

DECEMBER 5 1979

REGULATORY DOCKET FILE 50-261

DECEMBER 5 1979

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Docket No. 50-261

Mr. J. A. Jones  
Senior Vice President  
Carolina Power and Light Company  
336 Fayetteville Street  
Raleigh, North Carolina 27602

Dear Mr. Jones:

The Commission has issued the enclosed Amendment No. 45 to Facility Operating License No. DPR-23 for the H. B. Robinson Steam Electric Plant, Unit No. 2. The amendment consists of changes to the Technical Specifications and is in response to your requests dated March 6 and March 15, 1979.

The amendment consists of changes to the Technical Specifications which incorporate limiting conditions for operation and surveillance requirements for the control room ventilation system and editorial changes to the Technical Specifications for the Spent Fuel Building Filter System and the Containment Purge Filter System.

Copies of the Safety Evaluation and the Federal Register Notice are also enclosed.

Sincerely,

Original Signed By

A. Schwencer, Chief  
Operating Reactors Branch #1  
Division of Operating Reactors

Enclosures:

1. Amendment No. 45 to DPR-23
2. Safety Evaluation
3. Notice of Issuance

cc: w/enclosures  
See next page

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|---------|-------------|------------|-------------|---------|------------|
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| SURNAME | JDNeighbors | CS Parrish | WPGamill    | KARMAN  | ASchwencer |
| DATE    | 11/21/79:jb | 11/21/79   | 11/23/79    | 12/4/79 | 11/19/79   |



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

December 5, 1979

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Mr. J. A. Jones  
Senior Vice President  
Carolina Power and Light Company  
336 Fayetteville Street  
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The amendment consists of changes to the Technical Specifications which incorporate limiting conditions for operation and surveillance requirements for the control room ventilation system and editorial changes to the Technical Specifications for the Spent Fuel Building Filter System and the Containment Purge Filter System.

Copies of the Safety Evaluation and the Federal Register Notice are also enclosed.

Sincerely,

A handwritten signature in cursive script, appearing to read "A. Schwencer".

A. Schwencer, Chief  
Operating Reactors Branch #1  
Division of Operating Reactors

Enclosures:

1. Amendment No. 45 to DPR-23
2. Safety Evaluation
3. Notice of Issuance

cc: w/enclosures  
See next page

Mr. J. A. Jones  
Carolina Power and Light Company - 2 -

December 5, 1979

cc: G. F. Trowbridge, Esquire  
Shaw, Pittman, Potts and Trowbridge  
1800 M Street, N.W.  
Washington, D. C. 20036

U. S. Environmental Protection Agency  
Region IV Office  
ATTN: EIS COORDINATOR  
345 Courtland Street, N.E.  
Atlanta, Georgia 30308

Hartsville Memorial Library  
Home and Fifth Avenues  
Hartsville, South Carolina 29550

Mr. McCuen Morrell, Chairman  
Darlington County Board of Supervisors  
County Courthouse  
Darlington, South Carolina 29535

State Clearinghouse  
Division of Policy Development  
116 West Jones Street  
Raleigh, North Carolina 27603

Attorney General  
Department of Justice  
Justice Building  
Raleigh, North Carolina 27602

Michael C. Farrar, Chairman  
Atomic Safety and Licensing  
Appeal Board Panel  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Richard S. Salzman  
Atomic Safety and Licensing  
Appeal Board Panel  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Dr. W. Reed Johnson  
Atomic Safety and Licensing  
Appeal Board Panel  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Director, Technical Assessment Division  
Office of Radiation Programs (AK-459)  
U. S. Environmental Protection Agency  
Crystal Mall #2  
Arlington, Virginia 20460



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

CAROLINA POWER AND LIGHT COMPANY

DOCKET NO. 50-261

H. B. ROBINSON, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 45  
License No. DPR-23

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The applications for amendment by Carolina Power and Light Company (the licensee) dated March 6 and March 15, 1979 comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-23 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 45, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



A. Schwencer, Chief  
Operating Reactors Branch #1  
Division of Operating Reactors

Attachment:  
Changes to the  
Technical Specifications

Date of Issuance: December 5, 1979

ATTACHMENT TO LICENSE AMENDMENT NO. 45

FACILITY OPERATING LICENSE NO. DPR-23

DOCKET NO. 50-261

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by Amendment Number and contain vertical lines indicating the area of change.

