

REGULATORY DOCKET FILE COPY

SEPTEMBER 11 1979

Docket No. 50-366

Mr. Charles F. Whitmer  
Vice President - Engineering  
Georgia Power Company  
P. O. Box 4545  
Atlanta, Georgia 30302

Dear Mr. Whitmer:

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The Commission has issued Amendment No. 11 to Facility Operating License No. NPF-5 for the Edwin I. Hatch Nuclear Plant, Unit No. 2. The amendment consists of administrative changes and involves (1) correction of the surveillance requirements for diesel generators to reflect the largest single load that a generator must be capable of rejecting, (2) correction of the system identification code for two seismic monitoring instruments and (3) changing from 5 to 30 days the requirement for channel calibration of seismic monitoring instruments following a seismic event. The amendment is in response to your request dated July 19, 1979.

The currently approved surveillance requirements for each diesel generator include the verification that the generator has the capability to reject a load of  $\geq 1070$  KW. The intent of this surveillance is to demonstrate proper operation of the diesel during load shedding, and the value of the load should be the largest single load. Your request stated that the design basis is the case of a Residual Heat Removal pump being tripped and that the specification, as initially issued, substituted the horsepower (1070) instead of the kilowatt rating (798). We have reviewed your request and the diesel generator system emergency loads as described in Section 8.3 of the Hatch-2 FSAR and verified that the initial specifications were in error and should be corrected.

We have reviewed your request pertaining to changes in the system identification code for two seismic monitoring instruments and determined that these changes are pro forma in nature.

The current specifications on seismic monitoring instrumentation require that a channel calibration be performed within 5 days of a seismic event. The intent of the specification is to recalibrate actuated instruments within a reasonable period of time following an event. Your request stated that the vendor who performs your calibrations has indicated they cannot perform the required calibration within the specified period of time. You requested that the 5 day limit be changed to 30 days. We have reviewed your request and determined that

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Mr. Charles F. Whitmer

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since the Limiting Conditions for Operation permit such instruments to be inoperable for 30 days, your request is justified.

In view of the above, the changes to the Technical Specifications are acceptable as proposed.

We have evaluated the potential for environmental impact of plant operation in accordance with the enclosed amendment. The amendment applies to administrative details. Therefore we have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR 51.5(f)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Since the amendment applies only to administrative details, it does not involve significant new safety information of a type not considered by a previous Commission safety review of the facility. It does not involve a significant increase in the probability or consequences of an accident, does not involve a significant decrease in a safety margin, and therefore does not involve a significant hazards consideration. We have also concluded that there is reasonable assurance that the health and safety of the public will not be endangered by this action.

A copy of a related Notice of Issuance is also enclosed.

Sincerely,

Original signed by

Thomas A. Ippolito, Chief  
Operating Reactors Branch #3  
Division of Operating Reactors

Enclosures:

- 1. Amendment No. 11 to NPF-5
- 2. Notice

cc w/enclosures:  
See next page

OFFICE	ORB#3: DOR	ORB#3: DOR	C-ORB#3: DOR	AD-ORB#3: DOR	OELD	STSG
SURNAME	PKreutzer	DVerelli/cb	TIB...	WG...	B. Sm...	PK
DATE	9/5/79	9/5/79	9/5/79	9/5/79	9/5/79	9/5/79

Mr. Charles F. Whitmer  
Georgia Power Company

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September 11, 1979

cc:

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ATTN: EIS COORDINATOR  
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Appling County Public Library  
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Baxley, Georgia 31513

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Director, Technical Assessment  
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY  
OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION  
MUNICIPAL ELECTRIC ASSOCIATION OF GEORGIA  
CITY OF DALTON, GEORGIA

DOCKET NO. 50-366

EDWIN I. HATCH NUCLEAR PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 11  
License No. NPF-5

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Georgia Power Company et al. (the licensee) dated July 19, 1979 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-5 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 11, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Thomas A. Ippolito, Chief  
Operating Reactors Branch #3  
Division of Operating Reactors

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: September 11, 1979

ATTACHMENT TO LICENSE AMENDMENT NO. 11

FACILITY OPERATING LICENSE NO. NPF-5

DOCKET NO. 50-366

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

Remove

3/4 3-47

3/4 3-48

3/4 8-3

3/4 8-4\*

Insert

3/4 3-47

3/4 3-48

3/4 8-3

3/4 8-4\*

\* Overleaf

## INSTRUMENTATION

### SEISMIC MONITORING INSTRUMENTATION

#### LIMITING CONDITION FOR OPERATION

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3.3.6.2 The seismic monitoring instrumentation shown in Table 3.3.6.2-1 shall be OPERABLE.

