

JULY 26 1979

Docket No. 50-366

Mr. Charles F. Whitmer
Vice President - Engineering
Georgia Power Company
Post Office Box 4545
Atlanta, Georgia 30302

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Dear Mr. Whitmer:

The Commission has issued the enclosed Amendment No. 10 to Facility License No. NPF-5 for the Edwin I. Hatch Nuclear Plant Unit No. 2. This amendment consists of changes to the Technical Specifications in response to your request dated July 23, 1979.

This amendment modifies the Technical Specifications by increasing temporarily the limiting condition for operation on Drywell Average Air Temperature from 135°F to 145°F.

Copies of the Safety Evaluation and Notice of Issuance are also enclosed.

Sincerely,

Original Signed by
L. A. Ippolito

REGULATORY DOCKET FILE COPY

Thomas A. Ippolito, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Enclosures:

- 1. Amendment No. 10 to NPF-5
- 2. Safety Evaluation
- 3. Notice

cc w/enclosures:
see next page

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DATE	7/26/79	7/25/79	7/25/79	7/25/79	7/25/79	7/25/79

Mr. Charles F. Whitmer
Georgia Power Company

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cc:

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U. S. Environmental Protection
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Director, Technical Assessment
Division
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US EPA
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY
OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION
MUNICIPAL ELECTRIC ASSOCIATION OF GEORGIA
CITY OF DALTON, GEORGIA

DOCKET NO. 50-366

EDWIN I. HATCH NUCLEAR PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 10
License No. NPF-5

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Georgia Power Company, et al. (the licensee) dated July 23, 1979, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-5 is hereby amended to read as follows:

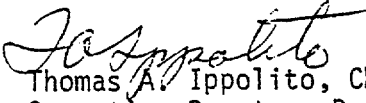
(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 10, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

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3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Thomas A. Ippolito, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 26, 1979

ATTACHMENT TO LICENSE AMENDMENT NO. 10

FACILITY OPERATING LICENSE NO. NPF-5

DOCKET NO. 50-366

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. Overleaf pages are provided for your convenience.

Remove

3/4 6-9
3/4 6/10

Insert

3/4 6-9
3/4 6-10

CONTAINMENT SYSTEMS

PRIMARY CONTAINMENT INTERNAL PRESSURE

LIMITING CONDITION FOR OPERATION

3.6.1.6 Primary containment internal pressure shall not exceed 0.75 psig.*

APPLICABILITY: CONDITIONS 1, 2 and 3.

ACTION:

With the primary containment internal pressure in excess of the specified limit, restore the internal pressure to within the limit within 1 hour or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.6 The primary containment internal pressure shall be determined to be less than or equal to 0.75 psig at least once per 12 hours.

*Except when performing the test required by Specification 4.6.4.1.b or the Special Startup Test authorized by Amendment No. 2.

CONTAINMENT SYSTEMS

DRYWELL AVERAGE AIR TEMPERATURE

LIMITING CONDITION FOR OPERATION

3.6.1.7 Drywell average air temperature shall not exceed 135°F*.

APPLICABILITY: CONDITIONS 1, 2 and 3.

ACTION:

With the drywell average air temperature > 135°F*, reduce the average air temperature to within the limit with 8 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.7 The drywell average air temperature shall be the volumetric average of the temperatures at the following drywell elevations and shall be determined to be within the limit at least once per 24 hours:

<u>Elevation</u>	<u>Azimuth</u>
a. 128-130 feet	115° or 255° or 315° or 320°
b. 162 feet	90° or 270°
c. 175 feet	90° or 270°
d. 187 feet	90° or 270°

* From July 26, 1979 until the end of the first refueling outage, a limit of 145°F applies.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 10 TO FACILITY OPERATING LICENSE NO. NPF-5

GEORGIA POWER COMPANY
OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION
MUNICIPAL ELECTRIC ASSOCIATION OF GEORGIA
CITY OF DALTON, GEORGIA

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 2

DOCKET NO. 50-366

Introduction

By letter dated July 23, 1979, Georgia Power Company (the licensee) proposed a change to the Technical Specifications appended to Operating License No. NPF-5 for the Edwin I. Hatch Nuclear Plant Unit No. 2. The change increases the Limiting Condition for Operation on Drywell Average Air Temperature from 135°F to 145°F. The change was requested because the licensee has encountered difficulty in maintaining the volumetric average temperature below this limit, particularly during the summer season.