Remove

ii  
3.8-3  
  
4.1-10  
4.12-1  
4.12-2  
4.12-3

Insert

ii  
3.8-3  
3.15-1  
3.15-2  
4.1-10  
4.12-1  
4.12-2  
4.12-3  
4.15-1  
4.15-2  
4.15-3

| <u>Section</u> | <u>Title</u>   | <u>Page</u> |
|----------------|--|-------------|
| 3.10.5         | Deleted  |             |
| 3.10.6         | Inoperable Control Rods  | 3.10-8      |
| 3.10.7         | Power Ramp Rate Limits   | 3.10-9      |
| 3.10.8         | Required Shutdown Margins  | 3.10-9      |
| 3.11           | Movable In-Core Instrumentation  | 3.11-1      |
| 3.12           | Seismic Shutdown   | 3.12-1      |
| 3.13           | Shock Suppressors (Snubbers)   | 3.13-1      |
| 3.14           | Fire Protection System   | 3.14-1      |
| 3.14.1         | Fire Detection Instrumentation   | 3.14-1      |
| 3.14.2         | Fire Suppression Water System  | 3.14-1      |
| 3.14.3         | CO <sub>2</sub> Fire Protection System                                       | 3.14-2      |
| 3.14.4         | Fire Hose Stations   | 3.14-2a     |
| 3.14.5         | Fire Barrier Penetration Fire Seals  | 3.14-3      |
| 3.15           | Control Room Filter System   | 3.15-1      |
| 4.0            | Surveillance Requirements  | 4.1-1       |
| 4.1            | Operational Safety Review  | 4.1-1       |
| 4.2            | Primary System Surveillance  | 4.2-1       |
| 4.3            | Primary System Testing Following Opening                                     | 4.3-1       |
| 4.4            | Containment Tests  | 4.4-1       |
| 4.4.1          | Operational Leakage Rate Tests   | 4.4-1       |
| 4.4.2          | Isolation Valve Tests  | 4.4-4       |
| 4.4.3          | Post Accident Recirculation Heat Removal System                              | 4.4-4       |
| 4.4.4          | Operational Surveillance Program   | 4.4-5       |
| 4.5            | Emergency Core Cooling, Containment Cooling and Iodine Removal Systems Tests | 4.5-1       |
| 4.5.1          | System Tests   | 4.5-1       |
| 4.5.2          | Component Tests  | 4.5-2       |
| 4.6            | Emergency Power System Periodic Tests  | 4.6-1       |
| 4.6.1          | Diesel Generators  | 4.6-1       |
| 4.6.2          | Diesel Fuel Tanks  | 4.6-2       |
| 4.6.3          | Station Batteries  | 4.6-2       |
| 4.7            | Secondary Steam and Power Conversion System                                  | 4.7-1       |
| 4.8            | Auxiliary Feedwater System   | 4.8-1       |
| 4.9            | Reactivity Anomalies   | 4.9-1       |
| 4.10           | Radioactive Effluents  | 4.10-1      |
| 4.11           | Reactor Core   | 4.11-1      |
| 4.12           | Refueling Filter Systems   | 4.12-1      |
| 4.13           | Shock Suppressors (Snubbers)   | 4.13-1      |
| 4.14           | Fire Protection System   | 4.14-1      |
| 4.15           | Control Room Filter System   | 4.15-1      |
| 5.0            | Design Features  | 5.1-1       |
| 5.1            | Site   | 5.1-1       |
| 5.2            | Containment  | 5.2-1       |
| 5.2.1          | Reactor Containment  | 5.2-1       |
| 5.2.2          | Penetrations   | 5.2-1       |
| 5.2.3          | Containment Systems  | 5.2-2       |
| 5.3            | Reactor  | 5.3-1       |
| 5.3.1          | Reactor Core   | 5.3-1       |
| 5.3.2          | Reactor Coolant System   | 5.3-2       |
| 5.4            | Fuel Storage   | 5.4-1       |
| 5.5            | Seismic Design   | 5.5-1       |

- j. If any of the specified limiting conditions for refueling are not met, refueling of the reactor shall cease; work shall be initiated to correct the conditions so that the specified limits are met; and no operations which may increase the reactivity of the core shall be made.
- k. The reactor shall be subcritical as required by 3.10.8.3 with  $T_{avg} \leq 140^{\circ}\text{F}$ .

3.8.2 The Spent Fuel Building filter system and the Containment Purge filter system shall satisfy the following conditions:

- a. The results of the in-place cold DOP and halogenated hydrocarbon tests at greater than 20 percent design flows on HEPA filters and charcoal adsorber banks shall show >99 percent DOP removal and >99 percent halogenated hydrocarbon removal.
- b. Verification by way of a laboratory carbon sample analysis from the Spent Fuel Building filter system carbon and the Containment Purge filter system carbon to show >90 percent radioactive methyl iodine removal in accordance with test 5.b of Table 5-1 of ANSI/ASME N509-1976 except that >70 percent relative humidity air is required.
- c. All filter system fans shall be shown to operate within +10% of design flow.
- d. During fuel handling operations, the relative humidity (R.H.) of the air processed by the refueling filter systems shall be <70 percent.
- e. From and after the date that the Spent Fuel Building filter system is made or found to be inoperable for any reason, fuel handling operations in the Spent Fuel Building shall be terminated immediately.

