

June 9, 1977

Docket No.: 50-261

Carolina Power & Light Company
ATTN: Mr. J. A. Jones
Senior Vice President
336 Fayetteville Street
Raleigh, North Carolina 27602

Gentlemen:

The Commission has issued the enclosed Amendment No. 28 to Facility Operating License No. DPR-23 for the H. B. Robinson Steam Electric Plant Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your application dated April 26, 1977 and staff discussions.

This amendment changes the Technical Specifications to reflect a change in your organizational structure and to correct a typographical error.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

[Signature]

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Enclosures:

- 1. Amendment No. 28
- 2. Safety Evaluation
- 3. Notice

cc w/enclosures: See next page

*R
Carter*

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SURNAME →	RIngram	GZwetzig:rm	A. Mittell	RReid		
DATE →	5/27/77	5/27/77	6/8/77	6/9/77		

Carolina Power & Light Company

cc w/enclosure(s):
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cc w/enclosures and incoming
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Office of Intergovernmental Relations
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Raleigh, North Carolina 27603

Hartsville Memorial Library
Home and Fifth Avenue
Hartsville, South Carolina 29550

Mr. McCuen Morrell, Chairman
Darlington County Board of Supervisors
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Chief, Energy Systems
Analyses Branch (AW-459)
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U. S. Environmental Protection Agency
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Washington, D.C. 20460

U. S. Environmental Protection Agency
Region IV Office
ATTN: EIS COORDINATOR
245 Courtland Street, NE
Atlanta, Georgia 30308



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CAROLINA POWER & LIGHT COMPANY

DOCKET NO. 50-261

H. B. ROBINSON STEAM ELECTRIC PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 28
License No. DPR-23

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Carolina Power & Light Company (the licensee) dated April 26, 1977, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

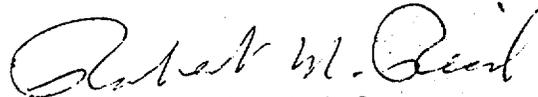
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-23 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 28, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 9, 1977

ATTACHMENT TO LICENSE AMENDMENT NO. 28

FACILITY OPERATING LICENSE NO. DPR-23

DOCKET NO. 50-261

Revise Appendix A Technical Specifications as follows:

Remove Pages

6-8

6-11 - 6-13

Figure 6.2-1

Insert Pages

6-8

6-11 - 6-13

Figure 6.2-1

The changed areas on the revised pages are shown by marginal lines.

6.5.2.2 Responsibility

The Manager - Corporate Nuclear Safety under the Vice President - System Planning & Coordination Department is charged with the overall responsibility for administering the independent off-site nuclear safety review program as follows:

- a. Approves selection of the person or persons to conduct off-site safety reviews.
- b. Has access to the plant operating records and operating personnel in performing the independent reviews.
- c. Prepares and retains written records of reviews.
- d. Assures independent safety review is conducted on all items required by Section 6.5.2.4.
- e. Distributes reports and other records to appropriate managers.

6.5.2.3 Personnel

- a. Personnel assigned responsibility for independent reviews shall be specified in technical disciplines, and shall collectively have the experience and competence required to review problems in the following areas:
 1. Nuclear power plant operations
 2. Nuclear engineering
 3. Chemistry and radiochemistry
 4. Metallurgy
 5. Instrumentation and control
 6. Radiological safety

- b. Recommendations and concerns will be submitted to the Manager - Nuclear Generation within 14 days of determination.
- c. A summation of Corporate Nuclear Safety Section recommendations and concerns will be submitted to the Company Chief Executive Officer; Chief Operating Officer; Senior Vice President - Power Supply; Vice President - System Planning & Coordination; Plant Manager and others, as appropriate on at least a bi-monthly frequency.

6.5.2.6 The Corporate Nuclear Safety review program shall be conducted in accordance with written, approved procedures.

6.5.3 Independent Off-Site Quality Assurance Audit Program

6.5.3.1 Purpose

Audits of activities shall be performed under the cognizance of the Corporate Quality Assurance Audit (CQAA) Section. These audits shall encompass:

- a. The conformance of facility operation to all provisions contained within the Technical Specifications and applicable license conditions at least once per year.
- b. The training and qualifications of the entire facility staff at least once per year.
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems, or method of operation that affect nuclear safety at least once per six months.
- d. The verification of compliance and implementation of the requirements of the Quality Assurance Program to meet the criteria of Appendix "B", 10CFR50, at least once per two years.
- e. The Emergency Plan and implementing procedures at least once per two years.

