

AUGUST 11 1978

Docket No. 50-366

Georgia Power Company
Oglethorpe Electric Membership Corporation
Municipal Electric Association of Georgia
City of Dalton, Georgia
ATTN: Mr. Charles F. Whitmer
Vice President - Engineering
Georgia Power Company
Atlanta, Georgia 30302

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Gentlemen:

The Commission has issued the enclosed Amendment No. 1 to Facility License No. NPF-5 for the Edwin I. Hatch Nuclear Plant Unit No. 2. This amendment consists of changes to the Technical Specifications in response to your request dated August 8, 1978.

This amendment modifies the Technical Specifications by (1) revising the surveillance requirement for measuring drywell air temperature by redefining the average temperature as a volume average instead of an arithmetical average of four sensors located at different elevations and (2) revising the surveillance requirements for the undervoltage setpoint for the Reactor Protection System instrumentation MG sets.

Copies of the related Safety Evaluation and Notice of Issuance are also enclosed.

Sincerely,

Original signed by

Thomas A. Ippolito, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Enclosures:

- 1. Amendment No. 1 to NPF-5
- 2. Safety Evaluation
- 3. Notice

cc w/enclosures:
see next page

*subject to change copy -
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SURNAME →	SSheppard	DVerrelli:acr	B. Smith	T. Ippolito	PSB
DATE →	8/10/78	8/10/78	8/11/78	8/11/78	8/10/78

Georgia Power Company
Oglethorpe Electric Membership Corporation
Municipal Electric Association of Georgia
City of Dalton, Georgia

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY
OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION
MUNICIPAL ELECTRIC ASSOCIATION OF GEORGIA
CITY OF DALTON, GEORGIA

DOCKET NO. 50-366

EDWIN I. HATCH NUCLEAR PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 1
License No. NPF-5

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Georgia Power Company, Oglethorpe Electric Membership Corporation, Municipal Electric Association of Georgia and City of Dalton, Georgia, (the licensees) dated August 8, 1978, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

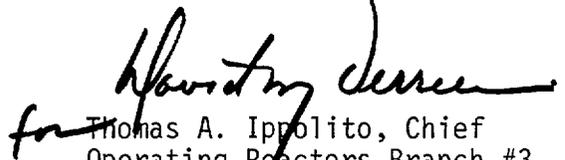
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-5 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 1, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


for Thomas A. Ippolito, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: AUGUST 11 1978

ATTACHMENT TO LICENSE AMENDMENT NO. 1

FACILITY OPERATING LICENSE NO. NPF-5

DOCKET NO. 50-366

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. Overleaf pages are provided for your convenience.

Remove

3/4 6-9
3/4 6-10
3/4 8-11
3/4 8-12

Insert

3/4 6-9
3/4 6-10
3/4 8-11
3/4 8-12

ELECTRICAL POWER SYSTEMS

SURVEILLANCE REQUIREMENTS

4.8.2.1.1 The above required A.C. distribution system buses and inverters shall be determined OPERABLE:

- a. At least once per 7 days by verifying correct breaker alignment and indicated power availability, and
- b. At least once per 31 days by determining that the 250 volt DC/600 volt AC inverters 2R44-S002 and 2R44-S003 are OPERABLE by verifying inverter output voltage of 600 volts \pm 5% while supplying their respective buses.

4.8.2.1.2 The above specified RPS instrumentation MG sets 2A and 2B shall be determined OPERABLE:

- a. At least once per 8 hours by verifying;
 1. RPS instrumentation MG sets 2A and 2B voltage to be between 108 and 132 VAC, and
 2. No unexplained change in RPS instrumentation MG set 2A and/or 2B current in excess of 5% from the value observed during the Startup Test Program.
- b. At least once per 6 months and prior to resetting the Reactor Protection System trips following a seismic event of Operational Basis Earthquake intensity, by demonstrating the OPERABILITY of RPS instrumentation MG set 2A and 2B over-voltage, under-voltage and under-frequency protective instrumentation by performance of a CHANNEL CALIBRATION including simulated automatic actuation of the protective relays, tripping logic and output circuit breakers and verifying the following setpoints;
 1. Over-voltage \leq 132 VAC,
 2. Under- voltage \geq 108 VAC, and
 3. Under- frequency \geq 57 Hz.

