

7/5/79

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Docket No. 50-366

Mr. Charles F. Whitmer
Vice President - Engineering
Georgia Power Company
P. O. Box 4545
Atlanta, Georgia 30302

Dear Mr. Whitmer:

The Commission has issued the enclosed Amendment No. 9 to Facility License No. NPF-5 for the Edwin I. Hatch Nuclear Plant Unit No. 2. This amendment consists of changes to the Technical Specifications in response to your request dated July 2, 1979.

This amendment modifies temporarily the Technical Specifications by deleting the High Pressure Coolant Injection (HPCI) and Reactor Core Isolation Cooling (RCIC) System isolation signals on indicated high steam flow in the HPCI and RCIC steam supply lines. This temporary change will permit the conduct of the special startup testing as described in your submittal.

Copies of the related Safety Evaluation and Notice of Issuance are also enclosed.

Sincerely,

Thomas A. Ippolito, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Enclosures:

1. Amendment No. 9 to NPF-5
2. Safety Evaluation
3. Notice

cc w/enclosures:
See page 2

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Don - Check enclosed on N. copy and your answer on file

OFFICE →	ORB #3	ORB #3	AD:ORP	OELD	ORB #3	
SURNAME →	PKreutzer	DVerrelli:mj	WGammill		Tippolito	PSB
DATE →	7/3/79	7/3/79	7/3/79	7/5/79	7/3/79	P. Check 7/5/79

Mr. Charles F. Whitmer
Georgia Power Company

- 2 -

cc:

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U. S. Environmental Protection
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ATTN: EIS COORDINATOR
345 Courtland Street, N. E.
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Appling County Public Library
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Baxley, Georgia 31513

Mr. R. F. Rodgers
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Director, Technical Assessment
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US EPA
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY
OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION
MUNICIPAL ELECTRIC ASSOCIATION OF GEORGIA
CITY OF DALTON, GEORGIA

DOCKET NO. 50-366

EDWIN I. HATCH NUCLEAR PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 9
License No1 NPF-5

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Georgia Power Company, et al. (the licensee) dated July 2, 1979, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-5 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 9, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Thomas A. Ippolito, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 5, 1979

ATTACHMENT TO LICENSE AMENDMENT NO. 9

FACILITY OPERATING LICENSE NO. NPF-5

DOCKET NO. 50-366

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove

3/4 3-13
3/4 3-14

Insert

3/4 3-13
3/4 3-14

TABLE 3.3.2-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION

<u>TRIP FUNCTION</u>	<u>VALVE GROUPS OPERATED BY SIGNAL(a)</u>	<u>MINIMUM NUMBER OPERABLE CHANNELS PER TRIP SYSTEM(b)(c)</u>	<u>APPLICABLE OPERATIONAL CONDITION</u>	<u>ACTION</u>
4. HIGH PRESSURE COOLANT INJECTION SYSTEM ISOLATION				
a. HPCI Steam Line Flow -High## (2E41-N004 and 2E41-N005)	3	1	1, 2, 3	26
b. HPCI Steam Supply Pressure - Low (2E41-N001 A, B, C, D)	3, 8	2	1, 2, 3	26
c. HPCI Turbine Exhaust Diaphragm Pressure - High (2E41-N012 A,B,C,D)	3	2	1, 2, 3	26
d. HPCI Equipment Room Temperature - High (2E41-N610 A, B)	3	1	1, 2, 3	26
e. Suppression Pool Area Ambient Temperature-High (2E51-N603 C, D)	3	1	1, 2, 3	26
f. Suppression Pool Area Δ Temp.-High (2E51-N604 C, D)	3	1	1, 2, 3	26
g. Suppression Pool Area Temperature Timer Relays (2E41-M603 A, B)	3 ⁽ⁱ⁾	1	1, 2, 3	26
h. Emergency Area Cooler Temperature- High (2E41-N602 A, B)	3	1	1, 2, 3	26
i. Drywell Pressure-High (2E11-N011 C, D)	8	1	1, 2, 3	26
j. Logic Power Monitor (2E41-K1)	NA ^(h)	1	1, 2, 3	27