APPLICABILITY: At all times.

#### ACTION:

- a. With one or more of the above required seismic monitoring instruments inoperable for more than 30 days, in lieu of any other report required by Specification 6.9.1, prepare and submit a Special Report to the Commission within the next 10 days outlining the cause of the malfunction and the plans for restoring the instrument(s) to OPERABLE status.
- b. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

#### SURVEILLANCE REQUIREMENTS

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4.3.6.2.1 Each of the above required seismic monitoring instruments shall be demonstrated OPERABLE by the performance of the CHANNEL CHECK, CHANNEL FUNCTIONAL TEST and CHANNEL CALIBRATION operations at the frequencies shown in Table 4.3.6.2-1.

4.3.6.2.2 Each of the above required seismic monitoring instruments actuated during a seismic event shall be restored to OPERABLE status within 24 hours and a CHANNEL CALIBRATION performed within 30 days following the seismic event. Data shall be retrieved from actuated instruments and analyzed to determine the magnitude of the vibratory ground motion. In lieu of any other report required by Specification 6.9.1, a Special Report shall be prepared and submitted to the Commission pursuant to Specification 6.9.2 within 10 days describing the magnitude, frequency spectrum and resultant effect upon facility features important to safety.

TABLE 3.3.6.2-1

SEISMIC MONITORING INSTRUMENTATION

<u>INSTRUMENTS AND SENSOR LOCATIONS</u>	<u>MEASUREMENT RANGE</u>	<u>MINIMUM INSTRUMENTS OPERABLE</u>
1. Triaxial Time-History Accelerographs <sup>(a)</sup>		
a. Diesel Generator Building El 130'0" <sup>(c)</sup> (2L51-N021)	0-0.5g	1
b. Reactor Building 87' Level on Drywell Pedestal (2L51-N020)	0-0.5g	1
c. Drywell - Feedwater Inlet to RPV (2L51-N004)	0-0.5g	1
d. Switchyard <sup>(c)</sup> (1L51-N005)	0-0.5g	1
2. Triaxial Peak Recording Accelerometers <sup>(a)</sup>		
a. Diesel Generator Base Support <sup>(c)</sup> (1L51-N007)	0-1.0g	1
b. Intake Structure <sup>(c)</sup> (1L51-N006)	0-1.0g	1
c. Control Building Main Control Room Floor <sup>(c)</sup> (1L51-N008)	0-1.0g	1
d. Control Building Floor El 112' <sup>(c)</sup> , (2L51-N028)	0-1.0g	1
e. Reactor Bldg Refueling Floor (2L51-N029)	0-1.0g	1
f. Reactor Pedestal Inside Biological Shield (2L51-N035)	0-2.0g	1
g. Reactor Piping - Feedwater Inlet to RPV (2L51-N034)	0-2.0g	1
3. Triaxial Seismic Switches <sup>(b)</sup>		
a. Reactor Building 87' Level on Drywell Pedestal (2L51-N022)	0.025-0.25g	1
b. Reactor Building 185' Level Out- side Biological Shield (2L51-N024)	0.025-0.25g	
4. Triaxial Response Spectrum Recorder <sup>(a)</sup>		
a. Hatch - Unit 1 Containment Foundation El 87' <sup>(c)</sup> (1L51-N105)	2-26 Hz 0-0.5g	1

<sup>a</sup>With main control room indication and annunciation.

<sup>b</sup>With main control room annunciation.

<sup>c</sup>Shared with Hatch - Unit 1.