ELECTRICAL POWER SYSTEMS

A.C. DISTRIBUTION - SHUTDOWN

LIMITING CONDITION FOR OPERATION

3.8.2.2 As a minimum, the following A.C. distribution system buses, inverters and motor-generator (MG) sets shall be OPERABLE:

- a. Two 4160 volt Essential Buses, 2E, 2F and/or 2G,
- b. One 600 volt Essential Bus, 2C or 2D,
- c. One 120/208 volt Essential Cabinet, 2A or 2B,
- d. One 120/208 volt Instrument Bus, 2A or 2B,
- e. A.C. inverters 2R44-S002 and 2R44-S003*, and
- f. If in service, Reactor Protection System instrumentation MG sets 2A and 2B.

APPLICABILITY: CONDITIONS 4 and 5.

ACTION:

- a. With less than the above required A.C. distribution system buses and inverters OPERABLE, suspend all operations involving CORE ALTERATIONS, irradiated fuel handling, positive reactivity changes or operations that have the potential of draining the reactor vessel. The provisions of Specification 3.0.3 are not applicable.
- b. With RPS instrumentation MG set 2A and/or 2B voltage outside the range of 108 to 132 VAC, demonstrate the OPERABILITY of all equipment which could have been subjected to the abnormal voltage for all Class IE loads connected to the associated bus(es) by performance of a CHANNEL FUNCTIONAL TEST or CHANNEL CALIBRATION, as required, within 24 hours.
- c. With RPS instrumentation MG set 2A and/or 2B inoperable, restore the inoperable MG set(s) to OPERABLE status within 30 minutes or remove the inoperable MG set(s) from service.

SURVEILLANCE REQUIREMENTS

4.8.2.2 At least the above required A.C. distribution system buses, inverters and MG sets shall be determined OPERABLE per Specifications 4.8.2.1.1 and 4.8.2.1.2.

CONTAINMENT SYSTEMS

DRYWELL AVERAGE AIR TEMPERATURE

LIMITING CONDITION FOR OPERATION

3.6.1.7 Drywell average air temperature shall not exceed 135°F.

APPLICABILITY: CONDITIONS 1, 2 and 3.

ACTION:

With the drywell average air temperature > 135°F, reduce the average air temperature to within the limit with 8 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.7 The drywell average air temperature shall be the volumetric average of the temperatures at the following drywell elevations and shall be determined to be within the limit at least once per 24 hours:

	<u>Elevation</u>	<u>Azimuth</u>
a.	128-130 feet	115° or 255° or 315° or 320°
b.	162 feet	90° or 270°
c.	175 feet	90° or 270°
d.	187 feet	90° or 270°

CONTAINMENT SYSTEMS

PRIMARY CONTAINMENT INTERNAL PRESSURE

LIMITING CONDITION FOR OPERATION

3.6.1.6 Primary containment internal pressure shall not exceed 0.75 psig.*

APPLICABILITY: CONDITIONS 1, 2 and 3.

ACTION:

With the primary containment internal pressure in excess of the specified limit, restore the internal pressure to within the limit within 1 hour or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.6 The primary containment internal pressure shall be determined to be less than or equal to 0.75 psig at least once per 12 hours.