Not required OPERABLE during performance of the special startup test program on HPCI and RCIC reliability authorized by Amendment No. 9

HATCH - UNIT 2

3/4 3-13

Amendment No. 9

TABLE 3.3.2-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION

<u>TRIP FUNCTION</u>	<u>VALVE GROUPS OPERATED BY SIGNAL(a)</u>	<u>MINIMUM NUMBER OPERABLE CHANNELS PER TRIP SYSTEM(b)(c)</u>	<u>APPLICABLE OPERATIONAL CONDITION</u>	<u>ACTION</u>
5. <u>REACTOR CORE ISOLATION</u>				
<u>COOLING SYSTEM ISOLATION</u>				
a. RCIC Steam Line Flow-High## (2E51-N017, 2E51-N018)	4	1	1, 2, 3	26
b. RCIC Steam Supply Pressure - Low (2E51-N019 A, B, C, D)	4, 9	2	1, 2, 3	26
c. RCIC Turbine Exhaust Diaphragm Pressure - High (2E51-N012 A, B, C, D)	4	2	1, 2, 3	26
d. Emergency Area Cooler Temperature - High (2E51-N602 A, B)	4	1	1, 2, 3	26
e. Suppression Pool Area Ambient Temperature-High (2E51-N603 A, B)	4	1	1, 2, 3	26
f. Suppression Pool Area Δ T-High (2E51-N604 A, B)	4	1	1, 2, 3	26
g. Suppression Pool Area Temperature Timer Relays (2E51-M602 A, B)	4(i)	1	1, 2, 3	26
h. Drywell Pressure - High (2E11-N011 A, B)	9	1	1, 2, 3	26
i. Logic Power Monitor (2E51-K1)	NA(h)	1	1, 2, 3	27
6. <u>SHUTDOWN COOLING SYSTEM ISOLATION</u>				
a. Reactor Vessel Water Level-Low (2B21-N017 A, B, C, D)	2, 5, 6, 10, 11, 12	2	3, 4, 5	26
b. Reactor Steam Dome Pressure-High (2B31-N018 A, B)	11	1	1, 2, 3	28

Not required OPERABLE during performance of the special startup test program on HPCI and RCIC reliability authorized by Amendment No. 9

HATCH - UNIT 2

3/4 3-14

Amendment No. 9



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 9 TO FACILITY OPERATING LICENSE NO. NPF-5

GEORGIA POWER COMPANY
OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION
MUNICIPAL ELECTRIC ASSOCIATION OF GEORGIA
CITY OF DALTON, GEORGIA

EDWIN I. HATCH NUCLEAR PLANT UNIT NO. 2

DOCKET NO. 50-366

Introduction

By letter dated July 2, 1979 (Reference 1), Georgia Power Company (the licensee) proposed a change to the Technical Specifications appended to Operating License No. NPF-5 for the Edwin I. Hatch Nuclear Plant Unit No. 2. The amendment would modify temporarily the Technical Specifications by deleting the High Pressure Coolant Injection (HPCI) and Reactor Core Isolation Cooling (RCIC) Systems automatic isolation signals on indicated high steam flow in the HPCI and RCIC steam supply lines. This temporary change will permit the conduct of the special startup testing as requested by us in Reference 2 and described in the licensee's submittal.

Background

On June 3, and 27, 1979, the Hatch Unit No. 2 HPCI and RCIC systems failed to perform as designed following reactor trips. Following the latter occurrence, the Commission's Office of Inspection and Enforcement, Region II, issued a letter (Reference 2) to the licensee confirming the actions to be taken by the licensee as they relate to these HPCI and RCIC System failures. The licensee's actions were to include, among others:

- * Investigate the HPCI and RCIC System failures and take corrective actions as necessary
- * Determine the cause of failures
- * Establish comprehensive retesting requirements which verify that the HPCI and RCIC systems perform as designed.

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The licensee's request for a temporary change to the Technical Specifications was submitted to permit the conduct of a special startup testing program of the RCIC and HPCI systems to demonstrate adequate performance.

Evaluation

The HPCI and RCIC turbine driven pumps are used to provide water to the core under various conditions. The steam lines which provide the turbine steam contain two normally open containment isolation valves to minimize reactor coolant loss and radioactive materials release from the nuclear steam process barrier in the event of a gross leak or rupture of the line. The HPCI and RCIC steam line isolation function can be initiated by a number of abnormal conditions in their respective equipment rooms or piping. These diverse isolation signals which are redundant within themselves include:

- a. High room ambient temperature,
- b. High suppression pool area ambient temperature,
- c. High suppression pool area differential temperature,
- d. Emergency area cooler high temperature,
- e. High steam flow,
- f. Low steam line pressure.

This variety of signals provides protection against both small and large steam leaks in the supply lines for either the HPCI or RCIC system. For small breaks, the high steam line flow isolation function is not required. The isolation functions, which have been assumed in the analyses of the small HPCI and RCIC steam line breaks, are high area air temperature (a through d above). Therefore the temporary deletion of the high steam line flow isolation function does not affect the consequences of the small HPCI or RCIC steam line breaks.

For large HPCI and RCIC steam line breaks the analysis assumes an isolation at 13 seconds based on the concurrent loss of all off-site power, i.e., isolation cannot occur until diesel generator power is available. The isolation signals for this type event are principally the high steam flow (>300%) and low steam supply pressure. Each of these signals is redundant. That is, if one low steam supply pressure isolation signal fails, there is another which would initiate automatic isolation. This assures isolation of any large break in the systems steam supply line. Therefore, the consequences of any "large" steam line break are not affected by the temporary bypassing of the high steam flow isolation signal. Further, the deletion of the high steam line flow isolation function does not affect the probability of occurrence of any transient or accident.

On the bases of the above, the proposed modification does not involve an unreviewed safety question and is, therefore, acceptable.

The licensee has proposed a compensatory measure for the purpose of conducting this special test program; i.e., personnel will be stationed at each HPCI and RCIC local panel. The sole function of these personnel will be to observe the HPCI and RCIC steam line flow indication (as well as other available indications) and to notify the control room operator in the event of the persistence of a high steam line flow. The licensee has established an allowable time for high steam line flow which is consistent with instrument response time (13 seconds), i.e., if a flow rate, which is greater than 300% of normal steady state for test conditions, persists for 13 seconds or more, the operator will be notified. Thus, the operator will have the capability to isolate the steam line in the event of a break. This adds another degree of protection to break mitigation. It does not affect the consequence or probability of occurrence of any accident or transient and is therefore considered acceptable although credit can not be taken for this operator action in the event.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR Section 51.5(d)(4) that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of the amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: July 5, 1979

References

1. Letter from C. F. Whitmer, Georgia Power Company, GPC, to United States Nuclear Regulatory Commission, USNRC, "HPCI and RCIC Special Test Program", dated July 2, 1979.
2. Letter from J. P. O'Reilly (USNRC) to J. H. Miller, Jr., (GPC) dated June 28, 1979.

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-366GEORGIA POWER COMPANY, ET AL.NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 9 to Facility Operating License No. NPF-5 issued to Georgia Power Company, Oglethorpe Electric Membership Corporation, Municipal Electric Association of Georgia and City of Dalton, Georgia, which revised Technical Specifications for operation of the Edwin I. Hatch Nuclear Plant, Unit No. 2, located in Appling County, Georgia. The amendment is effective as of its date of issuance.

This amendment modifies temporarily the Technical Specifications by deleting the High Pressure Coolant Injection (HPCI) and Reactor Core Isolation Cooling (RCIC) System isolation signals on indicated high steam flow on the HPCI and RCIC steam supply lines. This temporary change will permit the conduct of the special startup testing as described in your submittal.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4), an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated July 2, 1979, (2) Amendment No. 9 to License No. NPF-5, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C., and at the Appling County Public Library, Parker Street, Baxley, Georgia 31513. A single copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland this 5th day of July 1979.

FOR THE NUCLEAR REGULATORY COMMISSION


Thomas A. Appolito, Chief
Operating Reactors Branch #3
Division of Operating Reactors