### 3.15 CONTROL ROOM FILTER SYSTEM

#### Applicability

Applies to the Control Room filter system which is required for the safe operation of the plant. This system incorporates both HEPA filters and a charcoal adsorber bank.

#### Objective

To provide limiting conditions for operation which ensure the operability of the filter system during plant operation, such that normal operation or accidental plant conditions requiring operation of the system will not result in consequences more severe than those previously analyzed.

#### Specification

3.15.1 During all modes of operation, except cold shutdown, the Control Room filter system shall be capable of performing its intended function in the required manner, except as described below:

- a. If the system is determined to be inoperable, it shall be returned to operable status within seven days, or in lieu of any other report, prepare a Special Report which shall be submitted to the Commission within the next 14 days. This report shall outline the cause of the inoperability, the corrective actions taken, and the plans and schedule for restoring the system to an operable status.

3.15.2 If the system is determined to be inoperable while the reactor is in cold shutdown, the system shall be made operable prior to reactor startup.

#### Basis

Operability of the Control Room filter system ensures that the Control Room will remain habitable during an accidental atmospheric radiation

release to the extent that none of the occupants would receive a personnel radiation exposure in excess of 10 percent of the suggested limits in 10CFR100<sup>(1)</sup>. Because the system's protection is required only during low probability events, the system may be out of service for 7 days for repairs. Following this period, a Special Report detailing the status of the system will be submitted to the Commission. Since reactor startup should not commence without this system in service, the specification prohibits startup with the system inoperable.

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(1) FSAR Section 7.7.1

TABLE 4.1-3 (Continued)

|     | <u>Check</u>   | <u>Frequency</u>  | <u>Maximum<br/>Time<br/>Between<br/>Tests</u> |         |
|-----|--|---|---|---------|
| 13. | Turbine Inspection   | Visual, Magnaflux<br>and Die Penetrant  | Every five years                              | 6 years |
| 14. | Fans and<br>Associated Char-<br>coal and Absolute<br>Filters for Con-<br>trol Room and<br>Residual Heat<br>Removal<br>Compartments<br>(HVE-19, HVE-5a<br>and 5b respec-<br>tively) | Fans functioning.<br>Laboratory tests on<br>charcoal must show<br>> 99% iodine removal.<br>In-place test must<br>show > 99% removal of<br>polydispersed DOP<br>particles by the HEPA<br>filters and Freon by<br>the charcoal filters. | Once per operating cycle.                     | NA      |
| 15. | Isolation Seal<br>Water System   | Functioning   | Each refueling shutdown                       | NA      |

\*NA - Not applicable

## 4.12 REFUELING FILTER SYSTEMS

### Applicability

Applies to fans and associated charcoal adsorber banks and HEPA filters for Spent Fuel Building filter system and Containment Purge filter system.

### Objective

To verify that the refueling filter systems will adequately remove radioactivity that may be released accidentally into the Spent Fuel Building and Containment Building.

### Specification

- 4.12.1 At least once per operating cycle, the following conditions shall be demonstrated:
- a. Pressure drop across the combined HEPA filters and charcoal adsorber banks is less than 6 inches of water at system design flow rate.
  - b. Fan capacity shall be within  $\pm 10\%$  of the design flow.
- 4.12.2
- a. The tests of Specification 3.8.2.a for the refueling filter systems shall be performed initially, and at least once per operating cycle prior to each refueling outage operation or after every 720 hours of system operation whichever occurs first.
  - b. The tests and sample analysis of Specification 3.8.2.b for the refueling filter systems shall be performed initially, at least once per operating cycle prior to

each refueling outage operation or after every 720 hours of system operation, whichever occurs first, and following significant painting, fire, or chemical release in any ventilation zone communicating with the filter system.

- c. Cold DOP testing shall be performed after each complete or partial replacement of a HEPA filter bank or after any structural maintenance of the filter system housing.
- d. Halogenated hydrocarbon testing shall be performed after each complete or partial replacement of a charcoal adsorber bank or after any structural maintenance on the filter system housing.
- e. A uniform air distribution within  $\pm 20\%$  across HEPA filters and charcoal adsorbers must be demonstrated initially and after each major repair or modification to the systems which would affect the air distribution.