ELECTRICAL POWER SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

4.8.1.1.2 Each diesel generator shall be demonstrated OPERABLE:

- a. In accordance with the frequency specified in Table 4.8.1.1.2-1 on a STAGGERED TEST BASIS by:
  1. Verifying the fuel level in the day fuel tanks.
  2. Verifying the fuel level in the plant fuel storage tank.
  3. Verifying the fuel transfer pump can be started and transfers fuel from the storage system to the day tank.
  4. Verifying the diesel starts from ambient condition and accelerates to synchronous speed in  $\leq 12$  seconds.
  5. Verifying the generator is synchronized, loaded to 2764 kw for diesel generator 2A, 2360 kw for diesel generator 1B or 2742 kw for diesel generator 2C in  $\leq 120$  seconds, and operates for  $\geq 60$  minutes.
  6. Verifying the diesel generator is aligned to provide standby power to the associated emergency busses.
  7. Verifying the pressure in both diesel air start receivers to be  $\geq 225$  psig.
- b. At least once per 92 days by verifying that a sample of diesel fuel from the fuel storage tank, obtained in accordance with ASTM-D270-65, is within the acceptable limits specified in Table 1 of ASTM D975-74 when checked for viscosity, water and sediment.
- c. At least once per 18 months during shutdown by:
  1. Subjecting the diesel to an inspection in accordance with procedures prepared in conjunction with its manufacturer's recommendations for this class of standby service.
  2. Verifying that the automatic load sequence timer is OPERABLE with the interval between each load block within  $\pm 10\%$  of its design interval.
  3. Verifying the generator capability to reject a load of  $\geq 798$  kw while maintaining voltage at  $4160 \pm 400$  volts and frequency at  $60 \pm 2$  Hz.

## ELECTRICAL POWER SYSTEMS

### SURVEILLANCE REQUIREMENTS (Continued)

4. Verifying the generator capability to reject a load of 2764 kW for diesel generator 2A, 2360 kW for diesel generator 1B and 2742 kW for diesel generator 2C without exceeding 75% of the difference between nominal speed and the overspeed trip setpoint, or 15% above nominal, whichever is lower.
5. Simulating a loss of offsite power by itself, and:
  - a) Verifying de-energization of the emergency busses and load shedding from the emergency busses.
  - b) Verifying the diesel starts from ambient condition on the auto-start signal, energizes the emergency busses with permanently connected loads, energizes the auto-connected shutdown loads through the load sequencer and operates for  $\geq 5$  minutes while its generator is loaded with the shutdown loads.
6. Verifying that on an ECCS actuation test signal, without loss of offsite power, the diesel generator starts on the auto-start signal and operates on standby for  $\geq 5$  minutes.
7. Verifying that on a simulated loss of the diesel generator, with offsite power not available, the loads are shed from the emergency busses and that subsequent loading of the diesel generator is in accordance with design requirements.
8. Simulating a loss of offsite power in conjunction with an ECCS actuation test signal, and
  - a) Verifying de-energization of the emergency busses and load shedding from the emergency busses.
  - b) Verifying the diesel starts from ambient condition on the auto-start signal, energizes the emergency busses with permanently connected loads, energizes the auto-connected emergency (accident) loads through the load sequencer and operates for  $\geq 5$  minutes while its generator is loaded with the emergency loads.
  - c) Verifying that all diesel generator trips, except engine overspeed, low lube oil pressure and generator differential, are automatically bypassed upon loss of voltage on the emergency bus concurrent with an ECCS actuation signal.

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-366GEORGIA POWER COMPANY, ET AL.NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY  
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 11 to Facility Operating License No. NPF-5 issued to Georgia Power Company, Oglethorpe Electric Membership Corporation, Municipal Electric Association of Georgia, and City of Dalton, Georgia, which revised Technical Specifications for operation of the Edwin I. Hatch Nuclear Plant, Unit No. 2, (the facility) located in Appling County, Georgia. The amendment is effective as of the date of issuance.

The amendment consists of administrative changes to the Technical Specifications and involves (1) correction of the surveillance requirements for diesel generators to reflect the largest single load that a generator must be capable of rejecting, (2) correction of the system identification code for two seismic monitoring instruments and (3) changing from 5 to 30 days the requirement for channel calibration of seismic monitoring instruments following a seismic event.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

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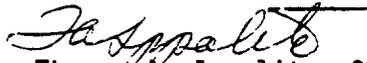
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The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated July 19, 1979, (2) Amendment No. 11 to License No. NPF-5, and (3) the Commission's letter dated September 11, 1979. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Appling County Library, Parker Street, Baxley, Georgia 31513. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 11th day of September, 1979:

FOR THE NUCLEAR REGULATORY COMMISSION



Thomas A. Ippolito, Chief  
Operating Reactors Branch #3  
Division of Operating Reactors