Evaluation

The current Technical Specifications require that in Mode 1, 2 or 3 operation, the drywell average air temperature shall not exceed 135°F. This value is one of the assumed initial conditions for evaluating the containment response to a LOCA to ensure that the structure's design limits are not exceeded. The licensee's analysis of the effect of the 10°F increase in drywell average air temperature would result in a short-term post-LOCA containment pressure increase of 1 psi or less.

We have reviewed the licensee's request as well as the initial analysis of containment response to a design basis accident as described in Section 6.2 of the Hatch Unit No. 2 FSAR. We have previously verified the analytical results of the licensee's model, (see NUREG-0411), and determined that an input value of 135°F drywell air temperature yields a calculated peak drywell pressure of 57.5 psig. Thus an input value of 145°F would yield a maximum containment pressure of <58.5 psig, which is still less than the ASME Code allowable pressure of 62 psig.

The licensee's submittal also addressed the effect of local temperatures on the environmental qualifications of safety related equipment which might be affected by the proposed 145°F limit. His submittal stated that he has established regional temperature limits based on equipment qualifications. In discussions with the licensee he indicated that these limits are lower than those for which the equipment is qualified for normal operation.

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Further, he stated that temperature will be monitored and maintained to assure a satisfactory long-term environment for components within the drywell.

We have reviewed the licensee's discussion of environmental qualification of equipment as well as data on temperature profiles within the drywell, as discussed in the Staff's Safety Evaluation supporting Amendment No. 1 to NPF-5. Since the Technical Specification on average drywell temperature is a volumetric average, the effect of a 10°F average increase on local temperatures within the drywell will not be linear. Thus, local temperatures may increase on the order of 20-30°F at certain locations. The effect of increased temperatures over a long period of time would accelerate aging of safety related equipment. Since data on local temperatures is not available, we suggested to the licensee that the increased limit of 145°F average drywell air temperature be permitted only until the first refueling outage of Hatch Unit No. 2. It is our judgement that the effect of the increased temperature limit until the first refueling outage would have negligible effect on aging, since the actual elevated temperatures will occur only during seasons of especially hot weather, i.e., only a few days per year. This temporary change would thus permit the licensee to continue operation of the facility while concurrently designing and performing appropriate modifications to reduce the air temperature in the drywell. He agreed.

In view of the foregoing we find the licensee's request as modified by the staff to be acceptable. The acceptability is based on the calculated peak containment pressure being within Code allowable and the insignificant effect of increased temperature on aging of equipment during the few days of especially hot weather that will be experienced.

Environmental Considerations

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR Section 51.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: July 26, 1979

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-366GEORGIA POWER COMPANY, ET AL.NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 10 to Facility Operating License No. NPF-5 issued to Georgia Power Company, Oglethorpe Electric Membership Corporation, Municipal Electric Association of Georgia and City of Dalton, Georgia, which revised Technical Specifications for operation of the Edwin I. Hatch Nuclear Plant, Unit No. 2, located in Appling County, Georgia. The amendment is effective as of its date of issuance.

This amendment modifies the Technical Specifications by increasing temporarily the limiting conditions for operation on Drywell Average Air Temperature from 135°F to 145°F.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4), an environmental impact statement or negative declaration

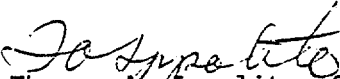
- 2 -

and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated July 23, 1979, (2) Amendment No. 10 to License No. NPF-5, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Appling County Public Library, Parker Street, Baxley, Georgia 31513. A single copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 26th day of July 1979.

FOR THE NUCLEAR REGULATORY COMMISSION


Thomas A. Ippolito, Chief
Operating Reactors Branch #3
Division of Operating Reactors