4.12.3 The relative humidity of the air processed by the refueling filter system shall be monitored hourly during fuel handling operations.

#### Basis

Pressure drop across the combined HEPA filters and charcoal adsorbers of less than 6 inches of water at the system design flow rate will indicate that the filters and adsorbers are not clogged by excessive amounts of foreign matter. Pressure drop and fan capacity should be determined at least once per operating cycle to show system performance capability.

The frequency of tests and sample analysis are necessary to show that the HEPA filters and charcoal adsorbers can perform as evaluated under postulated accident conditions. The charcoal adsorber efficiency test procedures should allow for the removal of one adsorber tray, emptying of one bed from the tray, mixing the adsorbent thoroughly and obtaining at least two samples. Each sample should be at least two inches in diameter and a length equal to the thickness of the bed. If test results are unacceptable, all adsorbent in the system shall be replaced

with an adsorbent qualified according to Table 5.1 of ANSI/ASME N509-1976. The replacement tray for the adsorber tray removed for the test should meet the same adsorbent quality. Tests of the HEPA filters with DOP aerosol shall be performed in accordance to ANSI N101.1. Any HEPA filters found defective shall be replaced with filters qualified pursuant to Regulatory Position C.3.d of Regulatory Guide 1.52.

The Containment Purge filter system is normally run continuously during the entire refueling outage to provide cooling and ventilation and periodically during plant operation to reduce airborne radioactivity leaks inside the containment. Operation time of the Containment Purge filter system after the fuel handling operation is completed should not be added to the operation time during fuel handling operations for determination of testing and surveillance requirements given in these specifications.

If significant painting, fire, or chemical release occurs such that the HEPA filter or charcoal adsorber could become contaminated from the fumes, chemicals, or foreign material, the same laboratory tests and sample analysis shall be performed as required for operational use. The determination of significant shall be made by the operator on duty at the time of the incident. Knowledgeable staff members should be consulted prior to making this determination.

The relative humidity of the Containment atmosphere and air downstream of the heaters in the Spent Fuel Building filter system shall be monitored at least hourly to assure that the R.H. is less than 70 percent during fuel handling and Containment Purge filter system operation.

## 4.15 CONTROL ROOM FILTER SYSTEM

### Applicability

Applies to the fan, associated charcoal adsorber bank, and HEPA filters of the Control Room filter system.

### Objective

To verify that the Control Room filter system will adequately remove radioactivity from the incoming ambient air should there be an accidental radiation release to the atmosphere.

### Specification

- 4.15.1 At least once per operating cycle or after 720 hours of system operation, whichever comes first, and (1) after structural modifications on the HEPA filter or charcoal adsorber housing which would adversely affect the air flow distribution and (2) following significant painting, fire, or chemical release in any ventilation zone communicating with the system, the following tests shall be performed:
- a. Verify that the system flow rate is equal to the design flow rate  $\pm$  10 percent.
  - b. Verify that the charcoal adsorbers remove  $\geq$  99 percent of a halogenated hydrocarbon refrigerant test gas when they are tested in-place while the ventilation system is operating at a flow equal to the design flow  $\pm$  10 percent.
  - c. Verify that the HEPA filter banks remove  $\geq$  99 percent of the DOP when they are tested in-place in accordance with ANSI N101.1 (1972) while operating the ventilation system at a flow equal to the design flow  $\pm$  10 percent.

d. Verify by way of a laboratory test that the system's carbon demonstrates a methyl iodine removal efficiency of  $\geq 90$  percent. The test shall be conducted in accordance with ANSI N509-1976, Table 5-1, Test 5b. The required carbon samples may be obtained by the following methods:

1. One sample obtained from a test canister designed to ANSI N509-1976. The sample must be at least two inches in diameter and with a length equal to or greater than the thickness of the cell's adsorber bed.
2. Two samples obtained by emptying an adsorber cell and mixing the carbon thoroughly. The samples must be at least two inches in diameter and with a length equal to or greater than the thickness of the cell's adsorber bed.

4.15.2 At least once per operating cycle, the following test shall be performed:

- a. Verify that the pressure drop across the combined HEPA filters and charcoal adsorber bank is  $< 6$  inches Water Gauge at system design flow rate  $\pm 10$  percent.
- b. Verify that on a containment isolation test signal, the system automatically switches into a recirculation mode of operation with flow through the HEPA filters and charcoal adsorber banks.

4.15.3 After each complete or partial replacement of the carbon adsorber bank, perform the tests under Specification 4.15.1b.

4.15.4 After each complete or partial replacement of the HEPA filter bank, perform the tests under Specification 4.15.1c.

