

JUL 06 1973

Docket No. 50-261

Carolina Power & Light Company
ATTN: E. E. Utley, Vice President
Bulk Power Supply Department
336 Fayetteville Street
Raleigh, North Carolina 27602

Change No. 21
License No. DPR-23

Gentlemen:

By letter dated May 1, 1973, you submitted an application for a change to the Technical Specifications appended to License No. DPR-23, as amended, for Unit No. 2 at the H. B. Robinson Steam Electric Plant. The proposed change would require Safety Injection System capability when the reactor is in the hot shutdown condition.

During our review of your application, we informed your staff that certain changes to your proposal and other editorial changes were necessary to meet our regulatory requirements. These changes have been made. We have designated our action as Change No. 21.

With the exception of the accumulators, Change No. 21 requires that systems related to safety be operable when the reactor is in the hot shutdown condition in order to provide protection during a steam line break accident. Availability of the accumulators is not required as indicated by the analysis of the steam line break accident in the FSAR.

We conclude that Change No. 21 does not present a significant hazards consideration and that there is reasonable assurance that the health and safety of the public will not be endangered. Pursuant to 10 CFR Part 50, Section 50.59, the Technical Specifications appended to License No. DPR-23 are changed as shown in Attachment A.

Sincerely,

Original signed by
Dennis L. Ziemann for

Donald J. Skovholt
Assistant Director for
Operating Reactors
Directorate of Licensing

Enclosure:

see next page

JUL 06 1973

Enclosure:
Attachment A - Change No. 21 to
the Technical Specifications

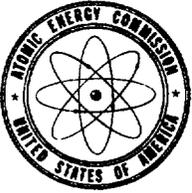
cc w/enclosure:
George F. Trowbridge, Esquire
Shaw, Pittman, Potts, Trowbridge
and Madden
910 - 17th Street, N. W.
Washington, D. C. 20006

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UNITED STATES
ATOMIC ENERGY COMMISSION
WASHINGTON, D.C. 20545

July 6, 1973

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Sincerely,

for 
Donald J. Skovholt
Assistant Director for
Operating Reactors
Directorate of Licensing

Enclosure:
see next page

Carolina Power & Light Company

- 2 -

July 6, 1973

Enclosure:

Attachment A - Change No. 21 to
the Technical Specifications

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ATTACHMENT A

CHANGE NO. 21 TO THE TECHNICAL SPECIFICATIONS

CAROLINA POWER & LIGHT COMPANY

DOCKET NO. 50-261

Section 1.0, DEFINITIONS

Paragraph 1.2.1 - Delete

Paragraph 1.2.2, Cold Shutdown Condition

Revise to read as follows:

"When the reactor is subcritical and T_{avg} is $\leq 200^{\circ}\text{F.}$ "

Paragraph 1.2.3, Hot Shutdown Condition

Revise to read as follows:

"When the reactor is subcritical and T_{avg} is $> 200^{\circ}\text{F.}$ "

Paragraph 1.2.6, Refueling Operation

Revise to read as follows:

"Any operation involving movement of core components when there is fuel in the containment vessel and the pressure vessel head is unbolted or removed."

Section 3.0, LIMITING CONDITIONS FOR OPERATION

Add the following new paragraph 3.2.5 following paragraph 3.2.4:

"3.2.5 When the reactor is in the hot shutdown condition, the requirements of 3.2.2, 3.2.3, and 3.2.4 shall be met. Except that any one component as defined in 3.2.3 may be inoperable for a period equal to the time period specified in the subparagraphs 3.2.2 plus 48 hours, after which the plant shall be placed in the cold shutdown condition utilizing normal operating procedures."

Add the following new paragraph 3.3.1.3 following paragraph 3.3.1.2:

"3.3.1.3 When the reactor is in the hot shutdown condition, the requirements of 3.3.1.1 and 3.3.1.2 shall be met. Except that the accumulators may be isolated, and in addition,

any one component as defined in 3.3.1.2 may be inoperable for a period equal to the time period specified in the subparagraphs of 3.3.1.2 plus 48 hours, after which the plant shall be placed in the cold shutdown condition utilizing normal operating procedures."

Add the following new paragraph 3.3.2.3 following paragraph 3.3.2.2:

"3.3.2.3 When the reactor is in the hot shutdown condition, the requirements of 3.3.2.1 and 3.3.2.2 shall be met. Except that any one component as defined in 3.3.2.2 may be inoperable for a period equal to the time period specified in the subparagraphs of 3.3.2.2 plus 48 hours, after which the plant shall be placed in the cold shutdown condition utilizing normal operating procedures."

Add the following new paragraph 3.3.3.3 following paragraph 3.3.3.2:

"3.3.3.3 When the reactor is in the hot shutdown condition, the requirements of 3.3.3.1 and 3.3.3.2 shall be met. Except that any one component as defined in 3.3.3.2 may be inoperable for a period equal to the time period specified in the subparagraphs of 3.3.3.2 plus 48 hours, after which the plant shall be placed in the cold shutdown condition utilizing normal operating procedures."

Add the following new paragraph 3.3.4.3 following paragraph 3.3.4.2:

"3.3.4.3 When the reactor is in the hot shutdown condition, the requirements of 3.3.4.1 and 3.3.4.2 shall be met. Except that any one component as defined in 3.3.4.2 may be inoperable for a period equal to the time period specified in the subparagraphs of 3.3.4.2 plus 48 hours, after which the plant shall be placed in the cold shutdown condition utilizing normal operating procedures."

Paragraph 3.3.5, Post Accident Containment Venting System

Basis - Add the following new paragraph after paragraph 6 (after first paragraph on page 3.3-8):

"Failure to complete repairs within the additional 48 hours requires that the reactor be put in the cold shutdown condition to provide

protection for the steam break accident. This condition is also indicative of a requirement for maintenance of such magnitude that it could not be dealt with routinely."

Paragraph 3.8

Add the following new subparagraph:

"k. The reactor shall be subcritical as required by 3.10.6.3 with $T_{avg} \leq 140^{\circ}\text{F}.$ "

Paragraph 3.10

Revise the title of the paragraph and "Applicability" and "Objective" as follows:

"3.10 REQUIRED SHUTDOWN MARGINS, CONTROL ROD, AND POWER DISTRIBUTION LIMITS

Applicability:

Applies to the required shutdown margins, operation of the control rods, and power distribution limits.

Objective

To ensure (1) core subcriticality after a reactor trip and during normal shutdown conditions, (2) limited potential reactivity insertions from a hypothetical control rod ejection, and (3) an acceptable core power distribution during power operation."

Paragraph 3.10.1.1 - Delete

Add the following new paragraphs 3.10.6 through 3.10.6.3 following paragraph 3.10.5.3:

"3.10.6 Required Shutdown Margins

3.10.6.1 When the reactor is in the hot shutdown condition, the shutdown margin shall be at least that shown in Figure 3.10-2.

- 3.10.6.2 When the reactor is in the cold shutdown condition the shutdown margin shall be at least 1% $\Delta k/k$.
- 3.10.6.3 When the reactor is in the refueling operation mode, the shutdown margin shall be at least 10% $\Delta k/k$."