- f. The Security Plan and implementing procedures at least once per two years.
- g. Any other area of facility operation considered appropriate by the Corporate Quality Assurance Audit Section or the Senior Vice President - Power Supply.

6.5.3.2 Responsibility

The Manager - Corporate Quality Assurance Audit is charged with the overall responsibility for the corporate quality assurance audit program as follows:

- a. Selects auditors.
- b. Has access to records and personnel necessary in performing the audits.

6.5.3.3 Personnel

- a. Audit personnel will be independent of the area audited. Selection for auditing assignments is based on experience or training which establishes that their qualifications are commensurate with the complexity or special nature of the activities to be audited. In selecting auditing personnel, consideration will be given to special abilities, specialized technical training, prior pertinent experience, personal characteristics, and education.
- b. Qualified outside consultants or other individuals within organizations reporting to the Chief Operating Officer will be used to augment the audit teams when necessary.

6.5.3.4 Reports

Results of audit are approved by the Manager - Corporate Quality Assurance Audit and transmitted directly to the Company Chief Executive Officer and

the Chief Operating Officer as well as to the Senior Vice President - Power Supply, Vice President - System Planning & Coordination, and others as appropriate within 30 days after the completion of the audit.

6.5.3.5 The corporate quality assurance audit program shall be conducted in accordance with written, approved procedures.

6.6 Reportable Occurrence Action

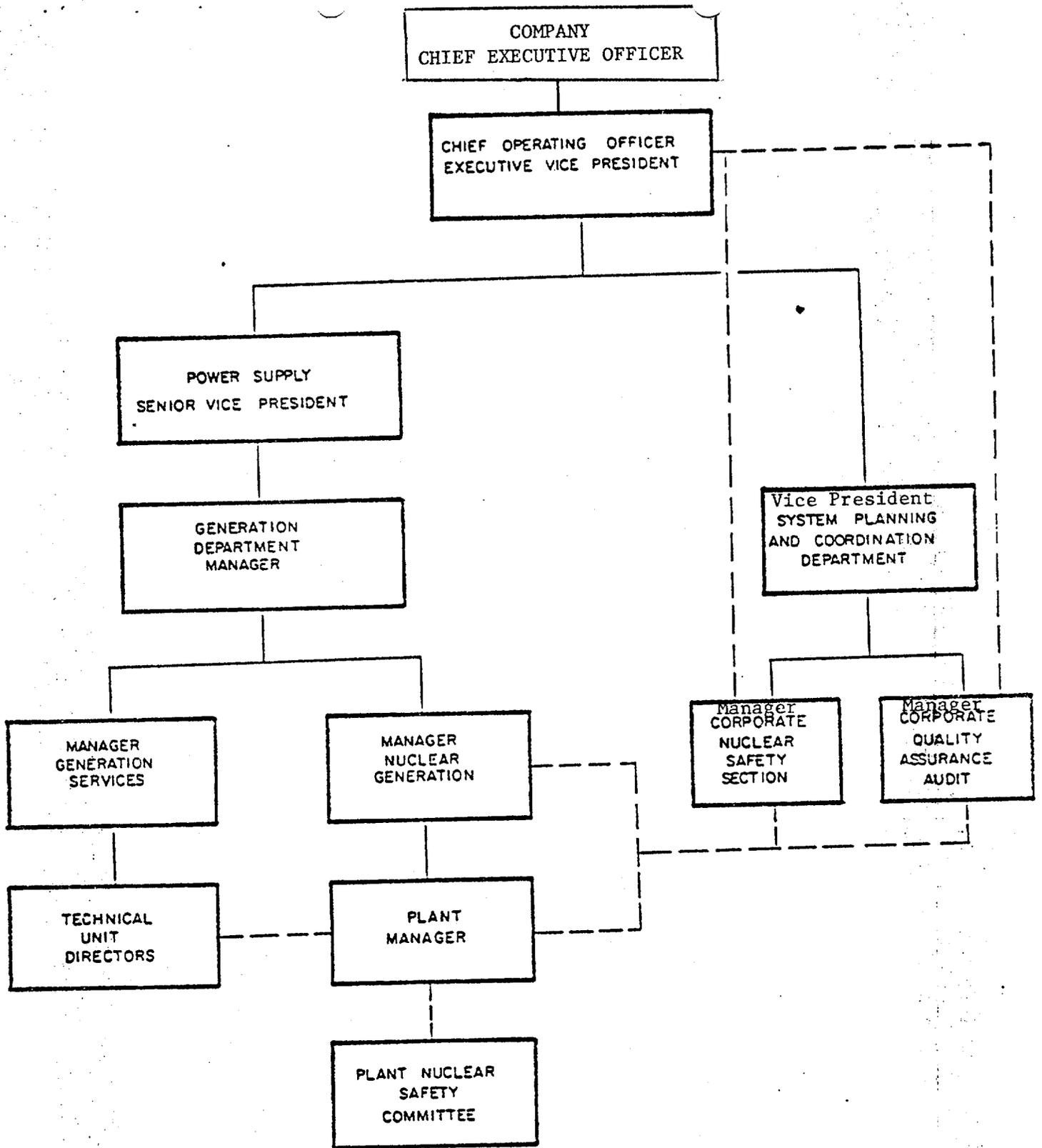
6.6.1 The following actions shall be taken in the event of a reportable occurrence:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.
- b. Each Reportable Occurrence Report shall be submitted to the Manager of Corporate Nuclear Safety and the Manager of Nuclear Generation.

