

NOV 9 1973

Docket No. 50-261

Carolina Power & Light Company
ATTN: Mr. E. E. Utley, Vice President
Bulk Power Supply Department
336 Fayetteville Street
Raleigh, North Carolina 27602

Change No. 26
License No. DPR-23

Gentlemen:

By letter dated October 23, 1973, you requested a change to the Technical Specifications appended to License No. DPR-23, as amended, for Unit No. 2 at the H. B. Robinson Steam Electric Plant. The proposed change would revise the setpoint given in Table 3.5-1 for the channels which monitor steam flow from the steam generators and also initiate safety injection and steam line isolation. Below 10% load, these protective actions would be delayed, and above 10% load, they would be expedited.

High steam line flow and low reactor coolant temperature are together indicative of a steam line break. As indicated in Section 14.2.5 of the Final Safety Analysis Report for Robinson-2, consequences of a steam line break accident are most significant when the reactor is hot and at zero load. If a steam line break accident should occur, safety injection and steam line isolation are necessary to limit the power transient which might result from the accident. Figures 14.2.5-3 through 14.2.5-6 in the FSAR imply that the rise time for the increase in steam flow from the no-load flow prior to the steam line break to maximum flow following the break is less than a few seconds. Increasing the high steam flow setpoint at zero load from 20% flow to 40% flow, as proposed, can delay protective action by no more than a fraction of a second. From the figures cited, we see that the effect of this delay on the power transient would be negligible.

Based on your application and the safety evaluation above, we conclude that the requested change does not present a significant hazards consideration and that there is reasonable assurance that the health and safety of the public will not be endangered.

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Pursuant to 10 CFR Part 50, Section 50.59, the Technical Specifications appended to License No. DPR-23 are changed as shown in Attachment A.

Sincerely,

Original signed by *for*
Robert J. Schemel

Donald J. Skovholt
Assistant Director for
Operating Reactors
Directorate of Licensing

Enclosure:
Attachment A - Change No. 26 to
the Technical Specifications

cc w/enclosure:
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ATTACHMENT A

CHANGE NO. 26 TO THE TECHNICAL SPECIFICATIONS

CAROLINA POWER & LIGHT COMPANY

DOCKET NO. 50-261

1. On page 3.5-4, revise Item 5 to read as follows:

"The high steam line flow limit is set at approximately 40% of the steam flow from no load to 20% load and at 110% of full steam flow at full load, with the steam flow differential pressure measurement linearly programmed between 20% load and 100% load in order to protect against large steam line break accidents⁽⁴⁾. The coincident low Tav_g setting limit for SIS and steam line isolation initiation is set below its hot shutdown value. The coincident steam line pressure setting limit is set below the full load operating pressure. The safety analysis shows that these settings provide protection in the event of a large steam line break⁽³⁾."

2. Under "Reference" on page 3.5-5, add the following:

"(4) CP&L letter to the Directorate of Licensing dated October 23, 1973."

3. In Table 3.5-1, revise No. 5 to read as follows:

<u>FUNCTIONAL UNIT</u>	<u>CHANNEL ACTION</u>	<u>SETTING LIMIT</u>
High Steam Flow in 2/3	a. Safety Injection*	< 40% (at zero load) of full steam flow
Steam Lines ***	b. Steam Line Isolation	< 40% (at 20% load) of full steam flow < 110% (at zero load) of full steam flow
Coincident with Low Tav _g or		> 541°F Tav _g
Low Steam Line Pressure		> 600 psig steam line pressure

*** Derived from equivalent AP measurements.

OFFICE ▶

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DATE ▶