4.15.5 The associated fan unit in the Control Room filter system shall be verified operable monthly.

### Basis

Determination that the system is operating at design flow  $\pm$  10 percent indicates that the fan is operating at or near the design point on its operating curve. Operation of the fan at flows significantly different from the design flow will change the removal efficiency of the HEPA filters and carbon adsorbers.

The frequency of in-place testing and sample analysis are necessary to show that the HEPA filters and charcoal adsorbers can perform as evaluated under postulated accident conditions. Any HEPA filters found defective shall be replaced with the filters qualified pursuant to Regulatory Position C.3.d of NRC Regulatory Guide 1.52. If the carbon fails to pass the laboratory test, all adsorbent in the system shall be replaced with an adsorber qualified according to Table 5.1 of ANSI N509-1976.

If significant painting, fire, or chemical release occurs such that the HEPA filters or carbon adsorbers could become contaminated from the fumes, chemicals, or foreign material, the same in-place testing and sample analysis shall be performed as required for operational use. The determination of whether the incident is significant enough to warrant the testing shall be made by the Shift Foreman on duty. Knowledgeable staff members should be consulted prior to making this determination.

A pressure drop across the combined HEPA filters and charcoal adsorbers of less than 6 inches of water at the system design flow rate  $\pm$  10 percent will indicate that the filters and adsorbers are not clogged by excessive amounts of foreign matter.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 45 TO FACILITY

OPERATING LICENSE NO. DPR-23

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2

DOCKET NO. 50-261

Introduction

By letters of March 6 and March 15, 1979, Carolina Power and Light Company (the licensee) requested changes to the Technical Specifications to Facility Operating License No. DPR-23 for the H. B. Robinson Steam Electric Plant, Unit No. 2. The proposed Technical Specifications would change the limiting conditions for operation and the surveillance requirements for the spent fuel building filter system and the containment purge filter system and add requirements for the control room filter system.

Discussion and Evaluation

The proposed changes to the Technical Specifications submitted by the licensee at our request assure that the assumptions used by the NRC staff for the control room dose analysis are appropriate.

Technical Specification 3.15 would require the control room filter system to be operable during all modes of reactor operation, except cold shut-down, to ensure that the control room will remain habitable during an accidental atmospheric radioactivity release. Technical Specification 4.15 would specify the surveillance requirements to verify that the control room filter system is operable and would provide the degree of protection assumed in the Safety Analysis on habitability of control room during postulated accidents. Technical Specification 4.15 also would require the verification of system response upon a containment isolation signal such that the system switches automatically into a recirculation mode of operation with flow through the HEPA filters and charcoal adsorber banks. The proposed control room filter system Technical Specifications are determined to be acceptable to satisfy the staff's concern on the operability and surveillance requirements necessary to support the staff's assumptions used to analyze control room habitability during postulated accidents.

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The licensee has also proposed to modify Technical Specification 3.8.2.b to require the laboratory analysis of carbon samples of the spent fuel building filter system and the containment purge filter system be performed in accordance with the test method of 5.b of Table 5-1 of ANSI/ASME N509-1976 with the exception that the relative humidity of air be required to be >70. The requirement of the relative humidity would not be changed from the existing Technical Specifications. To require the test to be in accordance with the ANSI/ASME standard is acceptable for the demonstration of methyl iodide removal efficiency to be equal or greater than 90 percent. We conclude that these changes are acceptable.

#### Environmental Considerations

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact, and pursuant to 10 CFR 51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

#### Conclusion

We have concluded, based on the consideration discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: December 5, 1979

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-261CAROLINA POWER AND LIGHT COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITYOPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 45 to Facility Operating License No. DPR-23, issued to Carolina Power and Light Company, which revised Technical Specifications for operation of the H. B. Robinson Unit No. 2 (the facility) located in Darlington County, South Carolina. The amendment is effective as of its date of issuance.

The amendment consists of changes to the Technical Specifications which incorporate limiting conditions for operation and surveillance requirements for the control room ventilation system and editorial changes to the Technical Specifications for the Spent Fuel Building Filter System and the Containment Purge Filter System.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

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- 2 -

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §1.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

For further details with respect to this action, see (1) the applications for amendment dated March 6 and March 15, 1979, (2) Amendment No. 45 to License No. DPR-23, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C. and at the Hartsville Memorial Library, Home and Fifth Avenues, Hartsville, South Carolina 29550. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 5th day of December, 1979.

FOR THE NUCLEAR REGULATORY COMMISSION



A. Schwencer, Chief  
Operating Reactors Branch #1  
Division of Operating Reactors