification. After additional discussions on this subject, the licensee proposed to further revise the Robinson Unit No. 2 Technical Specifications to provide an additional, general, action statement to complement those currently required to meet the existing limiting Conditions of Operation.

Background

The NRC's Standard Technical Specifications (STS) were formulated to preserve the single failure criterion for systems that are relied upon in the safety analysis report. By and large, the single failure criterion is preserved by specifying Limiting Conditions for Operation (LCOs) that require all redundant components of safety related systems to be OPERABLE. When the required redundancy is not maintained, either due to equipment failure or maintenance outage, action is required, within a specified time, to change the operating mode of the plant to place it in a safe condition. The specified time to take action, usually called the equipment out-of-service time, is a temporary relaxation of the single failure criterion, which, consistent with overall system reliability considerations, provides a limited time to fix equipment or otherwise make it OPERABLE. If equipment can be returned to OPERABLE status within the specified time, plant shutdown is not required.

LCOs are specified for each safety related system in the plant, and with few exceptions, the ACTION statements address single outages of components, trains or subsystems. For any particular system, the LCO does not address multiple outages of redundant components, nor does it address the effects of outages of any support systems - such as electrical power or cooling water - that are relied upon to maintain the OPERABILITY of the particular system. This is because of the large number of combinations of these types of outages that are possible. Instead, the STS employ general specifications and an explicit definition of the term OPERABLE to encompass all such cases.. These provisions have been formulated to assure that no set of equipment outages would be allowed to persist that would result in the facility being in an unprotected condition.

Because of the importance of assuring safety system availability, the staff has concluded that all facility technical specifications should contain these requirements, and that appropriate procedures should be implemented to assure that the necessary records, such as plant logs or similar documents, are reviewed to determine compliance with these specifications (1) promptly upon discovering a component, train, or subsystem to be inoperable, and (2) prior to removing a component from service.

Evaluation

In the past, when an LCO could not be met at Robinson-2, the licensee met the requirements of §50.36(c)(2) in Part 10 of the Code of Federal Regulations by "proceeding to HOT SHUTDOWN and, subsequently, COLD SHUTDOWN utilizing normal operating procedures." This procedure allowed the final step to be taken over a 24-hour period, although the actual time could be as short as 2 to 3 hours. The licensee's commitment to shorten this step to eight hours provides the additional protection sought by the staff when redundant ECCS systems become inoperable - using the clarified interpretation of "OPERABLE." The licensee does not consider it necessary to reference the new ACTION in each specific technical specification related to ECCS components because the new ACTION statement will only be a modification of "normal operating procedures."

The staff finds the licensee's actions to be acceptable.

II. ADMINISTRATIVE PROCEDURES

This amendment also revises Section 6.8.1 of the Robinson-2 Technical Specifications to reflect the licensee's commitment in their May 9, 1980 letter to develop procedures and administrative policies to meet the requirements of later revisions of ANSI N18.7 (1976) and Regulatory Guide 1.33 (Revision 2). This action had been requested by Region II of the Office of Inspection and Enforcement so that the licensee's use of these updated standards in other commitments would be consistent.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: March 18, 1982

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-261CAROLINA POWER AND LIGHT COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 67 to Facility Operating License No. DPR-23 issued to Carolina Power and Light Company (the licensee), which revised Technical Specifications for operation of the H. B. Robinson Steam Electric Plant, Unit No. 2, (the facility) located in Darlington County, South Carolina. The amendment is effective as of the date of issuance.

The amendment revises the Radiological Technical Specifications to clarify the meaning of "OPERABLE" and to incorporate a general action statement to complement those currently required to meet Limiting Conditions of Operation and by updating the standards issued for developing procedures and administrative policies.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since this amendment does not involve a significant hazards consideration.

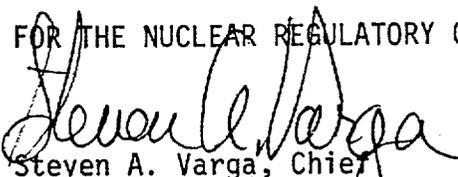
- 2 -

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated May 9, 1980, as supplemented January 8, 1982, (2) Amendment No. 67 to License No. DPR-23, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Hartsville Memorial Library, Home and Fifth Avenues, Hartsville, South Carolina 29550. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 18th day of March 1982.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch No. 1
Division of Licensing