6.7 Safety Limit Violation

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The provisions of 10CFR50.36(c)(1)(i) shall be complied with immediately.
- b. The Safety Limit violation shall be reported to the Commission, the Manager of Nuclear Generation and the Manager of Corporate Nuclear Safety within 24 hours.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PNSC. This report shall describe (1) applicable circumstances preceding the violation, (2) the effects of the violation upon facility components, systems of structures, and (3) corrective action taken to prevent recurrence.



————— Administrative Organization

- - - - - Lines of Communication

Figure 6.2-1



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 28 TO FACILITY OPERATING LICENSE NO. DPR-23
CAROLINA POWER & LIGHT COMPANY
H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2
DOCKET NO. 50-261

Introduction

By letter dated April 26, 1977, Carolina Power and Light Company (the licensee) requested amendment of the technical specifications appended to Facility License No. DPR-23 for the H. B. Robinson Steam Electric Plant, Unit No. 2 (Robinson-2). The proposed changes would reflect a change in the licensee's organizational structure and correct a typographical error.

Evaluation

We have reviewed the licensee's new organizational structure as defined by an organization chart included in the licensee's submittal, and have compared it with the organization which was the basis for the present wording of the technical specifications. We have identified the corresponding functional units and managers. We have compared the proposed changes in the wording of the technical specifications and find that the only significant effect is a change in titles. Functional responsibilities and the separation between and independence of operating units and review and audit units is maintained.

In the course of our review, we suggested the addition of position titles to some of the blocks shown on the new organization chart. This was suggested to clarify the relationship between the organization shown in the organization chart and the position titles used in the text of the technical specifications. We also suggested a minor revision in wording to clarify the meaning of "Operations Group," i.e. those organizations reporting to the Chief Operating Officer. The licensee has agreed with both of these modifications.

The licensee also proposed changing the reference in Specification 6.7.1.a from 10CFR40.36(c)(1)(i) to 10CFR50.36(c)(1)(i). We find that in the context of the specification the original reference was clearly a typographical error and that the proper reference is that proposed by the licensee.

Based on the foregoing, we conclude that changes proposed by the licensee are appropriate; that they will not adversely affect the quality or independence of the review and audit function and, therefore, are acceptable.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: June 9, 1977

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-261

CAROLINA POWER & LIGHT COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 28 to Facility Operating License No. DPR-23, issued to Carolina Power & Light Company (the licensee), which revised Technical Specifications for operation of the H. B. Robinson Steam Electric Plant Unit No. 2 (the facility) located in Darlington County, Hartsville, South Carolina. The amendment is effective as of its date of issuance.

The amendment changes the Technical Specifications to reflect a change in the licensee's organizational structure and to correct a typographical error.

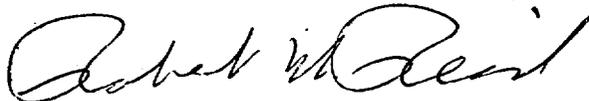
The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated April 26, 1977, (2) Amendment No. 28 to License No. DPR-23, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Hartsville Memorial Library, Home and Fifth Avenues, Hartsville, South Carolina. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 9th day of June 1977.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors