

July 16, 1976

Docket No.: 50-261

Carolina Power & Light Company
ATTN: Mr. J. A. Jones
Senior Vice President
336 Fayetteville Street
Raleigh, North Carolina 27602

Gentlemen:

The Commission has issued the enclosed Amendment No. 21 to Facility Operating License No. DPR-23 for the H. B. Robinson Steam Electric Plant Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your application dated May 13, 1976, as supplemented July 12, 1976.

This amendment clarifies the requirements for incore monitoring.

Copies of the Safety Evaluation and the Federal Register Notice are also enclosed.

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TBAbernathy
JRBuchanan

Sincerely,

Original Signed by

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Enclosures:

- 1. Amendment No. 21
- 2. Safety Evaluation
- 3. Federal Register Notice

cc w/enclosures: See next page

*subject to revision of Fed Reg. notice
per Davis note
done 7/16/76 - ri*

OFFICE →	ORB#4:DOR	ORB#4:DOR	OELD	C-ORB#4:DOR	
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DATE →	7/15/76	7/15/76	7/16/76	7/1/76	

Carolina Power & Light Company

cc w/enclosures:

G. F. Trowbridge, Esq.
Shaw, Pittman, Potts & Trowbridge
1800 M Street, N.W.
Washington, D.C. 20036

Mr. McCuen Morrell, Chairman
Darlington County Board of Supervisors
County Courthouse
Darlington, South Carolina 29532

Hartsville Memorial Library
Home and Fifth Avenues
Hartsville, South Carolina 29550

John D. Whisenhunt, Esquire
Bridges and Whisenhunt
Bridges Building
P. O. Box 26
Florence, South Carolina 29501

cc w/enclosures & incoming
dated 5/13/76 and 7/12/76

Office of Intergovernmental Relations
116 West Jones Street
Raleigh, North Carolina 27603



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20565

CAROLINA POWER & LIGHT COMPANY

DOCKET NO. 50-261

H. B. ROBINSON STEAM ELECTRIC PLANT UNIT NO. 2

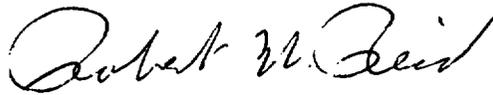
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 21
License No. DPR-23

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Carolina Power & Light Company (the licensee) dated May 13, 1976, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment.
3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Attachment:
Changes to the
Technical Specifications

Date of Issuance: July 16, 1976

ATTACHMENT TO LICENSE AMENDMENT NO. 21

FACILITY OPERATING LICENSE NO. DPR-23

DOCKET NO. 50-261

Revise Appendix A Technical Specifications as follows:

Remove Page

Insert Page

4.11-1

4.11-1

The changed area on the revised page is shown by a marginal line.

4.11

REACTOR CORE

Applicability:

Applies to surveillance of the reactor core.

Objective:

To ensure the integrity of the fuel cladding.

Specifications:

4.11.1

APDMS Operation

4.11.1.1

Prior to establishing normal operation with APDMS, at least six maps will be taken to determine applicable values of \bar{R} and σ for surveillance thimbles.

4.11.1.2

Plant operation up to full rated power shall be permitted for the purposes of obtaining the initial maps of Specification 4.11.1.1, provided the APDMS is operational and hot channel factors are shown to be below the limiting values set forth in Specification 3.10.2. Suitably conservative values of \bar{R} and σ shall be derived from maps previously ran during the current fuel cycle for use in the APDMS system during this initial period.

4.11.1.3

Subsequent update of \bar{R} and σ shall employ the last six maps run in accordance with Specification 4.11.1.1.

4.11.1.4

Each power distribution map will be based on flux traverses obtained from 36 or more of the 46 monitoring channels.

4.11.2

Except during physics tests and excore calibration, axial surveillance of $F(Z)S(Z)$ shall consist of traverses with the movable incore detectors in appropriate pairs of detector paths, taken every eight hours, or a frequency of approximately 0, 10, 30, 60, 120, 180, 240, 360, and 480 minutes following accumulated control rod motion in any one direction of five steps or more, exclusive of control rod movement within 15 steps from the top of the core. From the traverses, determination of $F(Z)S(Z)$ shall be made and shown to result in a value less than the limiting value specified in 3.10.2. If the APDMS is out of service, reactor operation above P_{APDMS} can be continued for fourteen equivalent full power days provided that traverses are taken manually at equivalent frequencies, and a log of accumulated rod motion and time of manual traverses is kept.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20565

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 21 TO LICENSE NO. DPR-23

CAROLINA POWER AND LIGHT COMPANY

H. B. ROBINSON STEAM ELECTRIC PLANT UNIT NO. 2

DOCKET NO. 50-261

Introduction

By letter dated May 13, 1976, Carolina Power and Light Company (CP&L) proposed changes to the Technical Specifications appended to Facility Operating License DPR-23 for H. B. Robinson Steam Electric Plant Unit No. 2 (Robinson 2). The proposed Technical Specification change clarifies the requirements for incore monitoring incorporated into the Technical Specifications by Amendment No. 20 to Facility Operating License DPR-23. By letter dated July 12, 1976, CP&L submitted additional information regarding this proposed change.

Discussion

Amendment No. 20 to Facility Operating License DPR-23 incorporated provisions in the Technical Specifications requiring additional incore monitoring using the Axial Power Distribution Measurement System (APDMS). The sections of the Technical Specifications affected were sections 3.10.2.1.1 through 3.10.2.1.3 and all of section 4.11.

Section 3.10.2.1.1 provides that if the value of F_{xy} for the unrodded plane of the core exceeds 1.435, the APDMS will be employed to monitor $F_Q(Z)$ above a specified power level. This same section also states that this limit is not applicable during physics tests and excore calibrations. This exception to applicability is justified in the bases for the Technical Specification.

Section 4.11.2 specifies the frequency at which axial surveillance using the APDMS shall be performed under various conditions but neglects to include the exception to applicability noted in section 3.10.2.1.1. There is therefore an inconsistency between these two sections.

Since section 3.10.2.1.1 excludes applicability of the APDMS during physics tests and excore calibrations and since the bases for the Technical Specification address the requirement for and acceptability of this exclusion, we conclude that the failure to include the exclusion in section 4.11.2 was the result of an oversight which the proposed revision of the Technical Specifications would correct. We therefore conclude that the proposed revision of the Technical Specification will provide consistency between the two related sections in the originally intended manner and is acceptable. We further conclude that the proposed change will not affect the operation or safety of the reactor.

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR 851.5(d)(4), that an environmental statement, negative declaration, or environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

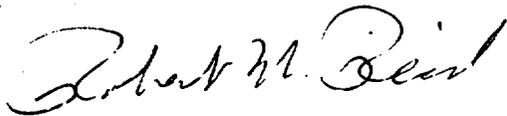
We have concluded, based on the considerations discussed above, that: (1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the change does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: July 16, 1976

July 12, 1976, (2) Amendment No. 21 to License No. DPR-23, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W. , Washington, D.C. and at the Hartsville Memorial Library, Home and Fifth Avenues, Hartville, South Carolina. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 16th day of July 1976.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief
Operating Reactors Branch # 4
Division of Operating Reactors