*Except when performing the test required by Specification 4.6.4.1.b.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 1 TO FACILITY OPERATING LICENSE NO. NPF-5

GEORGIA POWER COMPANY
OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION
MUNICIPAL ELECTRIC ASSOCIATION OF GEORGIA
CITY OF DALTON, GEORGIA

EDWIN I. HATCH NUCLEAR PLANT UNIT NO. 2

DOCKET NO. 50-366

I. Introduction

By letter dated August 8, 1978, Georgia Power Company proposed a change to the Technical Specifications appended to Operating License No. NPF-5 for the Edwin I. Hatch Nuclear Plant Unit No. 2. The change revises the surveillance requirement for measuring drywell air temperature by redefining the average temperature as a volume average instead of an arithmetical average of four (4) sensors located at different elevations. Also included in the amendment which this evaluation supports is a staff identified change to the surveillance requirements for the undervoltage setpoint of the Reactor Protection System (RPS) instrumentation MG sets. This staff identified change would correct an error in the specifications, as originally issued.

II. Evaluation

A. Drywell Air Temperature Surveillance

The current specification requires that in Mode 1, 2 or 3 operation, the drywell average air temperature shall not exceed 135°F. This average value is the input value for evaluating the containment structure response during LOCA conditions to ensure that the design limit is not exceeded. Accordingly, the proper average temperature is a volume weighed average rather than arithmetic average. A volumetric averaging specification has been previously approved by the staff for Brunswick Unit No. 2 (Docket No. 50-324). The licensee's request is consistent with the staff's review of the accident analysis for Hatch Unit No. 2 Operating License.

During the course of the staff's review of the licensee's submittal, we considered the effect of local temperatures higher than the 135°F limit because the environmental qualifications of safety related equipment might be affected. The temperatures experienced during the current startup testing program that are higher than 135°F are at the 175 ft. and 187 ft. elevation where the temperatures measure approximately 155 and 170°F respectively. At the other two lower elevations the measure temperatures are less than 115°F. There is no safety related equipment in locations which would be above the 135°F limit. This has been confirmed by our Office of Inspection and Enforcement. Thus, the licensee's proposed revision would not affect the environmental qualification limits of safety related equipment.

Based on the foregoing, we conclude that the licensee's proposed volumetric averaging technique is correct for the purpose of the operating limit basis and is acceptable.

B. RPS Power Supply Surveillance

The surveillance requirements for the RPS instrumentation MG sets include a verification that the undervoltage tripping logic and output circuit breakers are ≥ 96 VAC. This value is outside the limits set forth in limiting conditions for operation which specify an operating range of 108 to 132 VAC. Accordingly, the surveillance requirement is in error. The undervoltage limit should be ≥ 108 VAC. This staff identified change was discussed with the licensee. He agreed.

III. Environmental Considerations

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR Section 51.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

IV. Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: AUGUST 11 1978

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-366GEORGIA POWER COMPANY, ET ALNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 1 to Facility Operating License No. NPF-5 issued to Georgia Power Company, Oglethorpe Electric Membership Corporation, Municipal Electric Association of Georgia and City of Dalton, Georgia, which revised Technical Specifications for operation of the Edwin I. Hatch Nuclear Plant, Unit No. 2, located in Appling County, Georgia. The amendment is effective as of its date of issuance.

This amendment modifies the Technical Specifications by (1) revising the surveillance requirement for measuring drywell air temperature by redefining the average temperature as a volume average instead of an arithmetical average of four sensors located at different elevations and (2) revising the surveillance requirements for the undervoltage setpoint for the Reactor Protection System instrumentation MG sets.

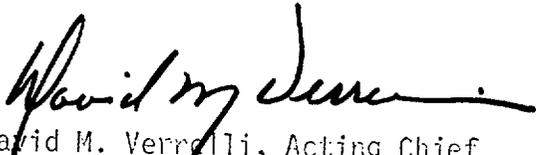
The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4), an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated August 8, 1978, (2) Amendment No. 1 to License No. NPF-5, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Appling County Public Library, Parker Street, Baxley, Georgia 31513. A single copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland this 11 day of August 1978.

FOR THE NUCLEAR REGULATORY COMMISSION


David M. Verrilli, Acting Chief
Operating Reactors Branch #3
Division of Operating Reactors